

January 17, 1990

Docket No. 50-311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: CONTAINMENT INTEGRATED LEAK RATE TESTING TEST INTERVAL RELAXATION
(TAC NO. 75127)

RE: SALEM GENERATING STATION, UNIT NO. 2

The Commission has issued the enclosed Amendment No. 85 to Facility Operating License No. DPR-75 for the Salem Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated September 25, 1989.

This amendment allows a one-time change to the surveillance interval for the conduct of containment integrated leak rate testing from 40 ±10 months to the 10-year inservice inspection outage (Cycle 6), about 9 months beyond the 50-month limit.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 85 to
License No. DPR-75
2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION w/enclosures:

Docket File MO'Brien (2)
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CMcCracken

JDyer
JLinville
BBoger
WButler
GHill (4)

[75127/AM]

PDI-2/PA
MO'Brien
12/21/89

PDI-2/PM
JStone:mj
12/19/89

OGC
BMB
01/03/89

PDI-2/D
WButler
1/17/89

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OGC Circumstance SUBJECT TO
INCLUSION OF 2 NITRO ADDITIONS TO
SE AND Amendment NOT TO ISSUE PRIOR TO EXPIRATION
OF 30 DAY
FOR NOTICE PERIOD

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[75127/AM]

PDI-2/LA
MO'Brien
1/17/89

PDI-2/PM
JStone:mj
12/19/89

OGC

PDI-2/D

WButler

01/05/89

1/17/89

OGC Considered Subject To
Inclusion of 2 NRCP Amendments To
SE and Amendment No. 75 Issued Prior To EARL



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

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2. Safety Evaluation

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Mr. Steven E. Miltenberger
Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

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Maryland People's Counsel
American Building, 9th Floor
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Baltimore, Maryland 21202

Public Service Commission of Maryland
Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC-SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY-OPERATING LICENSE

Amendment No. 85
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 25, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 17, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

3/4 6-2

Insert Page

3/4 6-2

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of less than or equal to L_a , 0.10 percent by weight of the containment air per 24 hours at design pressure (47.0 psig).
- b. A combined leakage rate of less than or equal to 0.60 L_a for all penetrations and valves subject to Type B and C tests as identified in Table 3.6-1, when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding 0.75 L_a , or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding 0.60 L_a , restore the overall integrated leakage rate to less than or equal to 0.75 L_a and the combined leakage rate for all penetrations subject to Type B and C tests to less than or equal to 0.60 L_a prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals* during shutdown at design pressure (47.0 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

* The test interval for conducting a Type A test shall be extended from 40 ± 10 months to allow the third Type A test, within the first 10-year service period, to be completed during the 10-year plant in-service inspection outage (6th refueling outage). This extension expires upon completion of the 6th refueling outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. DRP-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NO. 2

DOCKET NO. 50-311

1.0 INTRODUCTION

By letter dated September 25, 1989, Public Service Electric & Gas Company requested an amendment to Facility Operating License No. DPR-75 for the Salem Generating Station, Unit No. 2. The proposed amendment would revise the technical specifications (TS) to extend the test interval for the last of three containment integrated leak rate tests (CILRT) for Salem 2 from 40 \pm 10 months to the 10-year inservice inspection outage (Cycle 6), a period of about 59 months.

2.0 EVALUATION

Paragraph III.D.1(a) of Appendix J states "After the preoperational leakage rate test, a set of three Type A tests shall be performed at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections." In addition to the above quoted Appendix J requirement, Section 4.6.1.2.a of the Salem Unit 2 TS also specifies that "Three Type A tests shall be conducted at 40 \pm 10 month intervals during shutdown at design pressure (47 psig) during each 10 year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection (ISI)."

In the September 25, 1989 submittal, the licensee stated that the third Type A test would be required during the next refueling outage of Salem Unit 2 (i.e., Cycle 5, from 3-31-90 to 5-14-90) in order to prevent exceeding the 50-month limit on the TS maximum interval between Type A tests. However, subsequent testing during the Cycle 6 outage would violate the TS minimum 30-month test interval requirement. The plant's 10-year ISI outage (Cycle 6) is presently scheduled for 9-21-91 to 11-24-91. Therefore, the licensee proposed to postpone the CILRT to allow the third Type A test to be completed during the plant 10-year ISI outage (i.e., Cycle 6 refueling outage). The licensee provided the following justification to support its proposed amendment:

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1. The intent of the established testing intervals is to conduct three equally spaced Type A tests in a 10-year inservice period with the third test performed in conjunction with the 10-year inservice inspection outage.
2. The previous two Type A tests were completed successfully. There have not been any plant modifications which would impact containment integrity since the last Type A test.
3. Previous local leakage tests (Type B and C tests) have been completed satisfactorily and on schedule. Operability of the containment isolation valves and penetrations as required by current plant TS would continue to ensure containment integrity during an extended test interval.
4. This is a one-time Type A test interval extension request. A new Type A test schedule will be preplanned for the next 10 years.

The staff has reviewed the licensee's submittal and finds that the first CILRT for Salem Unit 2 was conducted on a shortened interval (about 21 months from the date of commercial operation). This necessitated the licensee's request for extending the third CILRT interval in order to coincide with the plant's 10 year ISI schedule. Since the licensee has planned to conduct the third CILRT during the scheduled shutdown for the 10 year ISI and because the licensee has justified the leaktight integrity of the containment based on previous leakage test results, the staff concludes that a one-time delay of approximately 9 months beyond the maximum permitted test interval will not, for reasons stated above, have a significant safety impact. The staff, therefore, concludes that the licensee's requested test interval exemption and the associated TS changes for delay in conducting the third CILRT are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (54 FR 51261) on December 13, 1989 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: J. S. Guo

Dated: January 17, 1990