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October 15, 1982

Docket No. 50-311

Mr. Richard A. Uderitz, Vice President -
 Nuclear
 Public Service Electric and Gas Company
 Mail Code T15A - P. O. Box 570
 Newark, New Jersey 07101

Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your request dated October 7, 1982.

The amendment revises the Technical Specifications to grant a one-time deferral of a pressure drop test of this unit's Containment Pressure Vacuum Relief Isolation Valves.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

for Joseph D. Neighban

William J. Ross, Project Manager
 Operating Reactors Branch No. 1
 Division of Licensing

Enclosures:

1. Amendment No. 12 to DPR-75
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page

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*FR NOTICE
 (AMENDMENT)*

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated October 7, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 12, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 15, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

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Insert Page

3/4 6-15

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.
- c. Verifying that on a feedwater isolation test signal, each feedwater isolation valve actuates to its isolation position.
- d. Verifying that on a Containment Purge and Pressure-Vacuum Relief isolation test signal, each Purge and Pressure-Vacuum Relief valve actuates to its isolation position.

4.6.3.3 At least once per 18 month, verify that on a main steam isolation test signal, each main steam isolation valve specified in Table 3.6-1 actuates to its isolation position.

4.6.3.4 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.5 Each containment purge isolation valve shall be demonstrated OPERABLE within 24 hours after each closing of the valve, except when the valve is being used for multiple cyclings, then at least once per 72 hours, by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Specification 4.6.1.2d. for all other Type B and C penetrations, the combined leakage rate is less than or equal to $0.60L_a$.

4.6.3.6 A pressure drop test to identify excessive degradation of resilient valve seals shall be conducted on the :

- a. Containment Purge Supply and Exhaust Isolation Valves at least once per 6 months.
- b.* Containment Pressure - Vacuum Relief Isolation Valves at least once per 3 months.

*The provision of Surveillance Requirement 4.6.3.6.b is not applicable during the remainder of the first fuel cycle.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 12 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

DOCKET NO. 50-311

Introduction

By letter dated October 7, 1982, Public Service Electric and Gas Company (the licensee) requested an amendment to License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. This change would defer a surveillance test for vacuum relief isolation valves until the first refueling outage.

Discussion

There has been recent concern over the use of containment isolation valves that incorporate resilient seals. Resilient seals used in containment purge and ventilation valves have been known to crack thus bringing into question their leak tightness and overall containment integrity.

As a result of these concerns, additional surveillance requirements have been added to the Salem Unit 2 Technical Specifications. Pressure drop tests (which are conducted by pressurizing the space between the two isolation valves and measuring the pressure decay) are required to be performed on the purge supply and exhaust isolation valves and the vacuum relief isolation valves. These tests are required to be performed every six months and three months, respectively. Although these valves are subject to volumetric leak testing during each refueling outage as required by Appendix J to 10 CFR 50, the pressure drop tests are designed to provide additional assurance of the valves integrity between Appendix J leak tests.

Vacuum relief isolation valves 2VC 5 & 6 have their test connections for performing the pressure drop test inside containment. The test connections are located approximately 20-25 feet above the nearest floor and are only accessible by climbing and walking over safety related cable trays. After performing their first pressure drop test for these valves on June 24, 1982, the licensee determined that undue risks were being taken by plant personnel. Therefore, a design change was made to relocate the test connections to a safely accessible area outside containment.

Relocation of the vacuum relief valves test connections are scheduled for the refueling outage of January 1983. Since testing of these valves is required by October 16, 1982, the licensee has requested not to perform Surveillance Requirement 4.6.3.6.b for the duration of the first fuel cycle.

Evaluation

Vacuum relief isolation valves 2VC 5 & 6 were successfully leak tested during preoperational testing on February 19, 1981. The pressure drop test of June 24, 1982, showed no increase in leakage thus verifying the integrity of the resilient seals.

Performing the pressure drop test before the refueling outage would require a containment entry. Access to the test connections would require personnel to climb and walk on safety related cable trays. We concur with the licensee that climbing to the ceiling with a test rig is a hazard to both test personnel and plant operation. Performing the test with the plant at power would also increase the dose rate to plant personnel. (Access by ladder is ruled out because both containment air-lock doors would have to be opened thus violating containment integrity.)

Previous leak tests performed on the vacuum relief isolation valves show that the valves' integrity has been maintained. We concur with the licensee that undue risks would be taken by plant personnel for the performance of this test. The licensee's request of October 7, 1982, to waive Surveillance Requirement 4.6.3.6.b for the duration of the first fuel cycle would only delete one pressure drop test for these valves. Installation of the more accessible, outside test connections during the January 1982 refueling outage will allow for the safe resumption of these tests.

Therefore, based on our review of the licensee's submittal, we conclude that the proposed Technical Specification change is acceptable. The change is a one-time change to exempt Surveillance Requirement 4.6.3.6.b for the duration of the first fuel cycle.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 15, 1982

Principal Contributor:
Doug Pickett

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-311PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 2 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications to grant a one-time deferral of a pressure drop test of this unit's Containment Pressure-Vacuum Relief Isolation Valves.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

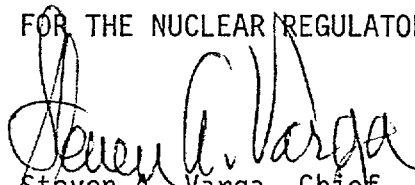
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 7, 1982, (2) Amendment No. 12 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 15th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing