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October 22, 1982 C. Parrish

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D. Brinkman

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Docket No. 50-311

Mr. Richard A. Uderitz
Vice President - Nuclear
Public Service Electric and Gas Company
Mail Code T15A - P. O. Box 570
Newark, New Jersey 07101

Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No. 14 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit No. 2. The amendment adds a new license condition in response to your request dated September 17, 1982.

The amendment authorizes deferral of certain 18-month Technical Specification surveillance items until the first refueling outage. These items are described in the enclosed Safety Evaluation.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

William J. Ross, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 14 to DPR-75
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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OFFICE	ORB 1	ORB	ORB 1	AD:OR:DL	OELD	B:DL	
SURNAME	CParrish	WRoss	SVarga	GLainas	M. K. [Signature]	DEisenhut	
DATE	10/8/82	10/8/82	10/22/82	10/12/82	10/18/82	10/5/82	

*FR NOTICE
&
AMENDMENT*

Mr. R. A. Uderitz
Public Service Electric and Gas Company

cc: Mark J. Wetterhahn, Esquire
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M.H.B. Technical Associates
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San Jose, California 95125



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 17, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the Facility Operating License No. DPR-75 is hereby amended by renumbering Paragraph 2.L to read 2.M and by adding a new Paragraph 2.L. to read as follows:

L. The licensee is authorized to defer certain eighteen-month surveillance items from the dates required by Technical Specification 4.0.2(a). These surveillances shall be completed prior to startup following the first refueling outage. The provisions of Technical Specification 4.0.2(b) are not changed. The affected items are identified in the Safety Evaluation accompanying this license change.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Date of Issuance: October 22, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 14

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Facility Operating License No. DPR-75 as follows:

Remove Page

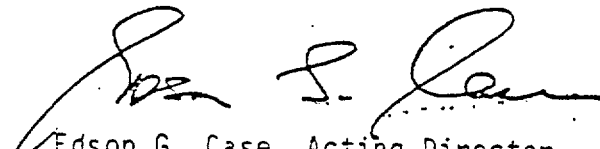
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- H. If PSE&G plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the Salem Nuclear Generating Station, the NRC shall be notified in writing regardless of whether the change affects the amount of radioactivity in effluents.
- I. PSE&G shall report any violations of the requirements contained in Section 2, Items C.(3) through C.(25), E., F., and G of this license within 24 hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the Director of the Regional Office, or his designee, no later than the first working day following the violation, with a written followup report within 14 days.
- J. The licensees shall immediately notify the Commission of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- K. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- L. The licensee is authorized to defer certain eighteen-month surveillance items from the dates required by Technical Specification 4.0.2(a). These surveillances shall be completed prior to startup following the first refueling outage. The provisions of Technical Specification 4.0.2(b) are not changed. The affected items are identified in the Safety Evaluation accompanying this license change.
- M. This license is effective as of the date of issuance and shall expire at midnight September 25, 2008.

FOR THE NUCLEAR REGULATORY COMMISSION



Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation

Attachment:
Appendices A & B

Date of Issuance: May 20, 1981

Amendment No. 14



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

DOCKET NO. 50-311

Introduction

By letter dated September 17, 1982, Public Service Electric & Gas Company (the licensee) requested an amendment to the Facility Operating License No. DPR-75 for Salem Nuclear Generating Station - Unit 2. This amendment would provide a one-time deferral of certain 18 month surveillance items as required by the plant Technical Specifications. The situation in which the licensee finds itself results from the 13 month time period between the Low Power Testing License and receipt of the Full Power License. During 8 months of this period the Unit was in cold shutdown.

The letter of September 17, 1982 included a list of items which would require action by January 22, 1983, the date of the first cycle refueling. The list is attached to this safety evaluation and includes dates by which these actions, primarily instrument calibrations, are to be performed. The dates shown encompass the 25% allowance permitted under Section 4.0.2 of the Technical Specifications. As such, the request for relief will extend the surveillance periods to as much as 15% over the 25% allowance. The surveillances are of the type that would require an extensive power outage to perform and Unit 2 has not experienced such since full-power operation.

Evaluation

Plant Technical Specifications include surveillance requirements for systems and components and specifies the frequency at which they are to be performed. The longest time interval allowed for fulfilling the surveillance requirement is 18 months. In addition, an allowance is provided such that any surveillance may exceed the specified interval by 25%. The 18 month surveillance interval was established on the basis that it would envelope a normal fuel cycle. The majority of surveillance requirements which are specified to be completed on an 18 month schedule are items for which it is more practical, as well as safe, to conduct when the plant is in a shutdown condition. In a few cases, such as overall system tests, the surveillance requirements can only be performed when the plant is in a shutdown condition. Further, the calibration of instrument transmitters on a refueling outage frequency is consistent with manufacturer recommendations. Thus, for the majority of the 18 month surveillance requirements, the basis is that this frequency contributes to the overall plant safety.

All of the items listed in the attached table have been reviewed. All of these items, except for the Containment Sump Level and the timing of the 4KV circuit breakers, are subjected to more frequent surveillance tests to ensure their operability during plant operation. The Containment Sump Level is used as one of several methods to detect leaks within containment, and therefore, can be compared against the other leak detection techniques to ensure continuous operability. The 4KV breakers have been historically satisfactory.

In order to evaluate the request for deferral of the required surveillances, several considerations were addressed. Each item was reviewed as to its function, importance to safety and ability for verification by other means. For those primary Reactor Protection System (RPS) and Engineered Safety Features Actuation System (ESFAS) items, the reduction in the setpoint analysis margin was evaluated.

For primary trip functions, instrument errors are one of many considerations that are used to establish trip setpoints. Calibrations of transmitters limits the magnitude of the errors which impact on the overall system accuracy. The transmitter errors are identified as sensor drift in the setpoint analyses. Since sensor drift is based upon transmitter performance for an 18 month interval, the impact of a longer calibration interval may be assessed by linear extrapolation to the longer interval.

Based on these considerations, an evaluation was made for all primary trip functions listed. In all cases, except one, the calibration errors due to a longer calibration interval were acceptable due to the available margin to safety limits identified in the Salem Unit 2 setpoint analyses. The only parameter for which this is not true was reactor coolant flow for which a negative 0.08% margin was predicted.

For many of the items in the list, including those channels that provide input to the RPS and ESFAS, control room indication is provided for redundant channels. Typically, the licensee is required by Technical Specifications to compare these control room indicators every 12 hours. A failed sensor or even one that has drifted compared to the others should be identified during this twice-a-day surveillance and any potential problem with that channel noted. Only if redundant sensors drift by exactly the same amount and in the same direction would this fail to identify a problem.

The Reactor Coolant Flow RPS (Loss of Flow) channel sensors are due to be calibrated at most only 9 days prior to the refueling outage. This includes all 3 channels in each of the 4 loops. The Loss of Flow protection is a primary trip with a .1% margin in the setpoint analysis. Linear extrapolation of the sensor drift will result in a negative margin of 0.08%. However, the trip is backed up by the Underfrequency and Undervoltage trips. We find that deferment of the calibration of these flow sensors is acceptable.

The Pressurizer Pressure RPS protection channel sensors are due to be calibrated between November 26 and November 30, 1982. The Pressurizer Pressure Low and High reactor trips and Low-Safety Injection trip have 0.3%, 0.9% and 1.6% margins respectively, based on a 0.75% sensor drift over 18 months.

These sensors would have to drift well beyond manufacturer specification to exceed the available margin. Therefore, we find that the surveillance deferral is acceptable for these sensors.

The Pressurizer Level sensors are due to be calibrated by January 1, 1983. They indicate a RPS trip on high level. The level sensors are not used for a primary trip and channel checks are conducted twice daily, thus deferral of surveillance is acceptable.

The Steam Generator Steam Flow sensors are due to be calibrated between November 14 and December 14, 1982. These sensors are used for one primary action - Steamline Pressure - Low. The Steamline Pressure - Low signal has a 5.9% margin which is more than 7 times the maximum sensor drift for an 18 month period. Deferment of the 18 month calibration of the Steam Generator Flow sensors is acceptable.

The Steam Generator Level protection sensors calibrations are due starting October 11, 1982. These sensors provide signals to protection channels for which credit is taken in the accident analyses (Steam Generator Water Level - Low, - Low-Low and - High-High). The margins in these channels are more than five times their design specification drift limits. Therefore, postponement of these 18 month calibrations is acceptable.

The four channels that constitute the Containment Pressure - High and - High-High trips are primary ESF functions. They have margins that would permit drift several times over design specifications. Sensor calibration deferral for these channels is acceptable.

The sensors that read the first-stage impulse pressure and the reactor coolant (RC) hot leg pressure are used for certain interlocks. Since the safety function is that of being interlocks, the requested two month deferral is acceptable.

The steam generator wide range level channels are used for post accident monitoring. Since they only perform a monitoring function, one-time deferral of their sensor calibrations is acceptable.

The turbine auxiliary speed sensor provides backup protection for turbine overspeed. The primary trip is a mechanical one. Being a secondary trip of a non-safety function, postponement of the scheduled surveillance is acceptable.

The Source Range, Neutron Flux protection channel is not taken credit for in the safety analyses. The deferment of the detector calibrations for this protection feature are acceptable.

The SSPS External Input Test checks certain contacts for manual operation of protective functions such as the contacts for the reactor coolant pump breakers. The contacts involved in these tests are separate and redundant. Failure of a set of contacts will not preclude the protective function from occurring. As such, postponement of the test is acceptable.

The Hot Shutdown Panel sensor calibrations for steam generator level and pressure and pressurizer level and pressure can also be acceptably delayed until the refueling outage. These sensors are separate from those that feed the protection channels. On a monthly basis, the output from these sensors, indicated at the Hot Shutdown Panel, are compared to companion indication in the control room.

The Containment Spray pump automatic start is another test that comes due prior to the scheduled refueling outage. There are two separate and redundant trains including separate pumps. Postponing of the test from January 7, 1983 is acceptable.

The Containment Phase A+B Isolation Tests are system level tests that come due on January 12, 1983. The functions have redundant trains and component level tests are routinely conducted that verify component operation. Test deferment is acceptable.

The Containment Sump Level Monitoring System channel calibration is due on December 13, 1982. This system does not provide an engineered safety features function, it is one of three means of leak detection of the reactor coolant system. In addition, this system is backed up by water inventory balancing. As such, postponing the calibration is acceptable.

The Hydrogen Recombiners are due to be calibrated on December 5, 1982. These electrical recombiners perform a safety function to mitigate the potential consequences of a hydrogen build-up inside containment during a post accident situation. Detailed functional checks are performed every 6 months and during these checks a severely out-of-calibration condition will be noted. Also, there exists two separate trains and in the event of their use, their operability can be verified by use of the hydrogen monitoring system. On this basis, deferral of the 18 month calibration is acceptable.

The deferral of all of the aforementioned 18 month tests and calibrations do not impact the other periodic surveillances performed on these systems. All other tests related to these systems will be conducted in accordance to the Salem Unit 2 Technical Specifications.

It should be recognized that for these 18 month surveillances the 25% extension allowed under Section 4.0.2.a of the Salem Technical Specifications has been used. As such, the requirement of Section 4.0.2.b, which states that "the combined time interval for any three consecutive surveillance interval shall not exceed 3.25 times the specified surveillance interval," for these and other surveillances that did not use the entire 25% allowance, is still in effect.

Summary

For those selected 18 month surveillance items mentioned above, the staff finds ample justification to accept the requested one-time deferral until the next refueling outage which is scheduled to begin January 22, 1983.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration,
- (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 22, 1982

Principal Contributor: M. Wigdor
J. Knox
W. Ross

Attachment

SALEM GENERATING STATION
UNIT NO. 2

Mode 5 - 18 Months Surveillance due prior to the next refueling outage

Operations Department

DESCRIPTION

DUE DATE

Containment Spray Pump Auto Start	1-07-83
Containment o A Isolation Test	1-12-83
Containment o B Isolation Test	1-12-83

Maintenance Department

DESCRIPTION

DUE DATE

Containment Sump Level Monitoring System Channel Calibration	12-13-82
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4kV Breaker Inspection and Timing:

2A4D	1-12-83
2B4D	1-12-83
2C4D	1-12-83

125VDC and 28VDC Eight Hour Battery Service Test:

2A128VDC	11-18-82
2A228VDC	11-18-82
2B128VDC	11-20-82
2B228VDC	11-20-82
2A125VDC	11-17-82
2B125VDC	11-17-82
2C125VDC	11-20-82

SEC Sequencer Test and Output Relay Time Response Test:

2A SEC	11-23-82
2B SEC	11-19-82
2C SEC	11-19-82

Hydrogen Recombiners Test

#21	12-05-82
#22	12-05-82

Diesel Generator Inspection

2A Diesel	12-17-82
2B Diesel	12-10-82
2C Diesel	12-20-82

Attachment

SALEM GENERATING STATION
UNIT NO. 2

Mode 5 - 18 Months Surveillance due prior to the next refueling outage

Instrument and Control Department

<u>DESCRIPTION</u>	<u>DUE DATE</u>
First Stage Turb. Impule Press - CH I Channel Calibr.	12-31-82
RC Flow - Loop 1 - CH I	Sensor Calibr. 1-16-83
RC Flow - Loop 1 - CH II	" " 1-13-83
RC Flow - Loop 1 - CH III	" " 1-13-83
RC Flow - Loop 2 - CH I	" " 1-20-83
RC Flow - Loop 2 - CH II	" " 1-14-83
RC Flow - Loop 2 - CH III	" " 1-20-83
RC Flow - Loop 3 - CH I	" " 1-16-83
RC Flow - Loop 3 - CH II	" " 1-14-83
RC Flow - Loop 3 - CH III	" " 1-17-83
RC Flow - Loop 4 - CH I	" " 1-17-83
RC Flow - Loop 4 - CH II	" " 1-14-83
RC Flow - Loop 4 - CH III	" " 1-17-83
Pzr. Press. Prot. CH I	" " 11-26-82
Pzr. Press. Prot. CH II	" " 11-30-82
Pzr. Press. Prot. CH III	" " 11-30-82
Pzr. Level Prot. CH I	" " 1-01-83
Pzr. Level Prot. CH II	" " 1-01-83
Pzr. Level Prot. CH III	" " 1-01-83
Pzr. Press Prot. CH IV	" " 11-29-82
First Stage Turbine Impule Press - CH I	" " 11-20-82
First Stage Turbine Impule Press - CH FI	" " 11-24-82
#21 S/G Stm Flow Prot. CH I	" " 12-13-82
#21 S/G Stm Flow Prot. CH II	" " 12-14-82
#21 S/G Level Prot. CH IV	" " 10-11-82 12-04-82
#21 S/G Level Prot. CH III	" " 12-18-82
#21 S/G Level Prot. CH II	" " 12-17-82
#22 S/G Stm Flow Prot. CH I	" " 12-05-82
#22 S/G Stm Flow Prot. CH II	" " 12-05-82
#22 S/G Level Prot. CH IV	" " 10-21-82
#22 S/G Level Prot. CH III	" " 12-28-82
#22 S/G Level Prot. CH I	" " 12-22-82
#23 S/G Stm Flow Prot. CH I	" " 12-07-82
#23 S/G Stm Flow Prot. CH II	" " 11-14-82
#24 S/G Level Prot. CH IV	" " 12-20-82
#23 S/G Level Prot. CH IV	" " 10-26-82
#23 S/G Level Prot. CH III	" " 11-13-82
#23 S/G Level Prot. CH I	" " 11-01-82
#24 S/G Stm Flow Prot. CH I	" " 12-06-82
#24 S/G Stm Flow Prot. CH II	" " 11-30-82
#24 S/G Level Prot. CH IV	" " 12-20-82
#24 S/G Level Prot. CH III	" " 11-05-82

Instrument and Control Department (Continued)

DESCRIPTIONDUE DATE

#24 S/G Level Prot.	CH II	"	"	11-05-82
Containment Press.	CH IV	"	"	11-23-82
Containment Press.	CH III	"	"	11-27-82
Containment Press.	CH II	"	"	11-25-82
Containment Press.	CH I	"	"	11-27-82
RC Hot Leg Press.		"	"	12-03-82
RC Hot Leg Press.		"	"	12-04-82
S/G #21 Wide Range Level		"	"	12-30-82
S/G #22 Wide Range Level		"	"	12-31-82
S/G #23 Wide Range Level		"	"	12-28-82
S/G #24 Wide Range Level		"	"	12-14-82
Turb. Aux. Speed				12-25-82
Replace WR RTD				1-01-83
Source Range Channel I Detector Calibration				1-08-83
Source Range Channel II Detector Calibration				1-08-83
SSPS External Input Test				12-25-82
Hot Shutdown Panel Sensor Calibrations:-				
#21 S/G Level				12-19-82
#22 S/G Level				12-19-82
#23 S/G Level				1-03-83
#24 S/G Level				1-02-83
#21 S/G Press				1-01-83
#22 S/G Press				1-01-83
#23 S/G Press				1-02-83
#24 S/G Press				1-02-83
PZR Press.				1-03-83
PZR Level				1-04-83

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-311PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 14 to Facility Operating License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised the license of the Salem Nuclear Generating Station, Unit No. 2 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment authorizes deferral of certain 18-month Technical Specification surveillance items until the first refueling outage. These items are described in the Safety Evaluation.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

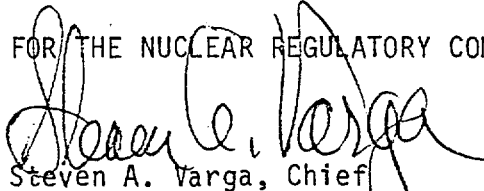
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 17, 1982, (2) Amendment No. 14 to License No. DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 22nd day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing