

February 19, 1998

Mr. Harold W. Keiser
Executive Vice President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: CORRECTION TO AMENDMENT NO. 202, SALEM NUCLEAR GENERATING
STATION, UNIT NO. 1 (TAC NO. M99760)

Dear Mr. Keiser:

On December 22, 1997, the Commission issued Amendment No. 202 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consist of changes to the Technical Specifications (TSs) in response to your application dated March 4, 1997.

On November 26, 1997, the Commission issued Amendment No. 201 for Salem Unit No. 1 that modified, in part, TS 3.1.3.1.c.2 and Bases Section 3/4.2.2 and 3/4.2.3. When Amendment No. 202 was issued, these changes from Amendment No. 201 were not incorporated into the revised pages issued with Amendment No. 202. Therefore, corrected Salem Unit No. 1 TSs pages 3/4 1-18 and B 3/4 2-4 are provided as replacements for those issued in Amendment No. 202.

The Commission regrets any inconvenience that this administrative error may have caused.

Sincerely,

/s/
Patrick D. Milano, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-272

Enclosures: TS Pages 3/4 1-18 and
B 3/4 2-4

cc w/encls: See next page

DISTRIBUTION

Docket File JMoore, OGC
PUBLIC ACRS
PDI-2 Reading WBeckner
JZwolinski MChatterton
JStolz GHill(2)
PMilano JLinville, RGN-I
TClark TCollins



OFFICE	PDI-2/PM	PDI-2/LA	PDI-2/D	OGC	
NAME	PMilano:rb	TClark	JStolz	RBachmann	
DATE	2/18/98	2/18/98	2/19/98	2/18/98	

OFFICIAL RECORD COPY

DOCUMENT NAME: SA99760.GEN

9803090292 980219
PDR ADOCK 05000272
PDR

FILE GENERATED COPY



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

February 19, 1998

Mr. Harold W. Keiser
Executive Vice President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

**SUBJECT: CORRECTION TO AMENDMENT NO. 202, SALEM NUCLEAR GENERATING
STATION, UNIT NO. 1 (TAC NO. M99760)**

Dear Mr. Keiser:

On December 22, 1997, the Commission issued Amendment No. 202 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consist of changes to the Technical Specifications (TSs) in response to your application dated March 4, 1997.

On November 26, 1997, the Commission issued Amendment No. 201 for Salem Unit No. 1 that modified, in part, TS 3.1.3.1.c.2 and Bases Section 3/4.2.2 and 3/4.2.3. When Amendment No. 202 was issued, these changes from Amendment No. 201 were not incorporated into the revised pages issued with Amendment No. 202. Therefore, corrected Salem Unit No. 1 TSs pages 3/4 1-18 and B 3/4 2-4 are provided as replacements for those issued in Amendment No. 202.

The Commission regrets any inconvenience that this administrative error may have caused.

Sincerely,

A handwritten signature in black ink, appearing to read "Patrick D. Milano", is written over a horizontal line.

Patrick D. Milano, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-272

Enclosures: TS Pages 3/4 1-18 and
B 3/4 2-4

cc w/encls: See next page

Mr. Harold W. Keiser
Public Service Electric & Gas
Company

cc:

Jeffrie J. Keenan, Esquire
Nuclear Business Unit - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

General Manager - Salem Operations
Salem Nuclear Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. Louis Storz
Sr. Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Senior Resident Inspector
Salem Nuclear Generating Station
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415

Maryland Office of People's Counsel
6 St. Paul Street, 21st Floor
Suite 2102
Baltimore, MD 21202

Ms. R. A. Kankus
Joint Owner Affairs
PECO Energy Company
965 Chesterbrook Blvd., 63C-5
Wayne, PA 19087

Mr. Elbert Simpson
Senior Vice President-
Nuclear Engineering
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Salem Nuclear Generating Station,
Units 1 and 2

Richard Hartung
Electric Service Evaluation
Board of Regulatory Commissioners
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Manager-Licensing and Regulation
Nuclear Busienss Unit - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. David Wersan
Assistant Consumer Advocate
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, PA 17120

Manager - Joint Generation
Atlantic Energy
6801 Black Horse Pike
Egg Harbor Twp., NJ 08234-4130

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
P.O. Box 231
Wilmington, DE 19899

Public Service Commission of Maryland
Engineering Division
Chief Engineer
6 St. Paul Centre
Baltimore, MD 21202-6806

REACTIVITY CONTROL SYSTEMS

3/4.1.3 MOVABLE CONTROL ASSEMBLIES

GROUP HEIGHT

LIMITING CONDITION FOR OPERATION

3.1.3.1 All full length (shutdown and control) rods, shall be OPERABLE and positioned within ± 18 steps (indicated position) when reactor power is $\leq 85\%$ RATED THERMAL POWER, or ± 12 steps (indicated position) when reactor power is $> 85\%$ RATED THERMAL POWER, of their group step counter demand position within one hour after rod motion.

APPLICABILITY: MODES 1* and 2*

ACTION:

- a. With one or more full length rods inoperable due to being immovable as a result of excessive friction or mechanical interference or known to be untrippable, determine that the SHUTDOWN MARGIN requirement of Specification 3.1.1.1 is satisfied within 1 hour and be in HOT STANDBY within 6 hours.
- b. With more than one full length rod inoperable or mis-aligned from the group step counter demand position by more than ± 18 steps (indicated position) at $\leq 85\%$ RATED THERMAL POWER or ± 12 steps (indicated position) at $> 85\%$ RATED THERMAL POWER, be in HOT STANDBY within 6 hours.
- c. With one full length rod inoperable due to causes other than addressed by ACTION a, above, or mis-aligned from its group step counter demand position by more than ± 18 steps (indicated position) at $\leq 85\%$ RATED THERMAL POWER or ± 12 steps (indicated position) at $> 85\%$ RATED THERMAL POWER, POWER OPERATION may continue provided that within one hour either:
 1. The rod is restored to OPERABLE status within the above alignment requirements, or
 2. The remainder of the rods in the bank with the inoperable rod are aligned to within ± 18 steps (indicated position) at $\leq 85\%$ RATED THERMAL POWER or ± 12 steps (indicated position) at $> 85\%$ RATED THERMAL POWER of the inoperable rod while maintaining the rod sequence and insertion limits in the COLR per specification 3.1.3.5. The THERMAL POWER level shall be restricted pursuant to Specification 3.1.3.5 during subsequent operation, or
 3. The rod is declared inoperable and the SHUTDOWN MARGIN requirement of Specification 3.1.1.1 is satisfied. POWER OPERATION may then continue provided that:

9803090298 980219
PDR ADOCK 05000272
P PDR

See special test exceptions 3.10.2 and 3.10.3.

THIS PAGE INTENTIONALLY LEFT BLANK

POWER DISTRIBUTION LIMITS

BASES

3/4.2.2 and 3/4.2.3 HEAT FLUX AND NUCLEAR ENTHALPY HOT CHANNEL AND RADIAL PEAKING FACTORS - $F_Q(Z)$, F_{DH}^N and $F_{xy}(Z)$

The limits on heat flux and nuclear enthalpy hot channel factors ensure that 1) the design limits on peak local power density and minimum DNBR are not exceeded and 2) in the event of a LOCA the peak fuel clad temperature will not exceed the 2200°F ECCS acceptance criteria limit.

Each of these hot channel factors are measurable but will normally only be determined periodically as specified in Specifications 4.2.2 and 4.2.3. This periodic surveillance is sufficient to insure that the hot channel factor limits are maintained provided:

- a. Control rod in a single group move together with no individual rod insertion differing from the group demand position by more than the allowed rod misalignment.
- b. Control rod groups are sequenced with overlapping groups as described in Specification 3.1.3.5.
- c. The control rod insertion limits of Specifications 3.1.3.4 and 3.1.3.5 are maintained.
- d. The axial power distribution, expressed in terms of AXIAL FLUX DIFFERENCE, is maintained within the limits.

The relaxation in F_{DH}^N as a function of THERMAL POWER allows changes in the radial power shape for all permissible rod insertion limits. F_{DH}^N will be maintained within its limits provided conditions a thru d above, are maintained.

When an F_Q measurement is taken, both experimental error and manufacturing tolerance must be allowed for 5% is the appropriate allowance for a full core map taken with the incore detector flux mapping system and 3% is the appropriate allowance for manufacturing tolerance.

When F_{DH}^N is measured, experimental error must be allowed for and 4% is the appropriate allowance for a full core map taken with the incore detection system. The specified limit for F_{DH}^N also contains an 8% allowance for uncertainties which mean that normal operation will result in

$F_{DH}^N \leq F_{DH}^{RTP} / 1.08$ where F_{DH}^{RTP} is the limit of RATED THERMAL POWER (RTP) specified in the CORE OPERATING LIMITS REPORT (COLR). The 8% allowance is based on the following considerations:

INFORMATION ONLY*

Percent of Rated
Thermal Power

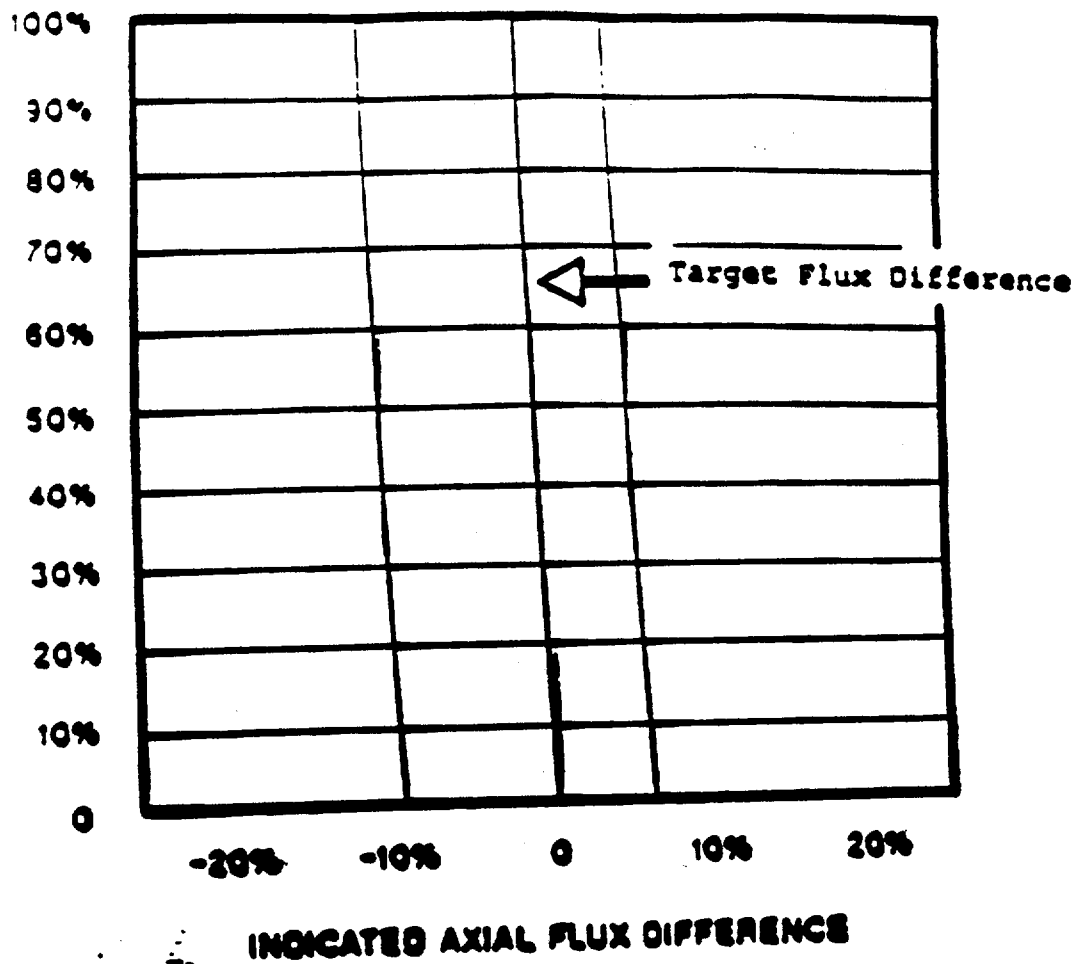


Figure B 3/4 2-1 TYPICAL INDICATED AXIAL FLUX DIFFERENCE
VERSUS THERMAL POWER

* Refer to COLR Figure 2 for Actual Limits