

June 24, 1997

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: CONTAINMENT COOLING SYSTEM TECHNICAL SPECIFICATION REVISION, SALEM
NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M98410 AND
M98411)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment Nos. 197 and 180 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Units 1 and 2. These amendments consist of changes to the Technical
Specifications (TSs) in response to your application dated April 11, 1997.

These amendments revise TS 3/4.6.2.3, "Containment Cooling System," and its
associated Bases section to ensure that the TSs properly test the containment
fan cooling units' post-accident mode of operation.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,
/s/

Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-272/311

- Enclosures: 1. Amendment No. 197 to License No. DPR-70
- 2. Amendment No. 180 to License No. DPR-75
- 3. Safety Evaluation

cc w/encls: See next page

DFD/11

DISTRIBUTION

Docket File	JStolz	GHill(4)	JLinville, RGN-I
PUBLIC	MO'Brien	CBerlinger	SDembek
PDI-2 Reading	LOlshan	CGrimes	
SVarga	OGC	ACRS	

OFFICE	SPO-L/PM	PDI-2/LA	BDI-2/PM	SCSB/BC	OGC	RDG-2/D
NAME	SDembek:rb	MO'Brien	LOlshan	CBerlinger	ADH	JStolz
DATE	6/15/97	4/15/97	6/15/97	6/19/97	6/11/97	6/24/97

OFFICIAL RECORD COPY
DOCUMENT NAME: SA98410.AMD

NRG FILE CENTER COPY

270043

9706270234 970624
PDR ADDCK 05000272
PDR



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 24, 1997

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: CONTAINMENT COOLING SYSTEM TECHNICAL SPECIFICATION REVISION, SALEM
NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M98410 AND
M98411)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment Nos. 197 and 180 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Units 1 and 2. These amendments consist of changes to the Technical
Specifications (TSs) in response to your application dated April 11, 1997.

These amendments revise TS 3/4.6.2.3, "Containment Cooling System," and its
associated Bases section to ensure that the TSs properly test the containment
fan cooling units' post-accident mode of operation.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-272/311

Enclosures: 1. Amendment No. 197 to
License No. DPR-70
2. Amendment No. 180 to
License No. DPR-75
3. Safety Evaluation

cc w/encls: See next page

Mr. Leon R. Eliason
Public Service Electric & Gas
Company

Salem Nuclear Generating Station,
Units 1 and 2

cc:

Mark J. Wetterhahn, Esquire
Winston & Strawn
1400 L Street NW
Washington, DC 20005-3502

Richard Hartung
Electric Service Evaluation
Board of Regulatory Commissioners
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Richard Fryling, Jr., Esquire
Law Department - Tower 5E
80 Park Place
Newark, NJ 07101

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. D. F. Garchow
General Manager - Salem Operations
Salem Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Mr. Louis Storz
Sr. Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Mr. David R. Powell, Manager
Licensing and Regulation
Nuclear Business Unit
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. Charles S. Marschall, Senior
Resident Inspector
Salem Generating Station
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038

Mr. David Wersan
Assistant Consumer Advocate
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, PA 17120

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415

P. M. Goetz
MGR. Joint Generation
Atlantic Energy
6801 Black Horse Pike
Egg Harbor Twp., NJ 08234-4130

Maryland Office of People's Counsel
6 St. Paul Street, 21st Floor
Suite 2102
Baltimore, Maryland 21202

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
P.O. Box 231
Wilmington, DE 19899

Ms. R. A. Kankus
Joint Owner Affairs
PECO Energy Company
965 Chesterbrook Blvd., 63C-5
Wayne, PA 19087

Public Service Commission of Maryland
Engineering Division
Chief Engineer
6 St. Paul Centre
Baltimore, MD 21202-6806

Mr. Elbert Simpson
Senior Vice President - Nuclear Engineering
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated April 11, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

9706270289 970624
PDR ADOCK 05000272
P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 197, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: June 24, 1997

ATTACHMENT TO LICENSE AMENDMENT NO.197

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 6-11a

B 3/4 6-3

Insert Pages

3/4 6-11a

B 3/4 6-3

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

=====

- a. At least once per 31 days by:
 - 1. Starting (unless already operating) each fan from the control room in low speed.
 - 2. Verifying that each fan operates for at least 15 minutes in low speed.
 - 3. Verifying a cooling water flow rate of greater than or equal to 2550 gpm to each cooler.

- b. At least once per 18 months by verifying that on a safety injection test signal:
 - 1. Each fan starts automatically in low speed.
 - 2. The automatic valves and dampers actuate to their correct positions and that the cooling water flow rate to each cooler is greater than or equal to 2550 gpm.

CONTAINMENT SYSTEMS

BASES

=====

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 CONTAINMENT SPRAY SYSTEM

The OPERABILITY of the containment spray system ensures that containment depressurization and cooling capability will be available in the event of a LOCA. The pressure reduction and resultant lower containment leakage rate are consistent with the assumptions used in the accident analyses.

3/4.6.2.2 SPRAY ADDITIVE SYSTEM

The OPERABILITY of the spray additive system ensures that sufficient NaOH is added to the containment spray in the event of a LOCA. The limits on NaOH minimum volume and concentration, ensure that 1) the iodine removal efficiency of the spray water is maintained because of the increase in pH value, and 2) corrosion effects on components within containment are minimized. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics. These assumptions are consistent with the iodine removal efficiency assumed in the accident analyses.

3/4.6.2.3 CONTAINMENT COOLING SYSTEM

The OPERABILITY of the containment cooling system ensures that adequate heat removal capacity is available when operated in conjunction with the containment spray systems during post-LOCA conditions.

3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. Containment isolation within the time limits specified ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

The opening of locked or sealed closed containment isolation valves on an intermittent basis under administrative control includes the following considerations: (1) stationing a dedicated individual, who is in constant communication with the control room, at the valve controls, (2) instructing this individual to close these valves in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valves and that this action will prevent the release of radioactivity outside the containment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 180
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated April 11, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.180 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: June 24, 1997

ATTACHMENT TO LICENSE AMENDMENT NO.180

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

3/4 6-13
B 3/4 6-3

Insert Pages

3/4 6-13
B 3/4 6-3

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

=====

- a. At least once per 31 days by:
 - 1. Starting (unless already operating) each fan from the control room in low speed.
 - 2. Verifying that each fan operates for at least 15 minutes in low speed.
 - 3. Verifying a cooling water flow rate of greater than or equal to 2550 gpm to each cooler.

- b. At least once per 18 months by verifying that on a safety injection test signal:
 - 1. Each fan starts automatically in low speed.
 - 2. The automatic valves and dampers actuate to their correct positions and that the cooling water flow rate to each cooler is greater than or equal to 2550 gpm.

CONTAINMENT SYSTEMS

BASES

=====

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

3/4.6.2.1 CONTAINMENT SPRAY SYSTEM

The OPERABILITY of the containment spray system ensures that containment depressurization and cooling capability will be available in the event of a LOCA. The pressure reduction and resultant lower containment leakage rate are consistent with the assumptions used in the accident analyses.

The containment spray system and the containment cooling system are redundant to each other in providing post accident cooling of the containment atmosphere. However, the containment spray system also provides a mechanism for removing iodine from the containment atmosphere and therefore the time requirements for restoring an inoperable spray system to OPERABLE status have been maintained consistent with that assigned other inoperable ESF equipment.

3/4.6.2.2 SPRAY ADDITIVE SYSTEM

The OPERABILITY of the spray additive system ensures that sufficient NaOH is added to the containment spray in the event of a LOCA. The limits on NaOH minimum volume and concentration, ensure that 1) the iodine removal efficiency of the spray water is maintained because of the increase in pH value, and 2) corrosion effects on components within containment are minimized. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics. These assumptions are consistent with the iodine removal efficiency assumed in the accident analyses.

3/4.6.2.3 CONTAINMENT COOLING SYSTEM

The OPERABILITY of the containment cooling system ensures that adequate heat removal capacity is available when operated in conjunction with the containment spray systems during post-LOCA conditions.

The containment cooling system and the containment spray system are redundant to each other in providing post accident cooling of the containment atmosphere. As a result of this redundancy in cooling capability, the allowable out of service time requirements for the containment cooling system have been appropriately adjusted. However, the allowable out of service time requirements for the containment spray system have been maintained consistent with that assigned other inoperable ESF equipment since the containment spray system also provides a mechanism for removing iodine from the containment atmosphere.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 197 AND 180 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated April 11, 1997, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Salem Nuclear Generating Station, Units 1 and 2, Technical Specifications (TSs). The licensee's amendment request would revise TS 3/4.6.2.3, "Containment Cooling System," and the associated Bases section, to ensure that the TS properly tests the containment fan cooling units' post-accident mode of operation.

2.0 EVALUATION

Currently the licensee's containment cooling fan TS surveillance requirement 4.6.2.3 states:

Each containment cooling fan shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Starting (unless already operating) each fan from the control room.
 2. Verifying that each fan operates for at least 15 minutes.
 3. Verifying a cooling water flow rate of greater than or equal to 700 gpm to each cooler.
- b. At least once per 18 months by verifying that on a safety injection test signal:
 1. Each fan starts automatically on low speed.
 2. The automatic valves and dampers actuate to their correct positions and that the cooling water flow rate to each cooler is greater than or equal to 2500 gpm.

Regarding the purpose of this surveillance requirement, the licensee's applicable TS Bases states:

The OPERABILITY of the containment cooling system ensures that 1) the containment air temperature will be maintained within limits during normal operation, and 2) adequate heat removal capacity is available when operated in conjunction with the containment spray systems during post-LOCA conditions.

The licensee found that the current TS surveillance requirement 4.6.2.3 does not adequately test the containment fan cooling units' post-accident speed (i.e., low speed) operation and performs an unnecessary test of the containment fan cooling units' normal cooling function.

To correct these shortcomings, the licensee is proposing to revise the TS wording to read as follows (changes are in italics):

Each containment cooling fan shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Starting (unless already operating) each fan from the control room *in low speed*.
 2. Verifying that each fan operates for at least 15 minutes *in low speed*.
 3. Verifying a cooling water flow rate of greater than or equal to 2550 gpm to each cooler.

- b. At least once per 18 months by verifying that on a safety injection test signal:
 1. Each fan starts automatically *in low speed*.
 2. The automatic valves and dampers actuate to their correct positions and that the cooling water flow rate to each cooler is greater than or equal to 2550 gpm.

The licensee found that the requirement to test the normal cooling function is more appropriately verified through TS surveillance requirement 4.6.1.5 which requires verification of the average containment temperature once every 24 hours in Modes 1 through 4. Therefore, the licensee proposed to revise the TS wording to ensure that the cooling water flow rate and fan speed being verified is representative of the containment fan cooling units' post-accident mode of operation and to delete the requirement to test the normal cooling operation. The staff agrees that since TS surveillance requirement 4.6.1.5 verifies average containment temperature once every 24 hours, in Modes 1 through 4, there is no need to verify the normal operation of the containment fan cooling units every 31 days (if the cooling units were not working properly, the containment temperature would rise).

Additionally, the licensee's proposed TS revision tests the low speed operation of the containment fan cooling units on a 31-day interval, which is a conservative improvement over the 18-month interval that the current TS

requires. Also, the current 18-month surveillance that tests each fan in low speed specifies a nonconservative minimum flow rate of 2500 gpm. The licensee found that a value of 2550 gpm (which accounts for an additional 50 gpm flow for cooling the fan motor) is a more appropriate minimum flow rate value. The staff agrees that the licensee's addition of the 50 gpm for cooling the fan motor provides for a more conservative minimum flow test requirement.

Based on the above discussion, the staff finds that the licensee's proposed TS wording provides for an enhanced test of the post-accident mode of containment fan cooling unit operation and is therefore acceptable.

Also, the licensee proposed to change the related TS Bases section to delete the statement that operability of the containment fan cooling units ensures that the containment air temperature is within limits during normal operation. The licensee's proposed TS wording deletes this operability test from TS surveillance requirement 4.6.2.3. Therefore, the licensee's proposed TS Bases change accurately reflects the bases of the proposed TS surveillance requirement and is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 27799). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: June 24, 1997