

December 27 1994

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M90635
AND M90636)

Dear Mr. Eliason:

The Commission has issued the enclosed Amendment Nos. 163 and 144 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Unit Nos. 1 and 2. These amendments consist of changes to the
Technical Specifications (TSs) in response to your application dated
September 29, 1994.

These amendments change the surveillance frequency from 5 to 10 years for
performing an air or smoke flow test through each containment spray header.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by
Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-272/50-311

Enclosures:

1. Amendment No. 163 to
License No. DPR-70
 2. Amendment No. 144 to
License No. DPR-75
 3. Safety Evaluation
- cc w/encls:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001
December 27, 1994

Mr. Leon R. Eliason
Chief Nuclear Officer & President-
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. M90635
AND M90636)

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These amendments change the surveillance frequency from 5 to 10 years for performing an air or smoke flow test through each containment spray header.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Leonard N. Olshan", is written over the typed name.

Leonard N. Olshan, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-272/50-311

Enclosures:

1. Amendment No. 163 to
License No. DPR-70
2. Amendment No. 144 to
License No. DPR-75
3. Safety Evaluation

cc w/encls:
See next page

Mr. Leon R. Eliason
Public Service Electric & Gas
Company

Salem Nuclear Generating Station,
Units 1 and 2

cc:

Mark J. Wetterhahn, Esquire
Winston & Strawn
1400 L Street NW
Washington, DC 20005-3502

Richard Hartung
Electric Service Evaluation
Board of Regulatory Commissioners
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Richard Fryling, Jr., Esquire
Law Department - Tower 5E
80 Park Place
Newark, NJ 07101

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. J. Hagan, Acting
General Manager - Salem Operations
Salem Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Mr. J. Hagan
Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Mr. Frank X. Thomson, Jr., Manager
Licensing and Regulation
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. Charles S. Marschall, Senior
Resident Inspector
Salem Generating Station
U.S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, NJ 08038

Mr. David Wersan
Assistant Consumer Advocate
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, PA 17120

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415

Ms. P. J. Curham
MGR. Joint Generation Department
Atlantic Electric Company
P.O. Box 1500
6801 Black Horse Pike
Pleasantville, NJ 08232

Maryland Office of People's Counsel
6 St. Paul Street, 21st Floor
Suite 2102
Baltimore, Maryland 21202

Carl D. Schaefer
External Operations - Nuclear
Delmarva Power & Light Company
P.O. Box 231
Wilmington, DE 19899

Mr. J. T. Robb, Director
Joint Owners Affairs
PECO Energy Company
955 Chesterbrook Blvd., 51A-13
Wayne, PA 19087

Public Service Commission of Maryland
Engineering Division
Chief Engineer
6 St. Paul Centre
Baltimore, MD 21202-6806

Mr. S. LaBruna
Vice President - Nuclear Engineering
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

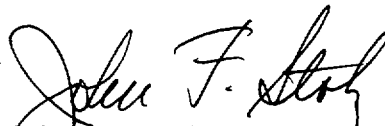
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 27, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Page

3/4 6-9

Insert Page

3/4 6-9

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

=====

3.6.2.1 Two independent containment spray systems shall be OPERABLE with each spray system capable of taking suction from the RWST and transferring suction to the RHR pump discharge.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one containment spray system inoperable, restore the inoperable spray system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable spray system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

=====

4.6.2.1 Each containment spray system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
 2. Verifying that each spray pump starts automatically on a Containment High-High pressure test signal.
- c. At least once per 10 years by:
 1. Performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 144
License No. DPR-75

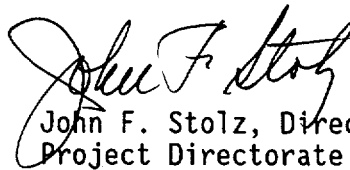
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 144, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 27, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 144

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

3/4 6-10

Insert Page

3/4 6-10

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

=====

3.6.2.1 Two independent containment spray systems shall be OPERABLE with each spray system capable of taking suction from the RWST and transferring suction to the RHR pump discharge.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one containment spray system inoperable, restore the inoperable spray system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable spray system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

=====

4.6.2.1 Each containment spray system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. By verifying, that on recirculation flow, each pump develops a discharge pressure of greater than or equal to 215 psig when tested pursuant to Specification 4.0.5.
- c. At least once per 18 months during shutdown, by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
 2. Verifying each spray pump starts automatically on a Containment High-High pressure test signal.
- d. At least once per 10 years by:
 1. Performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 163 AND 144 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated September 29, 1994, the Public Service Electric & Gas Company (the licensee) submitted a request for changes to the Salem Nuclear Generating Station, Unit Nos. 1 and 2, Technical Specifications (TS). The requested changes would change the surveillance interval for performing an air or smoke flow test through each containment spray header from 5 to 10 years.

2.0 EVALUATION

The spray nozzles and piping in the Containment Spray System are fabricated from stainless steel. The spray systems are currently tested on a 5-year interval to confirm the absence of blockage that could potentially impact the required flow rates following a loss-of-coolant accident.

NUREG-1366, "Improvements to Technical Specifications Surveillance Requirements," dated December 1992, evaluated the testing of spray nozzles in pressurized water reactors' containment spray systems with stainless steel piping. The conclusion drawn from this evaluation was that the corrosion of stainless steel is negligible during the extended surveillance interval. Therefore, since the spray systems are maintained dry and there are no other postulated mechanisms that could cause blockage, it was concluded that the surveillance interval could be increased from 5 to 10 years without any reduction in the plant safety. Moreover, the licensee examined the results of the last two surveillance tests and noted that there were no problems that would have resulted in nozzle blockage. The proposed reduced testing of the spray systems' nozzle remains adequate to ensure operability of the nozzles to mitigate the consequences of a Design Basis Accident. Based on all of the above, the staff finds the proposed changes to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 60385). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Olshan

Date: December 27, 1994

Public Service
Electric and Gas
Company

Joseph J. Hagan

Public Service Electric and Gas Company P.O. Box 236, Hancocks Bridge, NJ 08038 609-339-1200

Vice President - Nuclear Operations

SEP 29 1994

NLR-N94159
LCR 94-29

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

**LICENSE AMENDMENT APPLICATION
REVISION OF CONTAINMENT SPRAY SYSTEM SURVEILLANCE FREQUENCY
SALEM GENERATING STATION UNIT NOS. 1 and 2
FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75
DOCKET NOS. 50-272 AND 50-311**

This letter submits an application for amendment to Appendix A of Facility Operating Licenses DPR-70 and DPR-75 for the Salem Generating Station Unit Nos. 1 and 2, respectively, and is being filed in accordance with 10CFR50.90. Pursuant to the requirements of 10CFR50.91(b)(1), a copy of this request for amendment has been sent to the State of New Jersey.

The proposed Technical Specification change contained herein represents a change to Section 4.6.2.1 and implements a recommended line item improvement from Generic Letter 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation." The proposed change revises the surveillance interval for performing an air or smoke flow test through each containment spray header from once every five years to once every ten years. It should be noted that similar changes were approved by the NRC Staff for Surry Power Station Unit Nos. 1 and 2 in an SER dated May 20, 1994.

The proposed change has been evaluated in accordance with 10CFR50.91(a)(1), using the criteria in 10CFR50.92(c), and it has been determined that this request involves no significant hazards considerations.

A description of the requested amendment, supporting information and analyses for the change, and the basis for a no significant hazards consideration determination are provided in Attachment 1. The Technical Specification pages affected by the proposed change are provided in Attachment 2 and 3, with pen and ink changes, for Salem Units 1 and 2, respectively.

Upon NRC approval of this proposed change, PSE&G requests that the

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NLR-N94159

-2-

amendment be made effective on the date of issuance, but implemented within sixty days to provide sufficient time for associated administrative activities.

Should you have any questions regarding this request, please contact us.

Sincerely,

Affidavit
Attachments (3)

C Mr. T. T. Martin, Administrator - Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. J. C. Stone, Licensing Project Manager - Salem
U. S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

Mr. C. S. Marschall (S09)
USNRC Senior Resident Inspector

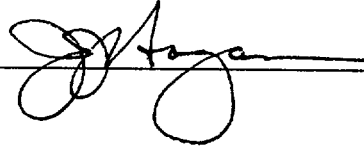
Mr. K. Tosch, Manager IV
NJ Department of Environmental Protection
Division of Environmental Quality
Bureau of Nuclear Engineering
CN 415
Trenton, NJ 08625



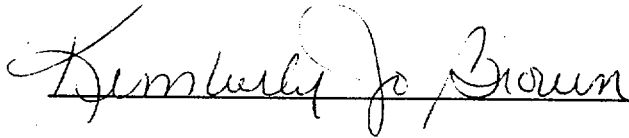
STATE OF NEW JERSEY)
) SS.
COUNTY OF SALEM)

J. J. Hagan, being duly sworn according to law, deposes and says:

I am Vice President - Nuclear Operations of Public Service Electric and Gas Company, and as such, I find the matters set forth in the above referenced letter, concerning the Salem Generating Station, Unit Nos. 1 and 2, are true to the best of my knowledge, information and belief.



Subscribed and Sworn to before me
this 29th day of September, 1994



KIMBERLY JO BROWN
NOTARY PUBLIC OF NEW JERSEY
My Commission Expires April 21, 1998

My Commission expires on _____

ATTACHMENT 1

PROPOSED CHANGES TO TECHNICAL SPECIFICATIONS

LICENSE AMENDMENT APPLICATION

REVISION OF CONTAINMENT SPRAY SYSTEM SURVEILLANCE FREQUENCY

SALEM GENERATING STATION UNIT NOS. 1 AND 2

FACILITY OPERATING LICENSES DPR-70 AND DPR-75

DOCKET NOS. 50-272 and 50-311

NLR-N94159

LCR 94-29

I. DESCRIPTION OF THE PROPOSED CHANGE

This amendment revises Surveillance Requirement 4.6.2.1 by requiring a Containment Spray System header flow test every ten (10) years instead of every five (5) years to confirm that the spray nozzles are unobstructed.

II. REASONS FOR THE CHANGE

Through this submittal, PSE&G is requesting implementation of the NRC's recommendation regarding the extension of the surveillance interval for performing flow tests through each spray header and verifying that each spray nozzle is unobstructed.

The proposed change to the Technical Specifications is consistent with the guidance of Generic Letter (GL) 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation" and incorporates the recommendation of NUREG 1366, "Improvements to Technical Specifications Surveillance Requirements."

The proposed change is further supported by the associated reduction in radiation exposure to plant personnel and the elimination of an unnecessary burden on plant staff.

III. JUSTIFICATION FOR CHANGE

Technical Specification 4.6.2.1 requires the Containment Spray System to be demonstrated operable at least once every five years by performing an air or smoke flow test through each spray header and verifying that each spray nozzle is unobstructed. The test only verifies that there is flow and does not provide quantitative data on flow rates through the nozzles.

As noted by the NRC in NUREG-1366 and GL 93-05, the NRC reviewed industry data for problems involving the containment spray system uncovered by means of this testing. The problems found in other facilities occurred during construction or were the result of coatings applied to the carbon steel piping. The Salem Unit 1 and Unit 2 Containment Spray Systems utilizes stainless steel piping and does not use coating materials such as sodium silicate, the cause of problems in containment spray systems using carbon steel, as noted in GL 93-05, Item 8.1. A review of the previous two (2) Containment Spray nozzle surveillance tests performed on either Salem Unit 1 or Unit 2, indicates there were no problems which would have resulted in nozzle blockage. The proposed change is therefore compatible with Salem Unit 1 and Unit 2 plant operating experience.

IV. DETERMINATION OF NO SIGNIFICANT HAZARDS CONSIDERATION

PSE&G has, pursuant to 10CFR50.92, reviewed the proposed amendment to determine whether our request involves a significant hazards consideration. We have determined that operation of Salem Generating Station Unit 1 and Unit 2, in accordance with the proposed changes:

1. Does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change does not affect the assumptions, design parameters or results of UFSAR accidents analyzed. The proposed change does not involve a hardware change, a change to the operation of any system or component, or a change to an existing structure. The proposed change leads to a reduction in radiation exposure to plant personnel and the reduction of an unnecessary burden on plant staff. The Containment Spray System header and nozzles are fabricated from corrosion resistant stainless steel and are maintained dry. Operating experience demonstrates that the proposed increase in the Containment Spray surveillance test interval would not affect operability of the system. Testing the Containment Spray System header and nozzles at the proposed increased surveillance interval does not increase the probability or consequences of an accident previously evaluated.

2. Does not create the possibility of a new or different kind of accident from any previously evaluated.

The proposed change does not modify equipment, affect the system design basis or operability. This change does not alter parameters utilized in the analyzed accident scenarios. The Containment Spray System piping and nozzles are fabricated from corrosion resistant stainless steel. The proposed change in surveillance frequency is consistent with the guidance provided in GL 93-05. Testing the Containment Spray System header and nozzles at the proposed increased surveillance interval does not create the possibility of a new or different kind of accident from those previously evaluated.

3. Does not involve a significant reduction in a margin of safety.

The proposed change only involves a decrease in surveillance frequency and does not alter the performance of the surveillance itself. System equipment and operation remains unchanged. Operability and reliability is still maintained by periodic testing. Testing the Containment Spray System header and nozzles at the proposed increased surveillance interval does not involve a significant reduction in the margins of safety.

V. CONCLUSIONS

Based on the above, PSE&G has determined that the proposed change does not involve a significant hazards consideration.

ATTACHMENT 2

TECHNICAL SPECIFICATION PAGES WITH PEN AND INK CHANGES

LICENSE AMENDMENT APPLICATION
REVISION OF CONTAINMENT SPRAY SYSTEM SURVEILLANCE FREQUENCY
SALEM GENERATING STATION UNIT 1
FACILITY OPERATING LICENSE DPR-70
DOCKET NO. 50-272

NLR-N94159
LCR 94-29

The following Technical Specification for Facility Operating License No. DPR-70 is affected by this License Amendment Request:

| <u>Technical Specification</u> | <u>Page</u> |
|--------------------------------|-------------|
| 4.6.2.1.c | 3/4 6-9 |

ATTACHMENT 3

TECHNICAL SPECIFICATION PAGES WITH PEN AND INK CHANGES

LICENSE AMENDMENT APPLICATION

REVISION OF CONTAINMENT SPRAY SYSTEM SURVEILLANCE FREQUENCY

SALEM GENERATING STATION UNIT 2

FACILITY OPERATING LICENSE DPR-75

DOCKET NO. 50-311

NLR-N94159

LCR 94-29

The following Technical Specification for Facility Operating License No. DPR-75 is affected by this License Amendment Request:

Technical Specification

Page

4.6.2.1.d

3/4 6-10