

Docket Nos. 50-272
and 50-311

June 5, 1991

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: 40-YEAR OPERATING LICENSE AMENDMENT, SALEM NUCLEAR GENERATING
STATION, UNIT NOS. 1 AND 2 (TAC NOS. 69267 AND 69268)

The Commission has issued the enclosed Amendment Nos.125 and 104 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Unit Nos. 1 and 2. These amendments consist of changes to the
Operating Licenses in response to your application dated August 3, 1987, as
supplemented August 10, 1990, August 21, 1990 and May 22, 1991.

These amendments extend the expiration date for the Salem Unit 1 Operating
License from September 25, 2008 to August 13, 2016 and for the Salem Unit 2
Operating License from September 25, 2008 to April 18, 2020. The original
date is 40 years from the date of issuance of the Construction Permit. The
revised date is 40 years from the date of issuance of the Operating License.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

James C. Stone, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.125 to
License No. DPR-70
2. Amendment No.104 to
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:

See next page

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Docket File	MO'Brien(3)	Wanda Jones, 7103	CGrimes, 13H-15
NRC & Local PDR	JStone	OC/LFMB	
PDI-2 Reading	SDembek	ACRS(10)	
SVarga	OGC	GPA/PA	
JCalvo	DHagan, MS-3206	JWhite, RGN-I	
WButler	GHill(8), P1-37	RBlough, RGN-I	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

June 5, 1991

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The Commission has issued the enclosed Amendment Nos. 125 and 104 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Operating Licenses in response to your application dated August 3, 1987, as supplemented August 10, 1990, August 21, 1990 and May 22, 1991.

These amendments extend the expiration date for the Salem Unit 1 Operating License from September 25, 2008 to August 13, 2016 and for the Salem Unit 2 Operating License from September 25, 2008 to April 18, 2020. The original date is 40 years from the date of issuance of the Construction Permit. The revised date is 40 years from the date of issuance of the Operating License.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink that reads "James C. Stone".

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 125 to
License No. DPR-70
2. Amendment No. 104 to
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 3, 1987, as supplemented August 10, 1990, August 21, 1990 and May 22, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-70 is hereby amended by changing paragraph 2.H. as follows*:

H. This amended license is effective as of the date of its issuance. Facility Operating License No. DPR-70, as amended, shall expire at midnight, August 13, 2016.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Page 5 of License

Date of Issuance: June 5, 1991

*Page 5 of the License is attached, for convenience, for the composite License to reflect this change.

- D. Paragraph 2.D. has been combined with paragraph 2.E. per Amendment No. 86, June 27, 1988.
- E. The licensees shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Salem Nuclear Generating Station Physical Security Plan," with revisions submitted through September 4, 1987; "Salem Nuclear Generating Station Guard Training and Qualification Plan," with revisions submitted through September 4, 1987; and "Salem Nuclear Generating Station Safeguards Contingency Plan," with revisions submitted through December 2, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. In accordance with the requirement imposed by the October 8, 1976, order of the United States Court of Appeals for the District of Columbia Circuit in Natural Resources Defense Council v. Nuclear Regulatory Commission, No. 74-1385 and 74-1586, that the Nuclear Regulatory Commission "shall make any licenses granted between July 21, 1976 and such time when the mandate is issued subject to the outcome of the proceedings herein," the license amendment issued herein shall be subject to the outcome of such proceedings.
- G. Prior to startup following the first regularly scheduled refueling outage, Public Service Electric and Gas Company shall install, to the satisfaction of the Commission, a long-term means of protection against reactor coolant system over-pressurization when water-solid.
- H. This amended license is effective as of the date of its issuance. Facility Operating License No. DPR-70, as amended, shall expire at midnight, August 13, 2016.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 3, 1987, as supplemented August 10, 1990, August 21, 1990, and May 22, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-75 is hereby amended by changing paragraph 2.M. as follows*:

M. This license is effective as of its date of issuance and shall expire at midnight April 18, 2020.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Page 23 of License

Date of Issuance: June 5, 1991

*Page 23 of the License is attached, for convenience, for the composite License to reflect this change.

- H. If PSE&G plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the Salem Nuclear Generating Station, the NRC shall be notified in writing regardless of whether the change affects the amount of radioactivity in effluents.
- I. PSE&G shall report any violations of the requirements contained in Section 2, Items C.(3) through C.(25), E., F., and G of this license within 24 hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the Director of the Regional Office, or his designee, no later than the first working day following the violation, with a written-followup report within 14 days.
- J. The licensees shall immediately notify the Commission of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- K. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended to cover public liability claims.
- L. The licensee is authorized to defer certain eighteen-month surveillance items from the dates required by Technical Specifications 4.0.2(a) and 4.7.10.2(c). These surveillances shall be completed prior to startup following the first refueling outage. The provisions of Technical Specifications 4.0.2(b) and 4.7.10.2(c) are not changed. The affected items are identified in the Safety Evaluation accompanying Amendment No. 14 issued October 22, 1982 and this license change.
- M. This license is effective as of the date of issuance and shall expire at midnight April 18, 2020.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by Edson G. Case

Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation

Attachment:
Appendices A & B

Date of Issuance: May 20, 1981

Amendment No. ~~14, 15~~, 104



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 125 AND 104 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated August 3, 1987, as supplemented August 10, 1990, August 21, 1990 and May 22, 1991, the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) submitted a request for changes to the Salem Nuclear Generating Station, Unit Nos. 1 and 2, Operating Licenses. The requested changes would extend the expiration date for the Salem Unit 1 Operating License from September 25, 2008 to August 13, 2016 and for the Salem Unit 2 Operating License from September 25, 2008 to April 18, 2020. The original date is 40 years from the date of issuance of the Construction Permit. The revised date is 40 years from the date of issuance of the Operating License. The August 10, 1990, August 21, 1990 and May 22, 1991, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

Section 103.c of the Atomic Energy Act of 1954, (the Act) provides that a license is to be issued for a specified period not exceeding 40 years. As specified in 10 CFR 50.51, each license will be issued for a fixed period of time, to be specified in the license, not to exceed 40 years from the date of issuance. In addition, 10 CFR 50.57 allows the issuance of an operating license pursuant to 10 CFR 50.56 for the full term specified in 10 CFR 50.51 in conformity with the Construction Permit (CP) and when other provisions specified in 10 CFR 50.57 are met. The current term of the license for the Salem Nuclear Generating Station, Units 1 and 2 (Salem), is 40 years commencing with the issuance of the CP. This results in an effective operating life of 32 years for Salem Unit 1 and 28 years for Salem Unit 2. Consistent with the Act and the Commission's regulations, as noted above, the licensee seeks an extension of the Operating License (OL) term for Salem Units 1 and 2 such that the fixed period of the licenses would be 40 years from the date of issuance of the OL.

Current NRC policy is to issue operating licenses for a 40-year term, commencing with the date of issuance of the OL. For Salem Units 1 and 2 these dates of OL issuance were August 13, 1976 and April 18, 1980, respectively. Thus a 40-year term would change the expiration date from September 25, 2008 (for Units 1 and 2) to August 13, 2016, for Salem Unit 1, and April 18, 2020, for Salem Unit 2.

The NRC staff has evaluated the safety issues associated with the issuance of the proposed license amendments which would allow approximately eight and twelve additional years of plant operation for Salem Units 1 and 2, respectively. The issues addressed during the staff's review consist of additional radiation exposure to the licensee's operating staff, impacts on the off-site population and the general aging of plant structures and equipment.

The impact of additional radiation exposure to the facilities' operating staff and the impact on the general population in the vicinity of the Salem nuclear plants are addressed in the NRC staff's Environmental Assessment dated April 30, 1991. The NRC staff concluded in the Environmental Assessment that the annual radiological effects during the additional years of operation that would be authorized by the proposed license amendments are not more than were previously estimated in the Final Environmental Statements, and are acceptable. The impact of the aging of plant structures and equipment are addressed in the following paragraphs.

The licensee's request for extension of the operating licenses is based, in part, on the fact that a 40-year service life was considered during the design and construction of the plants. Although this does not mean that some components will not wear out during the plants' lifetime, design features were incorporated which maximize the inspectability of structures, systems and equipment. Surveillance, inspectability and maintenance practices which were implemented in accordance with the ASME Code for Inservice Inspection and Inservice Testing of Pumps and Valves and the facility Technical Specifications provide assurance that any unexpected degradation in plant equipment will be identified and corrected. The specific provisions and requirements for ASME Code testing are set forth in 10 CFR 50.55a.

Reactor Pressure Vessel

The Salem reactor pressure vessels have been designed and fabricated to meet the requirements of 10 CFR 50.55a and the ASME Boiler and Pressure Vessel Code, Section III, 1965 Edition and all addenda through Winter 1965 for Unit 1, and all addenda through Winter 1966 for Unit 2, including Code Cases which were in effect prior to April 3, 1967. In addition, the vessel meets the requirements of 10 CFR Part 50, Appendices G and H. This was noted in the AEC Safety Evaluation Report (SER), dated October 11, 1974.

In its January 20, 1986 submittal in response to the final rule on Pressurized Thermal Shock (PTS), the licensee concluded that a 40-year service life will not result in reactor vessel RT_{PTS} values in excess of the NRC screening criteria. The staff reviewed the January 20, 1986 submittal in the course of

this license amendment request and in a letter dated April 10, 1990 requested that the licensee confirm the data concerning the reported RT_{PTS} values based on 32 effective full power years of operation. In an August 10, 1990 letter the licensee responded to the request by providing tables with current estimates for vessel fluence and vessel age for Salem Units 1 and 2. In a May 22, 1991 letter the licensee clarified their response to the staff's April 10, 1990 request. In their May 22, 1991 letter, the licensee stated that their analyses, based on current fluence estimates, verified that the 40-year service life will not result in a reactor vessel RT_{PTS} values in excess of the NRC screening criteria of 10 CFR 50.61. Based on our review of the licensee's above referenced submittals, the staff finds that the reactor vessels meet the criteria of 10 CFR 50.61 for the requested license extension. In addition to these calculations, surveillance capsules placed inside the reactor vessels provide a means of monitoring the cumulative effects of power operation.

Environmental Qualification of Electrical Equipment

Salem has in place a program for the environmental qualification of safety-related electrical equipment (EQ Program) in compliance with 10 CFR 50.49. This program was described in a February 19, 1985 letter from the licensee. As noted in Inspection Report Nos. 50-272/86-23 and 50-272-311/86-23, dated November 13, 1986, the NRC has inspected the Salem EQ Program and found it to be implemented in accordance with Section 50.49, though some deficiencies were identified. As noted in a May 22, 1989 letter from the licensee all of the deficiencies noted during the EQ inspection have been corrected.

All safety-related electrical equipment required for accident mitigation has been reviewed and analyzed, by the licensee, in accordance with the guidance provided in Regulatory Guide 1.97, IE Bulletin 79-01B, DOR Guidelines and NUREG-0588. Those analyses assumed a 40-year operating life. Any equipment aging concerns have been identified and incorporated into the appropriate equipment maintenance and replacement programs.

The staff has concluded, and the EQ inspection team has verified, that the licensee has implemented an EQ Program meeting the requirements of 10 CFR 50.49 and that such a program will be unaffected by the extension of the operating license to 40 years from the date of issuance of the OL.

Mechanical Equipment

Surveillance, maintenance and testing requirements for mechanical equipment are in place to verify operability of the equipment or detect degradation and ensure that equipment that does degrade is replaced or other corrective action is taken. In addition, subcomponents such as nonmetallics (gaskets, O-rings) are inspected and replaced, as necessary, as part of routine maintenance in order to ensure the design life of the equipment. Surveillance, inspection and testing requirements at Salem include the following:

1. ASME Code Section XI Equipment that is ASME Code Class 1, 2 or 3 is subject to the Inservice Testing and Inservice Inspection requirements of ASME Code Section XI and 10 CFR 50.55a. This includes hydrostatic and leakage testing of the reactor coolant pressure boundary, inspections of a representative sample of pressure retaining welds, inservice performance testing of pumps and valves and inservice testing of certain supports. These requirements apply throughout the operating life of a facility and will provide reasonable assurance that mechanical equipment will be properly monitored throughout the operating life.
2. Technical Specifications Equipment covered by Technical Specifications is subject to the surveillance and testing requirements of the applicable Technical Specifications, with specified testing and surveillance intervals. These surveillance requirements include calibration and inspection of systems and components to ensure that operation of the plant will remain in accordance with Limiting Conditions for Operation, as well as requirements for maintaining the structural integrity of reactor coolant system components.
3. 10 CFR Part 50, Appendix J Equipment and components associated with containment penetrations, including containment isolation valves, are subject to leak rate testing under 10 CFR Part 50, Appendix J. This includes local leak rate testing (Type B and C) of penetrations as well as Integrated Leak Rate Tests (Type A) to verify overall containment integrity.

In addition to the above programs, the licensee has initiated programs to address identified concerns with items of mechanical equipment. For example, the Integrated Performance Assessment Team Inspection (IPAT), discussed in NRC Region I Inspection Report Nos. 50-272/90-81, 50-311/90-81 dated July 3, 1990 identified the Configuration Baseline Documentation project as a noteworthy long-term initiative.

From our evaluation we conclude that compliance with the codes, standards, and regulatory requirements to which the mechanical equipment for the Salem Units were originally analyzed, constructed, tested and inspected provide adequate assurance that the structural integrity of components important to safety will be maintained during the additional periods authorized by this amendment. Any significant degradation by an active mechanism would be discovered and the mechanical equipment or component restored to an acceptable condition. Therefore, the age of the mechanical equipment or component should not be a consideration in the extension of the operating license.

Structures

The design and construction of structures and supports was in accordance with various codes and standards applicable at the time of plant construction. The design bases, fabrication, construction, and quality assurance criteria for the plant were reviewed by the staff and presented in the SER. Industrial experience with such structures and supports confirms that a service life in excess of 40 years can be anticipated.

The design of the concrete containment was based on the ultimate strength provisions of the American Concrete Institute (ACI) 318-63 Code. The steel liner was designed on the basis of allowable stresses using the criteria given in Section III of the ASME Boiler and Pressure Vessel Code, 1968 edition. Section 3.8.2 of the staff's SER states that the Class I structures and equipment were designed and analyzed in accordance with ACI 318-63 Code allowable values for reinforced concrete and the American Institute of Steel Construction (AISC) Code allowable values for structural steel.

Throughout the service life of the units the containment structure is subject to the inspection and testing program of Appendix J. The Appendix J program requires three Type A Integrated Leak Rate Tests (ILRT) during every ten-year cycle. This program includes visual examination of both interior and exterior surfaces of the containment for any indications of degradation affecting structural integrity.

The Appendix J leak rate testing program is well documented and provides reasonable assurance that containment structural integrity remains adequate throughout the service life of the facility, including the proposed extension period.

The staff has reviewed the above considerations on structures and has concluded that the plant structures will not be adversely affected by the proposed extension of the operating license to 40 years from the date of issuance of the OL.

In summary, the staff finds that extension of the operating license for the Salem Nuclear Generating Station, Units 1 and 2, to allow a 40-year service life is consistent with the Final Environmental Statement and Safety Evaluation Report for the plant and that the Commission's previous findings are not changed.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published (56 FR 20628) in the FEDERAL REGISTER on May 6, 1991. Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of the amendments will not have a significant effect on the quality of the human environment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Dembek

Date: June 5, 1991