

July 1, 1991

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: ADMINISTRATIVE CHANGES TO TECHNICAL SPECIFICATIONS, SALEM NUCLEAR
GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. 75125/75126 AND
79706/79707)

The Commission has issued the enclosed Amendment Nos.127 and 106 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Unit Nos. 1 and 2. These amendments consist of changes to the
Technical Specifications (TSs) in response to your two applications both
dated April 3, 1991.

These amendments correct administrative errors in the technical specifications.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice. You are
requested to notify the NRC, in writing, when the amendments have been
implemented at Salem 1 and 2.

Sincerely,

/s/

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 127 to License No. DPR-70
2. Amendment No. 106 to License No. DPR-75
3. Safety Evaluation

cc w/enclosures: 9107180263 910701
 See next page PDR ADDCK 05000272
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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

July 1, 1991

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Vice President and Chief Nuclear
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Public Service Electric & Gas Company
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requested to notify the NRC, in writing, when the amendments have been
implemented at Salem 1 and 2.

Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 127 to
License No. DPR-70
2. Amendment No. 106 to
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

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Public Service Commission of Maryland
Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The applications for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) both dated April 3, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 1, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 127

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Page

3/4 10-2

Insert Page

3/4 10-2

SPECIAL TEST EXCEPTIONS

GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS

LIMITING CONDITION FOR OPERATION
=====

3.10.2 The group height, insertion and power distribution limits of Specifications 3.1.3.1, 3.1.3.4, 3.1.3.5, 3.2.1, and 3.2.4 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER is maintained \leq 85% of RATED THERMAL POWER, and
- b. The limits of Specifications 3.2.2 and 3.2.3 are maintained and determined at the frequencies specified in Specification 4.10.2.2 below.

APPLICABILITY: MODE 1

ACTION:

With any of the limits of Specifications 3.2.2 or 3.2.3 being exceeded while the requirements of Specifications 3.1.3.1, 3.1.3.4, 3.1.3.5, 3.2.1 and 3.2.4 are suspended, either:

- a. Reduce THERMAL POWER sufficient to satisfy the ACTION requirements of Specifications 3.2.2 and 3.2.3, or
- b. Be in HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS
=====

4.10.2.1 The THERMAL POWER shall be determined to be \leq 85% of RATED THERMAL POWER at least once per hour during PHYSICS TESTS.

4.10.2.2 The below listed surveillance requirements shall be performed at least once per 12 hours during PHYSICS TESTS:

- a. Surveillances 4.2.2.2 and 4.2.2.3.
- b. Surveillances 4.2.3.1 and 4.2.3.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The applications for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) both dated April 3, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 106, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 1, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 106

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

3/4 2-17

3/4 3-40

3/4 4-17

3/4 4-18

3/4 4-19

3/4 10-2

Insert Pages

3/4 2-17

3/4 3-40

3/4 4-17

3/4 4-18

3/4 4-19

3/4 10-2

TABLE 3.2-1

DNB PARAMETERS

<u>PARAMETER</u>	<u>LIMITS</u>
	<u>4 Loops in Operation</u>
Reactor Coolant System T_{avg}	$\leq 582^{\circ}\text{F}$
Pressurizer Pressure	$\geq 2220 \text{ psia}^*$
Reactor Coolant System	$\geq 357200 \text{ gpm}^{\#}$

* Limit not applicable during either a THERMAL POWER ramp in excess of 5% RATED THERMAL POWER per minute or a THERMAL POWER step in excess of 10% RATED THERMAL POWER.

Includes a 2.2% flow uncertainty plus a 0.1% measurement uncertainty due to feedwater venturi fouling.

TABLE 3.3-6 (Continued)

TABLE NOTATION

- ACTION 23 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 24 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.7.1.
- ACTION 25 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 26 - With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours, and:
- 1) either restore the inoperable Channel(s) to OPERABLE status within 7 days of the event, or
 - 2) prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

REACTOR COOLANT SYSTEM
OPERATIONAL LEAKAGE
LIMITING CONDITION FOR OPERATION

=====

3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 40 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2230 ± 20 psig.
- f. 1 GPM leakage at a Reactor Coolant System pressure of 2230 ± 20 psig from any Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE and leakage from Reactor Coolant System Pressure Isolation Valves, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

=====

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by;

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment sump inventory at least once per 12 hours.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

=====

- c. Measurement of the CONTROLLED LEAKAGE from the reactor coolant pump seals at least once per 31 days when the Reactor Coolant System pressure is 2230 ± 20 psig and valve 2CV71 is fully closed,
- d. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours. The water inventory balance shall be performed with the plant at steady state conditions. The provisions of specification 4.0.4 are not applicable for entry into Mode 4, and
- e. Monitoring the reactor head flange leakoff system at least once per 24 hours.

4.4.7.2.2 Each Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1 shall be demonstrated OPERABLE pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:

- a. At least once per 18 months.
- b. Prior to entering MODE 2 whenever the plant has been in COLD SHUTDOWN for 72 hours or more and if leakage testing has not been performed in the previous 9 months.
- c. Prior to returning the valve to service following maintenance repair or replacement work on the valve.
- d. For the Residual Heat Removal and Safety Injection Systems hot and cold leg injection valves and accumulator valves listed in Table 3.4-1 the testing will be done within 24 hours following valve actuation due to automatic or manual action or flow through the valve. For all other systems testing will be done once per refueling.

The provisions of specification 4.0.4 are not applicable for entry into MODE 3 or 4.

REACTOR COOLANT SYSTEM

TABLE 3.4-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

<u>VALVE NO</u>	<u>FUNCTION</u>
21SJ43	Safety Injection (L.P. from RHR Pumps to Cold Legs)
22SJ43	Safety Injection (L.P. from RHR Pumps to Cold Legs)
23SJ43	Safety Injection (L.P. from RHR Pumps to Cold Legs)
24SJ43	Safety Injection (L.P. from RHR Pumps to Cold Legs)
21SJ55	Safety Injection (Accumulator Discharge to Cold Legs)
22SJ55	Safety Injection (Accumulator Discharge to Cold Legs)
23SJ55	Safety Injection (Accumulator Discharge to Cold Legs)
24SJ55	Safety Injection (Accumulator Discharge to Cold Legs)
21SJ56	Safety Injection (Accumulator Discharge to Cold Legs)
22SJ56	Safety Injection (Accumulator Discharge to Cold Legs)
23SJ56	Safety Injection (Accumulator Discharge to Cold Legs)
24SJ56	Safety Injection (Accumulator Discharge to Cold Legs)
21SJ17	Safety Injection (Boron Injection to Cold Legs)
22SJ17	Safety Injection (Boron Injection to Cold Legs)
23SJ17	Safety Injection (Boron Injection to Cold Legs)
24SJ17	Safety Injection (Boron Injection to Cold Legs)
2SJ150	Safety Injection (Boron Injection to Cold Legs)
21SJ139	Safety Injection (H.P. from SI Pumps to Hot Legs)
22SJ139	Safety Injection (H.P. from SI Pumps to Hot Legs)
23SJ139	Safety Injection (H.P. from SI Pumps to Hot Legs)
24SJ139	Safety Injection (H.P. from SI Pumps to Hot Legs)
21SJ156	Safety Injection (H.P. from SI Pumps to Hot Legs)
22SJ156	Safety Injection (H.P. from SI Pumps to Hot Legs)
23SJ156	Safety Injection (H.P. from SI Pumps to Hot Legs)
24SJ156	Safety Injection (H.P. from SI Pumps to Hot Legs)
21SJ144	Safety Injection (H.P. from SI Pumps to Cold Legs)
22SJ144	Safety Injection (H.P. from SI Pumps to Cold Legs)
23SJ144	Safety Injection (H.P. from SI Pumps to Cold Legs)
24SJ144	Safety Injection (H.P. from SI Pumps to Cold Legs)
2RH1	RHR Suction from Hot Leg No. 21
2RH2	RHR Suction from Hot Leg No. 21
23RH27	RHR Discharge to Hot Leg No. 23
24RH27	RHR Discharge to Hot Leg No. 24

SPECIAL TEST EXCEPTIONS

GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS

LIMITING CONDITION FOR OPERATION
=====

3.10.2 The group height, insertion and power distribution limits of Specifications 3.1.3.1, 3.1.3.4, 3.1.3.5, 3.2.1, and 3.2.4 may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER is maintained less than or equal to 85%[#] of RATED THERMAL POWER, and
- b. The limits of Specifications 3.2.2 and 3.2.3 are maintained and determined at the frequencies specified in Specification 4.10.2.2 below.

APPLICABILITY: MODE 1.

ACTION:

With any of the limits of Specifications 3.2.2 or 3.2.3 being exceeded while the requirements of Specifications 3.1.3.1, 3.1.3.4, 3.1.3.5, 3.2.1 and 3.2.4 are suspended, either:

- a. Reduce THERMAL POWER sufficient to satisfy the ACTION requirements of Specifications 3.2.2 and 3.2.3, or
- b. Be in HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS
=====

4.10.2.1 The THERMAL POWER shall be determined to be less than or equal to 85% of RATED THERMAL POWER at least once per hour during PHYSICS TESTS.

4.10.2.2 The below listed surveillance requirements shall be performed at least once per 12 hours during PHYSICS TESTS:

- a. Surveillances 4.2.2.2 and 4.2.2.3.
- b. Surveillances 4.2.3.1 and 4.2.3.2.

[#] See page 3/4 10-3



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 127 AND 106 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letters dated April 3, 1991, the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) submitted two requests for changes to the Salem Nuclear Generating Station, Unit Nos. 1 and 2, Technical Specifications (TS). The requested changes would correct administrative errors in the TS.

2.0 EVALUATION

a. The April 3, 1991, application (licensee no. NLR-N91032) revised a previous application dated February 4, 1991 (licensee no. NLR-N90239) in its entirety. The changes requested in the April 3, 1991 application are evaluated below:

1. Specification 4.10.2.2, for Salem 1 and 2, is currently written as follows:

"The Surveillance Requirements of Specifications 4.2.2 and 4.2.3 shall be performed at the following frequencies during PHYSICS TESTS:

a. Specification 4.2.2 - At least once per 12 hours.

b. Specification 4.2.3 - At least once per 12 hours.

The proposed change would modify Specification 4.10.2.2, for Salem 1 and 2, to the following:

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P PDR

"The below listed surveillance requirements shall be performed at least once per 12 hours during PHYSICS TESTS:

- a. Surveillance 4.2.2.2 and 4.2.2.3.
- b. Surveillance 4.2.3.1 and 4.2.3.2."

This change clarifies the wording of the specification and references the specific surveillances to be performed. There is no change in the surveillances that are performed. This is an administrative change and the staff finds it acceptable.

2. For Salem Unit 2, Table 3.2-1, DNB PARAMETERS, has the Pressurizer Pressure limit listed as \leq 2220 psia* (less than or equal to 2220 psia*). This is incorrect. The Pressurizer Pressure limit should be \geq 2220 psia* (greater than or equal to 2220 psia*). Pressure must be maintained equal to or above 2220 psia to maintain an adequate margin to the departure from nucleate boiling (DNB). This was apparently a typographical error in the original issuance of the TS. The staff finds this change acceptable.
- b. The April 3, 1991 application (licensee no. NLR-N91046) superseded a previous application dated September 13, 1989 (licensee no. NLR-N89033), and supplements dated July 10, 1990, November 6, 1990, and December 27, 1990. The September 13, 1989 application superseded the previous application dated August 29, 1986. The April 3, 1991 application consolidated all the requested changes into one application. These requested changes are evaluated below:
1. For Salem Unit 2, Table 3.3-6, TABLE NOTATION, ACTION 24, which is applicable to an inoperable radiation monitoring system channel in the reactor coolant leakage detection system, references Specification 3.4.6.1 which is applicable to steam generators. This reference is being changed to 3.4.7.1, which is the specification for reactor coolant leak detection. This is an administrative change and the staff finds it acceptable.
 2. For Salem Unit 2, there are two surveillances numbered 4.4.7.2. These are being renumbered 4.4.7.2.1 and 4.4.7.2.2. This is an administrative change and the staff finds it acceptable.
 3. For Salem Unit 2, Table 3.4-1 describes the function of the SJ144 valves as "Safety Injection (H.P. from SI Pumps to Hot Legs)." These valves are in the lines to the cold legs and should be described as "Safety Injection (H.P. from SI pumps to Cold Legs)". This is an administrative change and the staff finds it acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: James Stone

Date: July 1, 1991