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JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

June 14, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index
FIG Index
AP-RCS.3, Rev 8
FIG-14.0, Rev 1

A045

PARAMETERS: DOC TYPES - PROPS PRFIG PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	014	01/09/01	05/01/98	05/01/03	EF
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	014	05/18/00	08/17/99	08/17/04	EF
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	012	05/18/00	08/17/99	08/17/04	EF
AP-CR.1	CONTROL ROOM INACCESSIBILITY	017	05/11/01	01/11/00	01/11/05	EF
AP-CVCS.1	CVCS LEAK	012	05/01/98	05/01/98	05/01/03	EF
AP-CVCS.3	LOSS OF ALL CHARGING FLOW	002	02/11/00	02/26/99	02/26/04	EF
AP-CW.1	LOSS OF A CIRC WATER PUMP	010	07/16/98	05/01/98	05/01/03	EF
AP-ELEC.1	LOSS OF 12A AND/OR 12B BUSSES	020	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.2	SAFEGUARD BUSSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	009	03/22/99	03/22/99	03/22/04	EF
AP-ELEC.3	LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350 F)	008	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	003	03/15/01	06/09/97	06/09/02	EF
AP-ELEC.17/18	LOSS OF SAFEGUARDS BUS 17/18	002	10/18/99	06/09/97	06/09/02	EF
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	012	02/11/00	02/27/98	02/27/03	EF
AP-IA.1	LOSS OF INSTRUMENT AIR	017	12/02/99	05/01/98	05/01/03	EF
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	012	03/26/01	12/02/99	12/02/04	EF
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	007	05/22/01	05/14/98	05/14/03	EF
AP-RCC.2	RCC/RPI MALFUNCTION	008	11/16/98	02/06/97	02/06/02	EF
AP-RCC.3	DROPPED ROD RECOVERY	004	11/16/98	02/27/98	02/27/03	EF
AP-RCP.1	RCP SEAL MALFUNCTION	013	06/09/00	05/01/98	05/01/03	EF
AP-RCS.1	REACTOR COOLANT LEAK	015	09/08/00	05/01/98	05/01/03	EF
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	010	12/14/98	05/01/98	05/01/03	EF
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	008	06/14/01	08/05/97	08/05/02	EF
AP-RCS.4	SHUTDOWN LOCA	011	12/02/99	05/01/98	05/01/03	EF
AP-RHR.1	LOSS OF RHR	015	02/08/01	05/01/98	05/01/03	EF

PARAMETERS: DOC TYPES - PROPS PRFIG PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	009	10/13/00	03/31/00	03/31/05	EF
AP-SG.1	STEAM GENERATOR TUBE LEAK	000	09/08/00	09/08/00	09/08/05	EF
AP-SW.1	SERVICE WATER LEAK	015	10/18/99	06/03/98	06/03/03	EF
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	010	02/12/99	10/10/97	10/10/02	EF
AP-TURB.2	TURBINE LOAD REJECTION	017	02/11/00	05/13/98	05/13/03	EF
AP-TURB.3	TURBINE VIBRATION	010	02/11/00	02/10/98	02/10/03	EF
AP-TURB.4	LOSS OF CONDENSER VACUUM	014	05/01/98	05/01/98	05/01/03	EF
AP-TURB.5	RAPID LOAD REDUCTION	005	06/09/00	06/09/00	06/09/05	EF

TOTAL FOR PRAP 32

PARAMETERS: DOC TYPES - PROPS PRFIG PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
FIG-1.0	FIGURE MIN SUBCOOLING	000	05/01/98	05/01/98	05/01/03	EF
FIG-2.0	FIGURE SDM	002	10/13/00	05/01/98	05/01/03	EF
FIG-3.0	FIGURE NAT CIRC C/D WITH SHROUD FANS	000	05/01/98	05/01/98	05/01/03	EF
FIG-3.1	FIGURE NAT CIRC C/D WITHOUT SHROUD FANS	001	02/08/01	05/01/98	05/01/03	EF
FIG-3.2	FIGURE NC C/D WITH VOID IN UPPER HEAD	000	05/01/98	05/01/98	05/01/03	EF
FIG-4.0	FIGURE RCP SEAL LEAKOFF	002	02/28/01	05/01/98	05/01/03	EF
FIG-5.0	FIGURE RHR INJECTION	000	05/01/98	05/01/98	05/01/03	EF
FIG-6.0	FIGURE MIN RCS INJECTION	000	05/01/98	05/01/98	05/01/03	EF
FIG-7.0	FIGURE INTACT S/G PRESSURE	001	05/18/98	05/01/98	05/01/03	EF
FIG-8.0	FIGURE TSAT	000	05/01/98	05/01/98	05/01/03	EF
FIG-9.0	FIGURE TECH SPEC C/D	001	02/15/01	05/01/98	05/01/03	EF
FIG-9.1	FIGURE C/D LIMITS	000	05/01/98	05/01/98	05/01/03	EF
FIG-10.0	FIGURE LIMIT A	000	05/01/98	05/01/98	05/01/03	EF
FIG-11.0	FIGURE SOAK LIMITS	000	05/01/98	05/01/98	05/01/03	EF
FIG-12.0	FIGURE CNMT HYDROGEN	000	05/01/98	05/01/98	05/01/03	EF
FIG-13.0	FIGURE BACK PRESSURE	000	05/01/98	05/01/98	05/01/03	EF
FIG-14.0	FIGURE IA ISOL	001	06/14/01	05/01/98	05/01/03	EF

TOTAL FOR PRFIG 17

ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

RESPONSIBLE MANAGER

6-14-2001

EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: _____

A. PURPOSE - This procedure provides guidance necessary to operate the plant with indication of high reactor coolant activity.

B. ENTRY CONDITIONS/SYMPTOMS

1. SYMPTOMS - The symptoms of HIGH REACTOR COOLANT ACTIVITY are;

a. Unexplained increase in letdown line monitor, R-9, or

b. Sampling indicates I-131 equivalent GREATER THAN .64 µCi/gm at 40 gpm letdown or GREATER THAN .46 µCi/gm at 60 gpm letdown or

c. Sampling indicates gross degassed activity GREATER THAN 20 µCi/gm, or

d. Sampling indicates that total specific activity exceeds 100/E.

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED

* *

CAUTION

- IF LETDOWN FLOW EXCEEDS 60 GPM, THEN LOCALLY MONITOR D/P ACROSS THE CVCS DI(S) TO VERIFY THAT FLOW IS CONTINUING AND THAT RELIEF VALVE, V-209, HAS NOT LIFTED.
- LETDOWN FLOW THROUGH THE DI'S SHOULD BE LIMITED TO 90 GPM.

* *

NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION.

1 Verify RCS Activity:

a. Direct RP Tech to sample RCS for activity

b. RCS activity - GREATER THAN NORMAL (Check with RP Department for normal activity)

b. IF normal activity verified, THEN direct I&C to check operability of R-9, letdown line monitor, AND return to normal operations.

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2	Increase Letdown Flow To 60 GPM:	
	a. Verify deborating DI isolated - DIVERT VLV CATION DEBOR DI AOV-244 IN BYPASS POSITION	a. Place AOV-244 in bypass position.
	b. Place letdown controllers in MANUAL at 60% open • TCV-130 • PCV-135	
	c. Increase letdown flow as follows: 1) Close letdown orifice valve (AOV-200A or AOV-200B) 2) Immediately open 60 gpm letdown orifice valve, AOV-202	
	d. Adjust low pressure letdown pressure to approximately 250 psig	
	e. Place TCV-130 in AUTO at 105°F	
	f. Place PCV-135 in AUTO at 250 psig	
	g. Adjust charging pump speed and HCV-142 as necessary	

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3	Check Letdown Line Monitor, R-9 - LESS THAN 200 MR/HR ABOVE BACKGROUND	Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).

* *

CAUTION

PLACING A NEW DI IN SERVICE MAY RESULT IN A POSITIVE OR NEGATIVE REACTIVITY ADDITION DUE TO A BORON CHANGE.

* *

4	Direct RP Tech To Sample Letdown DI Efficiency - DECONTAMINATION FACTOR GREATER THAN 10	IF DI efficiency is NOT acceptable, THEN place a new mixed bed in service (Refer to S-3.2B, PLACING A MIXED BED DEMINERALIZER IN SERVICE - BORON CONCENTRATION DIFFERENT THAN RCS).
5	Evaluate AUX BLDG Radiation Levels:	
	a. Direct RP Tech to survey AUX BLDG	
	b. Check AUX BLDG radiation monitors - NORMAL • R-4 • R-9 • R-10B • R-13 • R-14	b. Perform the following: 1) Direct RP Tech to survey AUX BLDG areas as necessary. 2) Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	Determine If Plant Operation Can Continue (Consult Plant staff if necessary) - OPERATION CAN CONTINUE	IF plant shutdown is required, THEN refer to 0-2.1, NORMAL SHUTDOWN TO HOT SHUTDOWN.

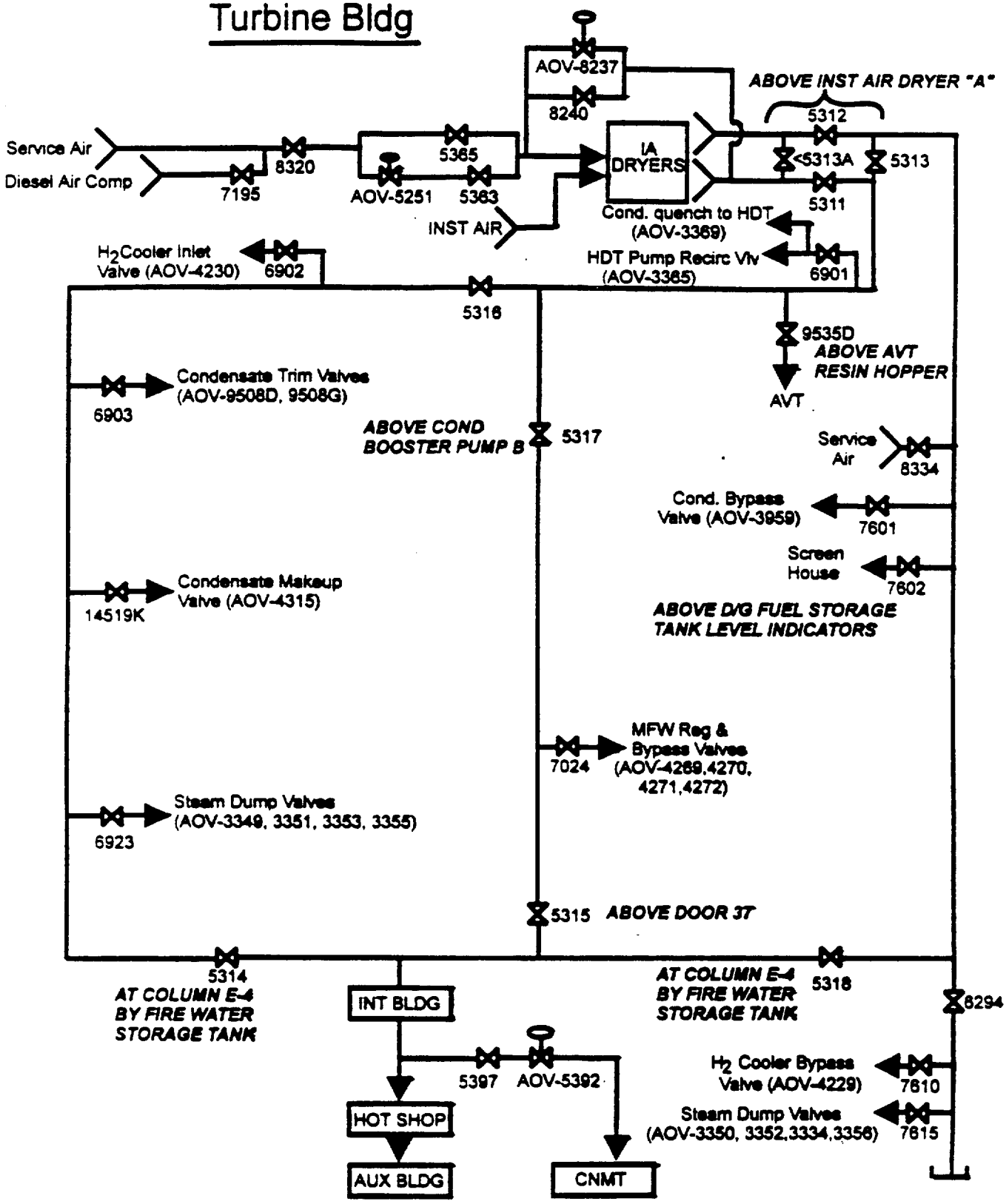
NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

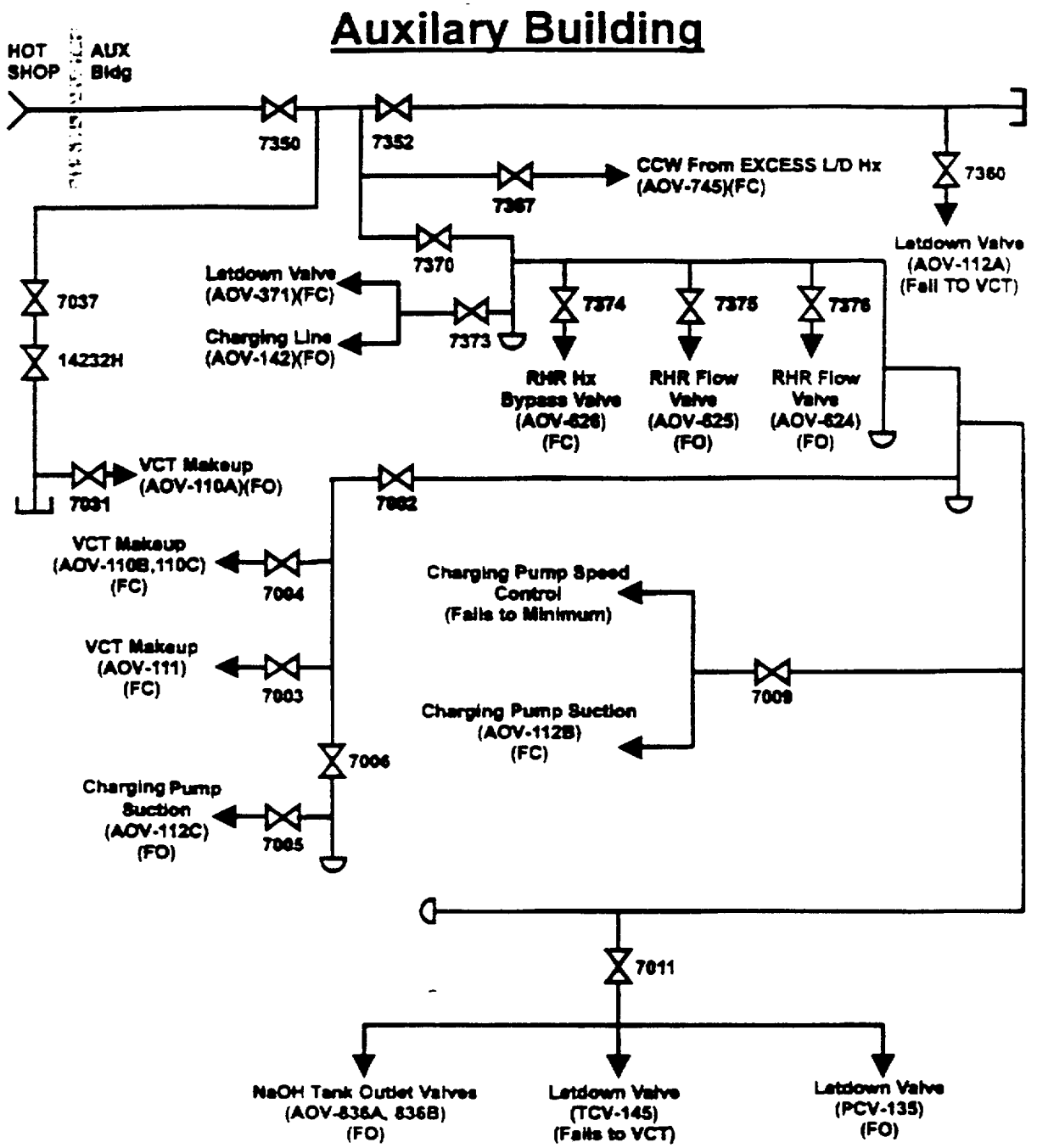
7 Notify Higher Supervision

-END-

EOP:	TITLE:	REV: 1
FIG-14.0	FIGURE IA ISOL	PAGE 1 of 2

Responsible Manager _____ Date 6-14-2001




Auxilary Building
HOT SHOP
AUX Bldg
7350
7352
7360
CCW From EXCESS L/D Hx (AOV-745)(FC)
7367
Letdown Valve (AOV-112A) (Fail TO VCT)
7370
Letdown Valve (AOV-371)(FC)
Charging Line (AOV-142)(FO)
7037
14232H
7373
7374
7375
7376
RHR Hx Bypass Valve (AOV-626) (FC)
RHR Flow Valve (AOV-625) (FO)
RHR Flow Valve (AOV-624) (FO)
VCT Makeup (AOV-110A)(FO)
7091
7092
VCT Makeup (AOV-110B,110C) (FC)
7004
Charging Pump Speed Control (Fails to Minimum)
VCT Makeup (AOV-111) (FC)
7003
7009
Charging Pump Suction (AOV-112B) (FC)
7006
Charging Pump Suction (AOV-112C) (FO)
7005
7011
NaOH Tank Outlet Valves (AOV-836A, 836B) (FO)
Letdown Valve (TCV-145) (Fails to VCT)
Letdown Valve (PCV-135) (FO)