Docket No. 50-271

Mr. L. A. Tremblay Licensing Engineer Vermont Yankee Nuclear Power Corporation 580 Main Street Bolton, Massachusetts 01740-1398

Dear Mr. Tremblay:

SUBJECT: CORRECTION TO AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. DPR-28

By letter dated September 15, 1989 we issued Amendment No. 116 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station.

Page 209a of the Technical Specifications, in the list of previously approved reports, shows that the report entitled, "Methods for the Analysis of Guide Fuel Rod Steady-State Thermal Effects (FROSSTEY)" is properly numbered as "YAEC-1249P." However, this same report, also referenced in our Safety Evaluation, on page 2 under (3)(d) is improperly shown as "YAEC-1242P." Kindly make a pen and ink change in the Safety Evaluation showing this as "YAEC-1249P."

In addition, we inadvertently sent you a revised page 209 of the Technical Specifications (TS) that omitted the change in the TS references to Linear Heat Generation Rates that are provided by page 2 - Item (2)(d) of our Safety Evaluation. We have enclosed a replacement page 209 to License Amendment No. 116 that properly reflects our Safety Evaluation in that it adds the Linear Heat Generation Rates references.

These two minor corrections will bring our Safety Evaluation and revised TS pages into agreement.

Sincerely,

Original signed by:

Morton B. Fairtile, Project Manager Project Directorate I-3 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Enclosure: TS Page 209

cc w/encl: See next page

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Mr. L. A. Tremblay

cc:

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Adjudicatory File (2)
Atomic Safety and Licensing Board
Panel Docket
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Robert M. Lazo, Chairman Atomic Safety and Licensing Board U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Frederick J. Shon Administrative Judge Atomic Safety and Licensing Board U. S. Nuclear Regulatory Commission

Jerry Harbour Administrative Judge Atomic Safety and Licensing Board U. S. Nuclear Regulatory Commission Washington, D.C. 20555

CORRECTION TO AMENDMENT NO. 116 FACILITY OPERATING LICENSE NO. DPR-28

DISTRIBUTION:

Docket File

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2. Annual Report

An annual report covering the previous calendar year shall be submitted prior to March 1 of each year. The annual report shall include a tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, 1/e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD or film badge measurement. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

3. Monthly Statistical Report

Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the fifteenth of each month following the calendar month covered by the report. These reports shall include a narrative summary of operating experience during the report period which describes the operation of the facility.

4. Core Operating Limits Report

The core operating limits shall be established and documented in the Core Operating Limits Report (COLR) before each reload cycle or any remaining part of a reload cycle for the following:
(a) The Average Planar Linear Heat Generation Rates (APLHGR) for Specifications 3.11.A and 3.6.G.la, (b) The K_f core flow adjustment factor for Specification 3.11.C., (c) The Minimum Critical Power Ratio (MCPR) for Specifications 3.11.C and 3.6.G.la, and (d) The Linear Heat Generation Rates (LHGR) for Specifications 2.1.A.la, 2.1.B.l, and 3.11.B. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in:

Report, E. E. Pilat, "Methods for the Analysis of Boiling Water Reactors Lattice Physics," YAEC-1232, December 1980 (Approved by NRC SER, dated September 15, 1982).

Report, D. M. VerPlanck, "Methods for the Analysis of Boiling Water Reactors Steady State Core Physics," YAEC-1238, March 1981 (Approved by NRC, SER, dated September 15, 1982).

^{1/} This tabulation supplements the requirements of 20.407 of 10CFR Part 20.