February 13, 1991

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

SUBJECT: STEAM GENERATOR SURVEILLANCE REQUIREMENTS, SALEM GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. 75172 AND 75173)

Dear Mr. Miltenberger:

The Commission has issued the enclosed Amendment Nos. 118 and 98 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 13, 1989 and supplemented by letters dated June 29, 1990 and December 28, 1990. The two supplemental letters did not increase the scope of the original amendment request and did not affect the staff's original no significant hazards determination.

These amendments modify Salem Unit 1 Steam Generator Surveillance Requirements 4.4.5.2 through 4.4.5.5 for the purpose of consistency with the Unit 2 surveillances and the Westinghouse Standard Technical Specifications. Salem Unit 2 Section 4.4.6.5.b. has been revised to clarify that the inservice inspection report shall be submitted in the Annual Operating Report.

You are requested to notify the Commission, in writing, when the enclosed amendments are implemented at Salem Units 1 and 2.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/
James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9102210090 910213
PDR ADOCK 05000272
P PDR

Enclosures:
1. Amendment No. 118 to License No. DPR-70
2. Amendment No. 98 to License No. DPR-75
3. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION w/enclosures:

Docket File	MO'Brien (2)	Wanda Jones	JXXXXSVarga
NRC PDR	OGC	JCalvo	RBlough
Local PDR	DHagan	ACRS (10)	BXXXXXEGreenman
PDI-2 Reading	EJordan	JRaleigh	WButler
GPA/PA	JStone	MFranovich	GHill (8)
OC/LFMB	RSwXXXXd JWhite		

[TAC 75172/173]

*Previous Concurrence

PDI-2/LA	*PDI-2/GE	PDI-2/PM	OGC	PDI-2/D	EMCB
MO'Brien	MFranovich	JStone	J. Hill	WButler	CCheng
1/23/91	07/17/90	1/23/91	2/1/91	2/12/91	1/23/91

DFO1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 13, 1991

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: STEAM GENERATOR SURVEILLANCE REQUIREMENTS, SALEM GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS. 75172 AND 75173)

The Commission has issued the enclosed Amendment Nos. 118 and 98 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 13, 1989 and supplemented by letters dated June 29, 1990 and December 28, 1990. The two supplemental letters did not increase the scope of the original amendment request and did not affect the staff's original no significant hazards determination.

These amendments modify Salem Unit 1 Steam Generator Surveillance Requirements 4.4.5.2 through 4.4.5.5 for the purpose of consistency with the Unit 2 surveillances and the Westinghouse Standard Technical Specifications. Salem Unit 2 Section 4.4.6.5.b. has been revised to clarify that the inservice inspection report shall be submitted in the Annual Operating Report.

You are requested to notify the Commission, in writing, when the enclosed amendments are implemented at Salem Units 1 and 2.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

James C. Stone

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 118to
License No. DPR-70
2. Amendment No. 98 to
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:
See next page

cc:

Mark J. Wetterhahn, Esquire
Conner and Wetterhahn
Suite 1050
1747 Pennsylvania Avenue, NW
Washington, DC 20006

Richard Fryling, Jr., Esquire
Law Department - Tower 5E
80 Park Place
Newark, NJ 07101

Mr. L. K. Miller
General Manager - Salem Operations
Salem Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. S. LaBruna
Vice President - Nuclear Operations
Nuclear Department
P.O. Box 236
Hancocks Bridge, New Jersey 08038

Mr. Thomas P. Johnson, Senior Resident Inspector
Salem Generating Station
U.S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, NJ 08038

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental Protection
CN 415
Trenton, NJ 08625-0415

Maryland People's Counsel
American Building, 9th Floor
231 East Baltimore Street
Baltimore, Maryland 21202

Mr. J. T. Robb, Director
Joint Owners Affairs
Philadelphia Electric Company
955 Chesterbrook Blvd., 51A-13
Wayne, PA 19087

Richard B. McGlynn, Commission
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, NJ 07102

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Mr. Bruce A. Preston, Manager
Licensing and Regulation
Nuclear Department
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. David Wersan
Assistant Consumer Advocate
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, PA 17120

Mr. Scott B. Ungerer
MGR. - Joint Generation Projects
Atlantic Electric Company
P.O. Box 1500
1199 Black Horse Pike
Pleasantville, NJ 08232

Mr. Jack Urban
General Manager, Fuels Department
Delmarva Power & Light Company
800 King Street
Wilmington, DE 19899

Public Service Commission of Maryland
Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 13, 1989, and supplemented by letters dated June 29, 1990 and December 28, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

9102210099 910213
PDR ADOCK 05000272
P PDR

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/ James C. Stone for

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 13, 1991

PDI-2/LA	PDI-2/GE	PDI-2/PM	OGC	PDI-2/D
MO'Brien	MFranovich	JStone/bah	J. Hill	WButler
1/ /90	7/17/90	7/17/90	2/1/90	2/12/90

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James C. Stone

For Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 13, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 4-7	3/4 4-7
3/4 4-8	3/4 4-8
3/4 4-9	3/4 4-9
3/4 4-10	3/4 4-10
3/4 4-11	3/4 4-11

STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing Tavg above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

b. The first inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

   1. All nonplugged tubes that previously had detectable wall penetrations (>20%), and

   2. Tubes in those areas where experience has indicated potential problems.

   3. A tube inspection (pursuant to Specification 4.4.5.4.a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.

c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial tube inspection provided:

1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.

2. The inspections include those portions of the tubes where imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three categories:

Category	Inspection Results
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.

4.4.5.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional has occurred, the inspection interval may be extended to a maximum of once per 40 months.

b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.5.3.a; the interval may then be extended to a maximum of once per 40 months.

c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions.

1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2,

2. A seismic occurrence greater than the Operating Basis Earthquake,

3. A loss-of-coolant accident requiring actuation of the engineered safeguards, or

4. A main steam line or feedwater line break.

4.4.5.4 Acceptance Criteria

a. As used in this Specification:

1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.

2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.

3. Degraded Tube means a tube containing imperfections $\geq$20% of the nominal wall thickness caused by degradation.

4. % Degradation means the percentage of the tube wall thickness affected or removed by degradation.

5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective.

6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service and is equal to 40% of the nominal tube wall thickness.

7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.3.c, above.

8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.

9. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service establish a baseline condition of the tubing. This inspection shall be performed after the field hydrostatic test and prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.

b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.5.5 Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which the inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9.1 prior to resumption of plant operation. The written followup to this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

THIS PAGE LEFT INTENTIONALLY BLANK

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 13, 1989, and supplemented by letters dated June 29, 1990 and December 28, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/ James C. Stone for

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 13, 1991

PDI-2/LA	PDI-2/GE	PDI-2/PM	OGC	PDI-2/D
MO'Brien	MFranovich	JStone/bah	J. Hull	WButler
7/17/90	7/17/90	7/17/90	2/1/90	2/12/90

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James C. Stone

For Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 13, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 98

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 4-13	3/4 4-13

9. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service establish a baseline condition of the tubing. This inspection shall be performed after the field hydrostatic test and prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.

b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.6.5 Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Operating Report for the period in which the inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9.1 prior to resumption of plant operation. The written followup to this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NOS. 118 AND 98 TO FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated September 13, 1989 and supplemented by letters dated June 29, 1990 and December 28, 1990, Public Service Electric & Gas Company requested an amendment to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. The proposed amendments would revise Salem Unit 1 Steam Generator (S/G) surveillances (Surveillance Requirements 4.4.5.2 through 4.4.5.5) to be consistent with Unit 2 Surveillance Requirements and the Westinghouse Standard Technical Specifications. In addition, the proposed amendments would revise Salem Unit 2 Section 4.4.6.5.b to clarify that the results of the steam generator inspections would be submitted in the Annual Operating Report.

The June 29, 1990 letter provided a corrected Technical Specification page for Salem Unit No. 2 which correctly labeled Section 4.4.6.5. The December 28, 1990 letter provided further detailed information on the results of previous steam generator tube inspections. The supplemental letters did not increase the scope of the original amendment request and did not affect the staff's original no significant hazards determination.

2.0 EVALUATION

The Unit 1 Technical Specifications, Sections 4.4.5.2 through 4.4.5.5, establish acceptance criteria and provide requirements for performing the surveillance inspections of the steam generators via an inservice inspection program. The modifications to Sections 4.4.5.2 through 4.4.5.5 are changes in the wording of the requirements to make them consistent with the Unit 2 Technical Specifications and the Westinghouse Standard Technical Specifications as discussed below.

The Unit 1 Technical Specifications, Section 4.4.5.2, establish criteria for steam generator tube sample selection and inspection. This

9102210111 910213
PDR ADOCK 05000272
P PDR

change provides an additional criterion that if the selected tube does not permit the passage of the eddy current probe, this shall be recorded, and an adjacent tube will be selected for inspection. The criteria for the second and third inspections, if they are necessary, have also been clarified. These changes clarify the requirements for sample selection and tests to be consistent with Unit 2 and the staff finds them acceptable.

Section 4.4.5.3 governs the frequency of S/G inservice inspections for Unit 1. Section 4.4.5.3.b has been changed to require the inspection frequency to be increased to once every 20 months if the inspection results fall in category C-3, as defined in Table 4.4-2. This change clarifies inspection frequencies and is consistent with the Unit 2 Technical Specifications and the Westinghouse Standard Technical Specifications. The staff finds it acceptable.

Section 4.4.5.4 establishes acceptance criteria for the steam generator inspection program for Unit 1. This change adds an acceptance criterion defining "Preservice Inspection", which parallels the Salem Unit 2 criterion. The requirement that any tube that does not permit the passage of the eddy current probe be deemed a defective tube has been deleted from the acceptance criteria for a defect in the Unit 1 Technical Specifications. The required action to be taken if an eddy current probe will not pass through the tube has been added as Surveillance Requirement 4.4.5.2.b.3 as described above. By letter dated December 28, 1990, PSE&G stated that the plugging limit for a dented tube is failure to pass a 0.540 inch diameter probe; tubes restricting a 0.720 inch diameter probe but passing a 0.540 inch diameter probe are recorded and tracked for any change in restriction. The acceptance criteria for "defect" and "plugging limit" were clarified to be consistent with the Unit 2 Technical Specifications and the Westinghouse Standard Technical Specifications. The staff finds these changes acceptable.

An editorial change was made to Section 4.4.5.5.b. in the Unit 1 Technical Specifications. The word "this" was changed to "the". The staff finds this change acceptable.

The changes to the Salem 1 Technical Specifications provide for consistency between Unit 1 and Unit 2 Technical Specifications. This reduces the potential for error by eliminating the possibility of using the wrong surveillance procedure because the steam generator surveillance requirements are now identical. For reasons stated above, the staff finds the changes to the Salem 1 Technical Specifications, Section 4.4.5.2 through 4.4.5.5, to be acceptable.

A change to the Salem Unit 2 Technical Specifications, Section 4.4.6.5.b clarifies that the results of the steam generator tube inspection will be included in the Annual Operating Report. The staff finds this change acceptable.

The staff, with the concurrence of PSE&G made the following editorial changes:

1. Unit 1, Section 4.4.5.2, in reference to Category C-3 inspection results, the spelling of the word "or" was corrected. That section now reads ". . . degraded tubes or more than . . .".

2. Unit 1, Section 4.4.5.2.b.3, the word "for" was changed to "to" in the last line. That section now reads ". . . and subjected to a tube inspection."

3. Unit 1, Section 4.4.5.5, the word "included" was inserted between "be" and "in" in the first sentence. That section now reads ". . . inspection shall be included in the Annual . . .".

The above changes brought the corrected technical specification pages, that were submitted with the amendment request, into agreement with the marked up pages that were also submitted with the amendment request. In order to preserve the page number sequence in the Unit 1 Technical Specifications, the staff, with the concurrence of PSE&G, inserted a blank page 3/4 4-11.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (55 FR 2444) on January 24, 1990 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Dated: February 13, 1991

Principal Contributor:

M. Franovich, J. Stone