#### Docket Nos. 50-272/311

Mr. Steven E. Miltenberger Vice President and Chief Nuclear Public Service Electric & Gas Company Post Office Box 236 Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: RESIDUAL HEAT REMOVAL SYSTEM OPEN PERMISSIVE TEST PRESSURE

(TAC NOS. 75121/75122)

SALEM GENERATING STATION, UNIT NOS. 1 AND 2 RE:

The Commission has issued the enclosed Amendment Nos.107 and 84 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated September 28. 1989.

These amendments reduce the Residual Heat Removal (RHR) system open permissive interlock test pressure from 580 psig or greater to 375 psig or greater.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely.

**/S/** 

James C. Stone, Project Manager Project Directorate I-2 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

#### Enclosures:

- Amendment No. 107to License No. DPR-70
- Amendment No. 84to License No. DPR-75
- Safety Evaluation

cc w/enclosures: See next page

DISTRIBUTION w/enclosures:

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Rita Jaques, ARM/LFMB

Wanda Jones DHagan GPA/PA **RJones** 

**JDyer BBoger JStone**  NRC PDR/LPDR PDI-2 Reading

OGC **EJordan** GHill (8)

[75121/22 LTR]

PDI-2/PM ₩3Stone:tr 12/19/89

Docket Nos. 50-272/311

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RE: SALEM GENERATING STATION, UNIT NOS. 1 AND 2

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/S/

James C. Stone, Project Manager Project Directorate I-2 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

### Enclosures:

 Amendment No. 107to License No. DPR-70

Amendment No. 84to License No. DPR-75

3. Safety Evaluation

cc w/enclosures:
See next page

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[75121/22 LTR]

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**RJones** 

JDyer BBoger JStone NRC PDR/LPDR PDI-2 Reading MThadani

OGC EJordan GHill (8)

PDI-2/D WButler

1/17/89

SRAB RJones 1 / 1/89



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

January 17, 1990

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger Vice President and Chief Nuclear Officer Public Service Electric & Gas Company Post Office Box 236 Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: RESIDUAL HEAT REMOVAL SYSTEM OPEN PERMISSIVE TEST PRESSURE

(TAC NOS. 75121/75122)

RE: SALEM GENERATING STATION, UNIT NOS. 1 AND 2

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These amendments reduce the Residual Heat Removal (RHR) system open permissive interlock test pressure from 580 psig or greater to 375 psig or greater.

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Sincerely.

James C. Stone, Project Manager

Project Directorate I-2

Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

Enclosures:

 Amendment No.107 to License No. DPR-70

2. Amendment No. 84 to License No. DPR-75

3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

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Kathy Halvey Gibson, Resident Inspector Salem Nuclear Generating Station U.S. Nuclear Regulatory Commission Drawer I Hancocks Bridge, NJ 08038

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Lower Alloways Creek Township c/o Mary O. Henderson, Clerk Municipal Building, P.O. Box 157 Hancocks Bridge, NJ 08038

Mr. Bruce A. Preston, Manager Licensing and Regulation Nuclear Department P.O. Box 236 Hancocks Bridge, NJ 08038

Mr. David Wersan Assistant Consumer Advocate Office of Consumer Advocate 1425 Strawberry Square Harrisburg, PA 17120

Mr. Scott B. Ungerer MGR. - Joint Generation Projects Atlantic Electric Company P.O. Box 1500 1199 Black Horse Pike Pleasantville, NJ 08232

Mr. Jack Urban General Manager, Fuels Department Delmarva Power & Light Company 800 King Street Wilmington, DE 19899

Public Service Commission of Maryland Engineering Division ATTN: Chief Engineer 231 E. Baltimore Street Baltimore, MD 21202-3486



# NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM GENERATING STATION, UNIT NO. 1

# AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107 License No. DPR-70

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 28, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I:
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

9001250271 900117 PDR ADOCK 05000272 P PDC (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 107, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director Project Directorate I-2

Division of Reactor Projects I/II

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Attachment: Changes to the Technical Specifications

Date of Issuance: January 17, 1990

# FACILITY OPERATING LICENSE NO. DPR-70 DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Page 3/4 5-5b

Insert Page

3/4 5-5b

#### SURVEILLANCE REQUIREMENTS (Continued)

i. The automatic interlock function of the RHR System shall be verified within the seven (7) days prior to placing the RHR System in service for cooling of the Reactor Coolant System. This shall be done by verifying with a test signal corresponding to a reactor coolant pressure of 375 psig or greater, that the 1RH1 and 1RH2 valves cannot be opened.



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

## PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM GENERATING STATION, UNIT NO. 2

## AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84 License No. DPR-75

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated September 28, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

# (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance. Implementation shall be before startup from the fifth refueling outage currently scheduled to begin in March 1990.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Director Project Directorate I-2

Division of Reactor Projects I/II

Attachment: Changes to the Technical Specifications

Date of Issuance: January 17, 1990

# ATTACHMENT TO LICENSE AMENDMENT NO. 84

# FACILITY OPERATING LICENSE NO. DPR-75

# DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Page

3/4 5-6a

Insert Page

3/4 5-6a

#### SURVEILLANCE REQUIREMENTS (Continued)

i. The automatic interlock function of the RHR System shall be verified within the seven (7) days prior to placing the RHR System in service for cooling of the Reactor Coolant System. This shall be done by verifying with a test signal corresponding to a reactor coolant pressure of 375 psig or greater, that the 2RH1 and 2RH2 valves cannot be opened.

Flow balance testing pursuant to Specification 4.5.2.h shall be performed the first time the unit is in COLD SHUTDOWN after December 15, 1981.

<sup>\*(</sup>Footnote from page 3/4 5-6)



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NOS. 107 AND 84 TO FACILITY OPERATING

LICENSE NCS. DPR-70 AND DPR-75

PUBLIC SERVICE FLECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

### 1.0 INTRODUCTION

By letter dated September 28, 1989, Public Service Electric & Gas Company requested an amendment to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. The proposed amendments would change Surveillance Requirement 4.5.2.i for Salem 1 and 2 to require testing the Residual Heat Removal (RHR) system open permissive interlock at a pressure of 375 psig or greater. The current requirement is to test the open permissive interlock at 580 psig or greater.

### 2.0 EVALUATION

Current Technical Specifications (TS) require the RHR interlocks to be tested, within 7 days of use, by simulating a reactor coolant system (RCS) pressure of 580 psig or greater and verifying that RHR isolation valves, PHR1 and RHR2, cannot be opened. The 580 psig was based on the setpoint of the automatic closure interlock (ACI) which has been removed from Salem Unit 1 and will be removed from Salem Unit 2 during the upcoming refueling outage.

The open permissive interlock (OPI) prevents opening valves RHR1 and RHR2 until the RCS pressure is reduced below the RHR system design pressure (600 psig). The OPI setpoint is 375 psig. This change will allow testing of the OPI at a pressure that corresponds more closely to the setpoint.

The staff concludes that the proposed change is acceptable.

Also, the staff, with the knowledge and concurrence of the licensee, made some administrative changes (i.e. page number and editorial) to the revised technical specification pages.

9001250274 900117 PDR ADOCK 05000272 P PDC

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# 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (54 FR 51262) on December 13, 1989 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Jim Stone

Dated: January 17, 1990