

March 25, 1991

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric & Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: EDUCTOR FLOW TESTING CLARIFICATION, SALEM NUCLEAR GENERATING
STATION, UNITS 1 AND 2 (TAC NOS. 79503/79504)

The Commission has issued the enclosed Amendment Nos.122 and 102 to Facility
Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating
Station, Units 1 and 2. These amendments consist of changes to the Technical
Specifications (TSs) in response to your application dated January 18, 1991.

These amendments clarify the testing methodology associated with surveillance
requirements for the spray additive system eductors and relocate the
surveillance requirements within the TSs.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

You are requested to notify the NRC, in writing, when these amendments have
been fully implemented at Salem 1 and 2.

Sincerely,

/s/

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 122 to
License No. DPR-70
- 2. Amendment No. 102 to
License No. DPR-75
- 3. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION w/enclosures:

Docket File	WButler	GHill(8)	OC/LFMB
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OFC	:PDI-2/PA	:PDI-2/PM	:OGC	:PDI-2/D	:*EMCB
NAME	:MO'Brien	:JStone:rb		:WButler	:CYCheng
DATE	:3/18/91	:3/18/91	:3/18/91	:3/18/91	:03/06/91

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 25, 1991

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Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
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SUBJECT: EDUCTOR FLOW TESTING CLARIFICATION, SALEM NUCLEAR GENERATING
STATION, UNITS 1 AND 2 (TAC NOS. 79503/79504)

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These amendments clarify the testing methodology associated with surveillance requirements for the spray additive system eductors and relocate the surveillance requirements within the TSs.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

You are requested to notify the NRC, in writing, when these amendments have been fully implemented at Salem 1 and 2.

Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.122 to
License No. DPR-70
2. Amendment No.102 to
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Steven E. Miltenberger
Public Service Electric & Gas Company Salem Nuclear Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated January 18, 1991 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 122, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1991

OFC	:PDI-2/A	:PDI-2/PM	:OGC	:PDI-2/D
NAME	:MO'Brien	:JStone:rb	:OPW	:WButler
DATE	:3/25/91	:2/26/91	:3/18/91	:3/18/91

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Document Name: [TAC NOS 79503/4]

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 122, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 122

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 6-9

3/4 6-10

- - - -

Insert Pages

3/4 6-9

3/4 6-10

3/4 6-10a

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.1 Two independent containment spray systems shall be OPERABLE with each spray system capable of taking suction from the RWST and transferring suction to the RHR pump discharge.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one containment spray system inoperable, restore the inoperable spray system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable spray system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.1 Each containment spray system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 18 months during shutdown, by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
 2. Verifying that each spray pump starts automatically on a Containment High-High pressure test signal.
- c. At least once per 5 years by:
 1. Performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.2 The spray additive system shall be OPERABLE with:

- a. A spray additive tank containing a volume of between 2568 and 4000 gallons of between 30 and 32 percent by weight NaOH solution, and
- b. Two spray additive eductors each capable of adding NaOH solution from the chemical additive tank to a containment spray system pump flow.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the spray additive system inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the spray additive system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.2 The spray additive system shall be demonstrated OPERABLE:

- a. At least once per 31 days also by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. At least once per 6 months by:
 1. Verifying the solution level in the tank, and
 2. Verifying the concentration of the NaOH solution by chemical analysis.
- c. At least once per 18 months during shutdown, by verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
- d. At least once per 5 years by:
 1. Verifying a NaOH solution flow rate of 12 ± 3 gpm from the spray additive tank through sample valve 1CS61 with the spray additive tank at 2.5 ± 0.5 psig and

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

2. Verifying that the spray additive tank eductor flow will be 35 ± 3.5 gpm to each containment spray system. Testing may be performed by measuring the flow of borated water from the RWST through the installed 2" test line and Valve CS31; using this test line up with the spray pump operating in the recirculation mode and the RWST level at 41 feet \pm 0.5 feet, the measured flow shall be 57 gpm \pm 5.7 gpm.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated January 18, 1991 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1991

OFC	:PDI-2/A	:PDI-2/PM	:OGC	:PDI-2/D
NAME	:M. Brien	:JStone:rb	:WButler	
DATE	:3/17/91	:2/26/91	:3/18/91	

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Document Name: TAC NOS 79503/4

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

3/4 6-10

3/4 6-11

- - - -

Insert Pages

3/4 6-10

3/4 6-11

3/4 6-11a

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.1 Two independent containment spray systems shall be OPERABLE with each spray system capable of taking suction from the RWST and transferring suction to the RHR pump discharge.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one containment spray system inoperable, restore the inoperable spray system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable spray system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.1 Each containment spray system shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- b. By verifying, that on recirculation flow, each pump develops a discharge pressure of greater than or equal to 215 psig when tested pursuant to Specification 4.0.5.
- c. At least once per 18 months during shutdown, by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
 2. Verifying each each spray pump starts automatically on a Containment High-High pressure test signal.
- d. At least once per 5 years by:
 1. Performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

LIMITING CONDITION FOR OPERATION

- 3.6.2.2 The spray additive system shall be OPERABLE with:
- a. A spray additive tank containing a volume of between 2568 and 4000 gallons of between 30 and 32 percent by weight NaOH solution, and
 - b. Two spray additive eductors each capable of adding NaOH solution from the chemical additive tank to a containment spray system pump flow.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the spray additive system inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the spray additive system to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

- 4.6.2.2 The spray additive system shall be demonstrated OPERABLE:
- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
 - b. At least once per 6 months by:
 1. Verifying the solution level in the tank, and
 2. Verifying the concentration of the NaOH solution by chemical analysis.
 - c. At least once per 18 months during shutdown, by verifying that each automatic valve in the flow path actuates to its correct position on a Containment High-High pressure test signal.
 - d. At least once per 5 years by:
 1. Verifying a NaOH solution flow rate of 12.0 ± 3.0 gpm from the spray additive tank through sample valve 2CS61 with the spray additive tank at 2.5 ± 0.5 psig and

CONTAINMENT SYSTEMS

SPRAY ADDITIVE SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

2. Verifying that the spray additive tank eductor flow will be 35 ± 3.5 gpm to each containment spray system. Testing may be performed by measuring the flow of borated water from the RWST through the installed 2" test line and Valve CS31; using this test line up with the spray pump operating in the recirculation mode and the RWST level at 41 feet \pm 0.5 feet, the measured flow shall be 57 gpm \pm 5.7 gpm.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 122 AND 102 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNITS 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated January 18, 1991, Public Service Electric and Gas Company requested an amendment to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Units 1 and 2. The proposed amendments would clarify existing technical specifications (TS) surveillance requirements 4.6.2.1.c.2 (Salem Unit 1) and 4.6.2.1.d.2 (Salem Unit 2) for the containment spray system. The proposed changes would clearly allow the use of the test line between the refueling water storage tank and the sodium hydroxide (NaOH) eductor to conduct the flow test and would relocate these surveillance requirements from Technical Specification Section 3.6.2.1 to Section 3.6.2.2.

2.0 EVALUATION

Surveillance Requirements 4.6.2.1.c.2 (Unit 1) and 4.6.2.1.d.2 (Unit 2) require that every 5 years the spray additive tank eductor flow rate be verified to be 35 ± 3.5 gpm with the spray pumps operating in the recirculation mode.

There are two different testing methods which may be used to verify the specified eductor flow rate. The first method involves measuring the flow rate to the eductor while taking suction from the spray additive tank (SAT). This method provides the most direct means of verifying the flow rate but requires that sodium hydroxide (NaOH) be injected into the system. Injection of NaOH into the system is an extremely undesirable action in that it would foul the system and require extensive clean up following testing. Additionally, injecting NaOH into the system could result in spraying the containment with NaOH if an equipment malfunction or operator error occurred.

The second method uses a test line from the refueling water storage tank (RWST) which ties into the eductor line downstream of the SAT isolation valves. This test line allows the flow test to be performed using RWST water. The SAT remains isolated from the system and NaOH injection is precluded. Because there are elevation differences between the SAT and the RWST and density differences between the borated water in the RWST and NaOH in the SAT, the indicated flow rate during testing with the flow from the RWST (RWST level at 41 ± 0.5 feet) must be $57 \text{ gpm} \pm 5.7 \text{ gpm}$ to ensure that the flow from the SAT would be $35 \text{ gpm} \pm 3.5 \text{ gpm}$. This correlation is based on a Westinghouse analysis that was verified during testing December 1980. All parameters that could affect the results of the correlation are the same for both of the Salem units.

Initial flow rate verification was carried out during startup using the first test method with demineralized water in the SAT. Subsequent tests have been carried out using the second test method (i.e., the test line from the RWST).

In order to clarify the acceptability of the use of the test line from the RWST, the existing surveillance requirement would be replaced with the following:

"Verifying that the spray additive tank eductor flow will be $35 + 3.5 \text{ gpm}$ to each containment spray system. Testing may be performed by measuring the flow of borated water from the RWST through the installed 2" test line and Valve CS31; using this test line up with the spray pump operating in the recirculation mode and the RWST level at $41 \text{ feet} \pm 0.5 \text{ feet}$, the measured flow shall be $57 \text{ gpm} \pm 5.7 \text{ gpm}$."

Although the use of the RWST test line does not directly measure the flow from the SAT to the eductor, the test configuration has been correlated to the actual configuration. The validity of the correlation has been verified through testing. Also, the use of the RWST test line precludes the inadvertent spraying of the containment with NaOH during the conduct of the test. The staff finds the proposal to allow the use of the RWST test line during the testing of the SAT eductor to be acceptable.

As specified in Section 6.2.2.1 of the Updated Final Safety Analysis Report, the containment spray system functions to provide the following:

1. Capability to spray cool water into the containment atmosphere in the event of a LOCA thereby ensuring that containment pressure is maintained below its design limit.
2. Capability to remove elemental iodine from the containment atmosphere should it be released during a LOCA.

The TSs contain two separate Limiting Conditions for Operation (LCO) intended to ensure that these capabilities are maintained. LCO 3.6.2.1

is intended to address the containment cooling function of the containment spray system while LCO 3.6.2.2 is intended to address the spray additive function of the system. In order to verify that proper flow exists in the line between the SAT and the point at which the test line from the RWST connects to the eductor supply, a second test is performed. This second test is included under Surveillance Requirement 4.6.2.2.d.

The proposed change would relocate Surveillance Requirements 4.6.2.1.c.2 for Salem 1 and 4.6.2.1.d.2 for Salem 2 from LCO 3.6.2.1. to 3.6.2.2. as an addition to Surveillance Requirement 4.6.2.2.d. This will consolidate the spray additive eductor testing under a single LCO. The Action Statements associated with LCOs 3.6.2.1 and 3.6.2.2 are identical and as a result, actions required because of failure to meet the flow test requirements remain the same. Based on the above, the staff finds the relocation of the Surveillance Requirements from 3.6.2.1 to 3.5.2.2 to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (56 FR 6881) on February 20, 1991 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 25, 1991

Principal Contributor: James Stone