Februar 5, 1990

Docket Nos. 50-272
and 50-311

Mr. Steven E. Miltenberger
Vice President and Chief Nuclear
Officer
Public Service Electric and Gas
Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Posted
Correction to
Amdt. 108 to DPR-70

Dear Mr. Miltenberger:

SUBJECT: PRESSURE-TEMPERATURE LIMITS CORRECTION LETTER
(TAC NOS. 71774 AND 71775)

RE: SALEM GENERATING STATION, UNIT NOS. 1 AND 2

By letter dated January 29, 1990, we issued Amendment Nos. 108 and 86 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. Through an oversight, the Bases for the Reactor Head Vents (Section B 3/4.4.12), which had been previously approved in Amendments 101 and 78 were left out of the revised Bases sections included with Amendments 108 and 86. No changes were made to section B 3/4.4.12 by Amendments 108 and 86. However, because of extensive revisions to Section B 3/4.4.10, Pressure/Temperature Limits, the page numbers changed. The enclosed pages are to be used to replace the pages sent in Amendments 108 and 86 as follows:

Salem Unit 1, Amendment 108 - Replace page B 3/4 4-17 with enclosed page B 3/4 4-17

Salem Unit 2, Amendment 86 - Replace page B 3/4 4-18 with enclosed page B 3/4 4-18.

Sincerely,

/s/
James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosure:
Technical Specification
Replacement pages

cc w/enclosure: See next page

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[71774/5]

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 15, 1990

Docket Nos. 50-272
and 50-311

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Vice President and Chief Nuclear
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Public Service Electric and Gas
Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

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Sincerely,

James C. Stone

James C. Stone, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosure:
Technical Specification
Replacement pages

cc w/enclosure: See next page

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Engineering Division
ATTN: Chief Engineer
231 E. Baltimore Street
Baltimore, MD 21202-3486

3/4.4.10 STRUCTURAL INTEGRITY

The inspection programs for ASME Code Class 1, 2 and 3 components ensure that the structural integrity of these components will be maintained at an acceptable level throughout the life of the plant. To the extent applicable, the inspection program for these components is in compliance with Section XI of the ASME Boiler and Pressure Vessel Code.

3/4.4.11 STRUCTURAL INTEGRITY

The inservice inspection and testing programs for ASME Code Class 1, 2 and 3 components ensure that the structural integrity and operational readiness of these components will be maintained at an acceptable level through the life of the plant. These programs are in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50.55a(g) except where specific written relief has been granted by the Commission pursuant to 10 CFR Part 50.55a(g)(6)(i).