



Kewaunee Nuclear Power Plant  
N490, State Highway 42  
Kewaunee, WI 54216-9511  
920-388-2560

Operated by  
Nuclear Management Company, LLC



June 13, 2001

10 CFR 50, App. E

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D.C. 20555

Ladies/Gentlemen:

DOCKET 50-305  
OPERATING LICENSE DPR-43  
KEWAUNEE NUCLEAR POWER PLANT  
RADIOLOGICAL EMERGENCY RESPONSE PLAN IMPLEMENTING PROCEDURES

Pursuant to 10 CFR 50 Appendix E, attached is the latest revisions to the Kewaunee Nuclear Power Plant Radiological Emergency Response Plan Implementing Procedures (EPIPs). These revised procedures supersede the previously submitted procedures.

Pursuant to 10 CFR 50.4, two additional copies of this letter and attachment are hereby submitted to the Regional Administrator, U. S. Nuclear Regulatory Commission, Region III, Lisle, Illinois. As required, one copy of this letter and attachment is also submitted to the Kewaunee Nuclear Power Plant NRC Senior Resident Inspector.

Sincerely,

Thomas J. Webb  
Site Licensing Director

SLC

Attachment

cc - US NRC Senior Resident Inspector, w/attach.  
US NRC, Region III (2 copies), w/attach.  
Electric Division, PSCW, w/o attach.  
QA Vault, w/attach.

Ho 45

**KEWAUNEE NUCLEAR POWER PLANT**

June 12, 2001

**EMERGENCY PLAN IMPLEMENTING PROCEDURES TRANSMITTAL FORM**

**RETURN TO DIANE FENCL - KNPP**

**OUTSIDE AGENCY COPIES (1-20)**

T. Webb - NRC Document Control Desk (1)*	Bob Hayden - Wisconsin Electric Power Co. (10)*
T. Webb - NRC Region III (2 & 3)*	Craig Weiss - Wisconsin Power & Light (11)*
T. Webb - NRC Resident Inspector (4) (receives Appx. A phone numbers)*	
T. Webb - State of Wisconsin (5)*	Jim Holthaus - Nuclear Management Company (12)*
T. Webb - KNPP QA Vault w/NRC Letter (15)*	

**PERSONAL COPIES (21-40)** These copies are for the personal use of the listed individuals for reference or emergency response.

J. Bennett (33)	D. Mielke (35)	H. Kocourek (13)	K. Hoops (28)
D. Masarik (32)	D. Seebart (24)	B. Bartelme (34)	

**REFERENCE COPIES - CUSTODIAN (41-100)** These copies are for general reference by anyone. They are distributed throughout the plant and corporate offices. The named individual is the responsible custodian for the procedures and shall insure they are properly maintained.

STF (86, 87, 88)	LOREB - STF (62, 66, 67, 68, 70, 72, 73, 74)
L. Duggan - Fuel Services (65)	STF Library (43)
NO Library - KNPP (59)	Resource Center (82, 89, 94, 131)
C. Sternitzky - ATF-2 (44)	D. Schrank - Maintenance Off. (41)
D. Braun - ATF-3 (45)	M. Anderson - CR/SS Office (51, 56)
P. Ehlen - I&C Office (42)	P&FS Adm - GB-D2 (Nuclear Library) (84)
M. Daron - Security Building (46)	H. Kocourek - TSC (50)
P&FS Adm - GB D2-3 (EOF) (81)	W. Galarneau - RAF (53)
H. Kocourek - OSF (52)	W. Galarneau - SBF/EMT (54)
C. Hutter - ATF-1 (64)	T. Schmidli - RPO (55)

**WORKING COPIES (101-199)** These copies of procedures are kept in the areas designated for use in response to an emergency. These are not complete sets, but contain only those procedures that are used to implement activities in the location where they are kept. Please dispose of any sections distributed that are not tabbed in the indicated copy.

W. Galarneau - RAF/RPO (106, 107)	M. Anderson - CR/Communicator (116)(Partial Distribution)
W. Galarneau - SBF/ENV (108, 109)	Simulator/Communicator (117)
W. Galarneau - SBF/EM Team (110, 111, 111A)	J. Fletcher - Security (121)
W. Galarneau - Aurora Medical Center (118, 119)	N. Deda - Security Building (120)
W. Flint - Cold Chem/HR Sample Room (113)	S. VanderBloomen (125)
N. Deda - SBF/SEC (114)	J. Stoeger (126)

**Originals to KNPP QA Vault**

Please follow the directions when updating your EPIP Manual. **WATCH FOR DELETIONS!!!** These are controlled procedures and random checks may be made to ensure the manuals are kept up-to-date.

**\*THIS IS NOT A CONTROLLED COPY. IT IS A COPY FOR INFORMATION ONLY.**

**KEWAUNEE NUCLEAR POWER PLANT**  
**REVISION OF EMERGENCY PLAN IMPLEMENTING PROCEDURES**  
June 12, 2001

Please follow the directions listed below. If you have any questions regarding changes made to the EIPs, please contact Dave Seebart at ext. 8719. If you are a controlled copy holder (see cover page), return this page to Diane Fencel by July 12, 2001, SIGNED AND DATED to serve as a record of revision.

EPIP Index, dated 06-12-2001.

REMOVE		INSERT	
PROCEDURE	REV.	PROCEDURE	REV.
EP-RET-2D	L	EPIP-RET-02D	M
EP-RET-4	P	EPIP-RET-04	Q
EP-RET-4A	C	EPIP-RET-04A	D
EP-TSC-3	T	EPIP-TSC-03	U
<del>Figure EPIPG APPX A-06-03</del> <del>(filed after EPIP SEC 05)</del>	<del>A</del>	<del>Figure EPIPEC APPX A-06-03</del> <del>(filed after EPIP SEC 05)</del>	<del>B</del>
<del>Figure EPIPG APPX A-06-03 ONLY</del> <del>(filed after EPIP APPX A-06)</del>	<del>A</del>	<del>Figure EPIPEC APPX A-06-03 ONLY</del> <del>(filed after EPIP APPX A-06)</del>	<del>B</del>
EPIP FORM RET 4	D	Form EPIPF-RET-04-01	E
EPIP FORM TSC 3.1	K	Form EPIPF-TSC-03-01	L
EPIP FORM TSC 3.2	K	Form EPIPF-TSC-03-02	L
EPIP FORM TSC 3.3	I	Form EPIPF-TSC-03-03	J
EPIP FORM TSC 3.4	G	Form EPIPF-TSC-03-04	H

I CERTIFY Copy No. \_\_\_\_\_ (WPSC No.) of the  
Kewaunee Nuclear Power Plant's EIPs has been  
updated.

\_\_\_\_\_  
SIGNATURE

\_\_\_\_\_  
DATE

Please return this sheet to *DIANE FENCL*.

  
Diane Fencel

Enclosure

**EMERGENCY PLAN IMPLEMENTING PROCEDURES****INDEX****DATE: 06-12-2001**

<b>PROC. NO.</b>	<b>TITLE</b>	<b>REV.</b>	<b>DATE</b>
<b>EP-AD</b>			
EPIP-AD-01	Personnel Response to the Plant Emergency Siren	G	06-05-2001
EPIP-AD-02	Emergency Class Determination	Z	03-07-2000
EPIP-AD-03	KNPP Response to an Unusual Event	AB	03-20-2001
EPIP-AD-04	KNPP Response to Alert or Higher	AC	03-20-2001
EP-AD-5	Site Emergency	Deleted	04-27-87
EPIP-AD-05	Emergency Response Organization Shift Relief Guideline	C	06-05-2001
EP-AD-6	General Emergency	Deleted	04-24-87
EPIP-AD-07	Initial Emergency Notifications	AM	01-30-2001
EP-AD-8	Notification of Alert or Higher	Deleted	02-26-96
EP-AD-9	Notification of Site Emergency	Deleted	04-27-87
EP-AD-10	Notification of General Emergency	Deleted	04-27-87
EP-AD-11	Emergency Radiation Controls	P	08-10-99
EP-AD-12	Personnel Assembly and Accountability	Deleted	03-26-94
EP-AD-13	Personnel Evacuation	Deleted	04-25-94
EP-AD-13A	Limited Area Evacuation	Deleted	03-01-83
EP-AD-13B	Emergency Assembly/Evacuation	Deleted	03-01-83
EP-AD-13C	Site Evacuation	Deleted	03-01-83
EP-AD-14	Search and Rescue	Deleted	05-25-94
EPIP-AD-15	Recovery Planning and Termination	N	08-29-2000
EP-AD-16	Occupational Injuries or Vehicle Accidents During Emergencies	Deleted	03-14-97
EP-AD-17	Communications	Deleted	03-05-84
EPIP-AD-18	Potassium Iodide Distribution	O	06-05-2001
EPIP-AD-19	Protective Action Guidelines	P	10-31-2000
<b>EP-ENV</b>			
EPIP-ENV-01	Environmental Monitoring Group Organization and Responsibilities	U	10-31-2000
EPIP-ENV-02	Environmental Monitoring Team Activation	W	06-15-2000

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<b>PROC. NO.</b>	<b>TITLE</b>	<b>REV.</b>	<b>DATE</b>
EP-ENV-3A	Environmental Protection Director Actions and Directives	Deleted	09-26-84
EP-ENV-3B	EM Team Actions	Deleted	09-26-84
EPIP-ENV-03C	Dose Projection Using RASCAL Version 2.2 Software	U	02-16-2000
EP-ENV-3D	Revision and Control of ISODOSE II	Deleted	02-14-95
EP-ENV-3E	Manual Determination of X/Q	Deleted	04-24-87
EP-ENV-3F	Manual Determination of X/Q (Green Bay Meteorological Data)	Deleted	05-30-86
EP-ENV-3G	Manual Dose Projection Calculation	Deleted	06-02-89
EP-ENV-3H	Protective Action Recommendations	Deleted	04-13-90
EPIP-ENV-04A	Portable Survey Instrument Use	S	06-15-2000
EPIP-ENV-04B	Air Sampling and Analysis	V	09-12-2000
EP-ENV-4C	Environmental Monitoring Teams	Deleted	04-13-90
EPIP-ENV-04C	Ground Deposition Sampling and Analysis	V	09-12-2000
EPIP-ENV-04D	Plume Tracking for Environmental Monitoring Teams	M	09-12-2000
EP-ENV-5A	LCS-1 Operation	Deleted	04-14-86
EP-ENV-5B	MS-3 Operation	Deleted	04-14-86
EP-ENV-5C	SAM II Operation	Deleted	04-14-86
EP-ENV-5D	PAC-4G (Alpha Counter) Operation	Deleted	04-14-86
EP-ENV-5E	Reuter-Stokes Operation	Deleted	08-27-85
EP-ENV-6	Data Analysis, Dose Projections and Protective Action Recommendations	Deleted	12-21-81
EP-ENV-6	Alternate Sample Analysis and Relocation of EM Team	Deleted	04-14-86
EP-ENV-6A	Relocation of Site Access Facility (Habitability)	Deleted	03-23-84
EP-ENV-6B	SAF Environmental Sample Analysis Relocation	Deleted	03-23-84
EP-ENV-7	Site Access Facility Communications	Deleted	09-26-84
EP-ENV-8	Total Population Dose Estimate Calculations	Deleted	04-14-86

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PROC. NO.	TITLE	REV.	DATE
<b>EP-EOF</b>			
EP-EOF-1	Corporate Emergency Response Organization	Deleted	03-11-94
EPIP-EOF-2	Emergency Operations Facility (EOF) Activation	X	10-24-2000
EPIP-EOF-03	Corporate Action for Unusual Event	Z	06-01-2000
EPIP-EOF-04	Corporate Action for Alert or Higher	AF	06-01-2000
EP-EOF-5	Corporate Staff Action for Site Emergency	Deleted	04-24-87
EP-EOF-6	Corporate Staff Action for General Emergency	Deleted	04-24-87
EP-EOF-7	Notification of Unusual Event	Deleted	04-06-94
EP-EOF-8	Relocation of EOF	Deleted	03-01-83
EPIP-EOF-08	Continuing Emergency Notifications	S	09-26-2000
EP-EOF-9	Interface with Support Organizations	Deleted	03-05-84
EP-EOF-9	Notification of Site Emergency	Deleted	04-24-87
EP-EOF-10	Notification of General Emergency	Deleted	04-24-87
EPIP-EOF-11	Internal Communication and Documentation Flow	T	06-01-2000
EPIP-EOF-12	Media Center/Emergency Operation Facility/Joint Public Information Center Security	O	10-24-2000
<b>EP-OP</b>			
EP-OP-1	Control Room Emergency Organization	Deleted	04-24-87
EP-OP-2	Emergency Control Room Activation for Emergency Response	Deleted	04-24-87
EP-OP-3	Control Room Communications	Deleted	04-24-87
<b>EP-OSF</b>			
EP-OSF-1	Operation Support Facility Emergency Organization	Deleted	04-24-87
EPIP-OSF-02	Operational Support Facility Operations	S	08-29-2000
EPIP-OSF-03	Work Requests During an Emergency	N	09-12-2000
EP-OSF-4	Operational Support Facility Communications	Deleted	04-24-87
EPIP-OSF-04	Search and Rescue	D	09-12-2000

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EPIP-RET-02	In-Plant Radiation Emergency Team	S	06-05-2001
EPIP-RET-02A	RPO - RAF Activation	R	06-05-2001
EP-RET-2B	Gaseous Effluent Sample and Analysis	Q	03-03-98
EP-RET-2C	Containment Air Sampling and Analysis	Deleted	03-01-83
EPIP-RET-02D	Emergency Radiation Entry Controls and Implementation	M	06-12-2001
EP-RET-2E	Handling of Injured Personnel	Deleted	04-16-96
EP-RET-2F	Personnel Decontamination	Deleted	04-13-90
EPIP-RET-03	Chemistry Emergency Team	O	02-01-2000
EP-RET-3A	Liquid Effluent Release Paths	K	01-12-99
EP-RET-3B	Post-Accident Reactor Coolant Alternate Sampling Procedure	Deleted	01-25-88
EP-RET-3C	Post Accident Operation of the High Radiation Sample Room	O	01-18-2000
EP-RET-3D	Containment Air Sampling Analysis Using CASP	M	01-18-2000
EP-RET-3E	Post Accident Operation of High Rad Sample Room Inline Multiported Count Cave	Deleted	08-27-85
EPIP-RET-04	SBF Activation	Q	06-12-2001
EP-RET-4A	EOF Radiological Monitoring	Deleted	03-10-83
EPIP-RET-04A	SBF Operation/Relocation	D	06-12-2001
EP-RET-4B	Radiological Controls at Site Access Facility	Deleted	07-12-94
EP-RET-4C	Site Radiological Monitoring	Deleted	07-12-94
EP-RET-4D	SAM-II Operation	Deleted	07-12-94
EP-RET-5	Plume Projection	Deleted	09-26-84
EPIP-RET-05	Site Boundary Dose Rates During Controlled Plant Cooldown	G	07-18-2000
EP-RET-5A	Plume Projection	Deleted	04-27-87
EP-RET-6	Dose Projection	Deleted	04-24-87

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EPIP-RET-08	Contamination Control of the Aurora Medical Center	O	06-15-2000
EPIP-RET-09	Post-Accident Population Dose	K	08-29-2000
<b>EP-SEC</b>			
EP-SEC-1	Security Organization	Deleted	04-24-87
EPIP-SEC-02	Security Force Response to Emergencies	V	10-31-2000
EP-SEC-2A	Manual Activation of Emergency Sirens	Deleted	04-16-82
EPIP-SEC-03	Personnel Assembly and Accountability	X	03-28-2000
EPIP-SEC-04	Security Force Actions for Dosimetry Issue	O	02-16-2000
EP-SEC-5	Security Force Response to the EOF	Deleted	07-28-88
EPIP-SEC-05	Personnel Evacuation	E	02-16-2000
<b>EP-TSC</b>			
EP-TSC-1	Technical Support Center Organization and Responsibilities	O	04-01-99
EPIP-TSC-02	Technical Support Center Activation	R	07-18-2000
EPIP-TSC-03	Plant Status Procedure	U	06-12-2001
EPIP-TSC-04	Emergency Physical Changes, Major Equipment Repair	L	08-29-2000
EP-TSC-5	Technical Support Center Communications Equipment	Deleted	04-24-87
EP-TSC-6	Assessment of Reactor Core Damage	Deleted	09-30-86
EPIP-TSC-07	RV Head Venting Time Calculation	H	03-07-2000
EPIP-TSC-08A	Calculations for Steam Release from Steam Generators	M	03-07-2000
EPIP-TSC-08B*	STMRLS Computer Program	E	03-07-2000
EP-TSC-8C*	See EP-TSC-8B	Deleted	04-16-92
* EP-TSC-8B was totally deleted; therefore, EP-TSC-8C was changed to EP-TSC-8B			



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EP-TSC-9	Core Damage Assessment Using Released Radionuclides	Deleted	09-30-86
EP-TSC-9A*	Core Damage Assessment	I	02-23-99
EPIP-TSC-09B*	CORE Computer Program	I	03-07-2000
EP-TSC-9C*	See EP-TSC-9B	Deleted	04-16-92
* EP-TSC-9A, Rev. D was totally deleted; therefore, EP-TSC-9B became EP-TSC-9A. EP-TSC-9B was previously EP-TSC-9C.			
EPIP-TSC-10	Technical Support for IPEOPs	I	03-20-2001

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EPIP-APPX-A-06	EP-FIG-005	APPX-A-06-02	Site Boundary Facility - KNP Floor Plan	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-008	APPX-A-06-01	Radiological Analysis Facility - KNP Floor Plan	A	10-31-2000
EPIP-EOF-12 Form EPIPG-EOF-02-01	EP-FIG-009	EOF-12-01	Division Office Building (2nd Floor) Floor Plan	B	10-24-2000
EPIP-APPX-A-06	EP-FIG-012	APPX-A-06-08	State/County Work Area - WPSC D2-1 Floor Plan	C	10-31-2000
EPIP-APPX-A-06	EP-FIG-013	APPX-A-06-09	NRC Work Area - WPSC D2-4 Floor Plan	A	10-31-2000
EPIP-AD-19	EP-FIG-014	AD-19-01	Population Distribution by Geographical Sub-Areas (with sectors)	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-022	APPX-A-06-04	EOF - WPSC D2-3 Floor Plan	B	10-31-2000
EPIP-EOF-12	EP-FIG-024	EOF-12-02	Map - Location of JPIC, MBC, GOB, DOB, etc.	A	10-24-2000
EP-SEC-5	EP-FIG-026		Site Map	A	07-21-98
APPX-A-6	EP-FIG-034	---	Floor Plan - Media Briefing Center	DEL	08-04-98
EPIP-EOF-12 EPIP-APPX-A-06	EP-FIG-035	APPX-A-06-06	General Office Building - WPSC (1st Floor) Floor Plan	C	10-24-2000
APPX-A-6	EP-FIG-037	---	Floor Plan - Corporate Response Center	DEL	08-04-98
APPX-A-6	EP-FIG-038	---	Floor Plan - JPIC	DEL	08-04-98
EP-OSF-2	EP-FIG-039		High Priority Work	ORIG	07-08-98
EP-OSF-2	EP-FIG-039A		Lower Priority Work	ORIG	07-08-98
EPIP-APPX-A-06	EP-FIG-043	APPX-A-06-10	JPIC - Federal Work Area - WPSC D2-9	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-044	APPX-A-06-07	JPIC - State and County Work Area - WPSC D2-8	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-045	APPX-A-06-05	JPIC - Utility Work Area - WPSC D2-7	A	10-31-2000
RET-08	EP-FIG-046	RET-08-01	Aurora Medical Center Location	A	06-15-2000
EPIP-APPX-A-02	---	APPX-A-02-01	ERO Call Tree	A	01-30-2001

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EPIP-APPX-A-02	Response Personnel Call List	BJ	04-03-2001
EPIP-APPX-A-03	Off-Site Telephone Numbers	BK	04-03-2001
EPIP-APPX-A-06	WPSC Emergency Response Telephone Numbers	X	05-16-2001

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<b>APPENDIX B</b>			
<b>EP-AD</b>			
AD 7.1	Event Notice (Wisconsin Nuclear Accident Reporting Form)	P	02-01-2000
AD 7.2	State Call-Back - Question Guideline	B	04-16-96
AD 11.1	Emergency Radiation Work Permit	F	04-16-96
<b>EP-ENV</b>			
ENV-01-01	Environmental Dispatch Area Activation Checklist	D	10-31-2000
ENV-01-02	EMT Status	B	10-31-2000
ENV-01-03	Meteorological and Plant Status Data	B	10-31-2000
ENV-01-04	EMT Orders/Field Data	B	10-31-2000
ENV-02-01	EMT Activation Checklist	M	06-15-2000
<b>EP-EOF</b>			
EOF-02-01	EOF Activation Checklist	R	10-24-2000
EOF-02-02	EOF Deactivation Checklist	K	10-24-2000
EOF 4.1	SRCL Initial Action Checklist	B	09-16-97
EOF 4.2	Telephone Communications Log Sheet	ORIG	04-16-96
EOF 8.3	Fax for Emergency Declaration or Status Updates	F	09-21-99
EOF 8.5	Plant Emergency Status Report	ORIG	02-21-95
EOF 8.6	Radiological Status Report	C	03-14-97
EOF 11.2	Operating Status	E	02-14-95
EOF 11.3	Environmental Status Board	E	07-31-95
EOF-12-01	I.D. Badge Registration Form	G	10-24-2000
<b>EP-OSF</b>			
OSF 2.2	Maintenance Work in Progress	Deleted	07-08-98
OSF-03-01	Operational Support Facility Team Briefing	B	09-12-2000
<b>EP-RET</b>			
RET-02A-02	Emergency Sample Worksheet	E	06-05-2001
RET 2B.1	Containment Stack Release (Grab Sample)	C	04-16-96
RET 2B.2	Auxiliary Building Stack (Grab Sample)	C	04-16-96

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RET 2B.4	Containment Stack (Sping Reading)	B	04-16-96
RET 2B.5	Steam Release	C	04-16-96
RET 2B.6	Field Reading (Grab Sample)	A	04-16-96
RET-04-01	SAM-2 Counting Equipment Worksheet	E	06-12-2001
RET 8.3	Hospital Survey 1	Deleted	06-05-2001
RET 8.4	Hospital Survey 2	Deleted	07-25-97
RET 8.5	Hospital Survey 3	Deleted	07-25-97
RET-08-06	Hospital Survey 4	F	06-15-2000
RET 9	Environmental TLD Record Sheet	C	02-14-95
<b>EP-SEC</b>			
SEC-03.01	Emergency Accountability Log	A	03-28-2000
SEC 4.1	Emergency Dosimeter Log	F	02-16-2000
<b>EP-TSC</b>			
TSC 1.1	Plant Status Summary for SAM Implementation	A	04-01-99
TSC 1.2	Severe Accident Management Summary and Strategy Recommendation	A	04-01-99
TSC 1.3	Severe Accident Management - Status	A	04-01-99
TSC 2.1	TSC and OSF Activation Checklist	N	04-01-99
TSC 2.2	TSC Ventilation Checklist	H	04-01-99
TSC-02-03	Emergency Response Data System (ERDS) Link Initiation Checklist	G	05-04-2001
TSC-02-04	TSC Chart Recorder Operation Checklist	D	01-30-2001
TSC 2.5	TSC and OSF De-activation Checklist	ORIG	04-01-99
TSC-03-01	Plant System Status	L	06-12-2001
TSC-03-02	Plant Equipment Status	L	06-12-2001
TSC-03-03	Environmental Status Board	J	06-12-2001
TSC-03-04	Radiation Monitors	H	06-12-2001
TSC-04-01	Emergency Physical Change Request	F	08-29-2000
TSC-04-02	Emergency Physical Change Safety Review	F	08-29-2000

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TSC-07-01	Head Venting Calculation	F	10-31-2000
TSC 8A.1	Steam Release Data Sheet (Energy Balance)	G	02-14-95
TSC 8A.2	Steam Release Calculation Sheet (Energy Balance)	F	02-14-95
TSC 8A.3	Steam Release Data/Calculation Sheet (Open Valve)	D	02-14-95
TSC 8A.4	Steam Release Data/Calculation Sheet (STMRLS Program)	C	04-16-96
TSC 9A.1	Core Damage Based on Reactor Vessel Level & Fuel Rod Temp.	C	02-14-95
TSC 9A.2	Core Damage Based on Radiation Monitors	C	02-14-95
TSC 9A.3	Cs-134 and Cs-137 PCF Determination	D	04-16-96
TSC 9A.4	Core Damage Based on Activity Ratios	C	02-14-95
TSC 9A.5	Core Damage Assessment (Monitoring Data)	D	04-16-96
TSC 9A.6	Core Damage Summary	C	02-14-95

<b>WISCONSIN PUBLIC SERVICE CORP.</b>  <b>Kewaunee Nuclear Power Plant</b>  <i>Emergency Plan Implementing Procedure</i>		No.	EPIP-RET-02D	Rev.	M
		Title	Emergency Radiation Entry Controls and Implementation		
		Date	JUN 12 2001	Page 1 of 4	
Reviewed By <i>Jeannine M. Ferris</i>		Approved By <i>Walter M. Benich</i>			
Nuclear Safety Related	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	PORC Review Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	SRO Approval Of Temporary Changes Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No

## 1.0 Purpose

- 1.1 This procedure provides instruction for entering areas that have become radioactively contaminated and/or have increased or unknown radiation levels during emergency repair/operation or search and rescue operations.

## 2.0 General Notes

- 2.1 None

## 3.0 Precautions and Limitations

- 3.1 Personnel engaged in either emergency repair/operation or search and rescue operations should keep in mind the concepts of time, distance, and shielding to minimize radiation exposure.
- 3.2 Personnel shall not be allowed to travel into or through an unknown high radiation area unless they are within the sight of a buddy, and are carrying a continuously indicating portable high range dose rate meter.
- 3.3 A Radiation Work Permit (RWP) shall be completed prior to all entries, except when urgent actions are required for a Priority Entry.
- 3.3.1 A Priority Entry means an entry into the plant under unknown conditions to perform a task considered to be vitally important for swiftly mitigating actual or potential radiological consequences. A Priority Entry allows bypassing normal administrative procedures provided that entrants are briefed on known conditions and are continuously accompanied by a Radiation Technologist qualified to provide the same level of protection normally afforded by a Radiation Work Permit. Administrative documents shall be completed postentry.
- 3.4 Any radiation exposure in excess of 10CFR20 dose limits shall be authorized by the Emergency Director (ED) with the concurrence of the Radiological Protection Director (RPD) (See EPIP-AD-11).
- 3.5 Portable radios should be issued to each team prior to entry.
- 3.6 Iodine blocking agents are available for use when radioiodine airborne concentrations and projected exposure durations are such that a possible thyroid dose could occur (See EPIP-AD-18).

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#### 4.0 Initial Conditions

- 4.1 This procedure shall be implemented upon declaration of an **Alert, Site Emergency, General Emergency**, or when directed by the Shift Manager or Emergency Director.

#### 5.0 Procedure

##### 5.1 In-Plant Radiation Emergency Team (IRET)

###### **Note**

*A senior IRET member or RPO/RAF Leadsman shall direct, control, and brief team members for all RCA entries.*

- 5.1.1 If necessary, assign an IRET member to accompany any Entry Teams as directed.
- 5.1.2 To establish radiological conditions in the affected areas, review shielding maps and critical equipment location maps for the determination of an access pathway.
- 5.1.3 If necessary, complete Radiation Work Permit (RWP) prior to entry, except as allowed by EPIP-AD-11.
- 5.1.4 Ensure that proper instrumentation, respiratory protection, clothing, and dosimetry are used in compliance with the RWP (See EPIP-AD-11).
- 5.1.5 Make available all completed radiation surveys of areas traveled by Entry Team personnel.
- 5.1.6 Use self-contained pressure demand respiratory equipment for worker protection during emergency entries into areas of unknown airborne concentration.
- 5.1.7 Review exposure records of all team members.
- 5.1.8 Determine each team member's remaining annual dose.
  - a. Calculate the stay time based on this dose.
  - b. Review these calculations with the Entry Team Coordinator.

###### **Note**

*It is recommended that annual doses to Entry Team members remain below 10CFR20 dose limits. For entries where a team member is likely to exceed 10CFR20 dose limits, ED approval is required with the concurrence of the RPD, per EPIP-AD-11.*

- 5.1.9 Brief Entry Team members on dose limits and conditions set in accordance with "EPA Radiation Dose Guidelines (EPA-400) Table 2-2," in EPIP-AD-11, for their assigned task, noting TEDE <10 rem for protecting valuable property and TEDE <25 rem for Search and Rescue.



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5.1.10 During the team entry:

- a. Monitor the communications from the Entry Team.
- b. Note radiation levels in specific areas.
- c. Monitor dose rates and doses received to the team members.
- d. Maintain a list of any and all important events, their location, and time of occurrence.

5.1.11 Remove from emergency duty those personnel who have exceeded 10CFR20 dose limits for evaluation per EPIP-AD-11. IF personnel has exceeded or is suspected of exceeding 25 rem TEDE dose, THEN this person should be referred for appropriate medical care.

5.1.12 Report significant data to the RPD.

## 6.0 Final Conditions

6.1 Plant Emergency has been Terminated or Recovery actions have begun and the responsible director has suspended the use of EIPs.

## 7.0 References

- 7.1 Kewaunee Nuclear Power Plant Emergency Plan
- 7.2 EPIP-AD-11, Emergency Radiation Controls
- 7.3 EPIP-AD-18, Potassium Iodide Distribution
- 7.4 EPA-400-R-92-001 Manual of Protective Action Guides and Protective Actions for Nuclear Incidents (October 1991)
- 7.5 Code of Federal Regulations, 10CFR Part 20
- 7.6 Health Physics Procedure Manual
- 7.7 COMTRAK 97-125, Repair Personnel

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## 8.0 Records

- 8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

### 8.1.1 QA Records

- WPSC/KNPP Radiation Work Permit
- Emergency Radiation Work Permit, Form EPIPF-AD-11-01

### 8.1.2 Non-QA Records

None

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Reviewed By <i>[Signature]</i>		Approved By <i>[Signature]</i>			
Nuclear Safety Related	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	PORC Review Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	SRO Approval Of Temporary Changes Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No

## 1.0 Purpose

- 1.1 This procedure provides instruction for the steps to be taken to fully activate the Site Boundary Facility (SBF).

## 2.0 General Notes

- 2.1 None

## 3.0 Precautions and Limitations

- 3.1 None

## 4.0 Initial Conditions

- 4.1 The Site Radiation Emergency Team (SRET) will be activated during an **Alert, Site Emergency, General Emergency**, or at the discretion of the Radiological Protection Director (RPD).
- 4.2 The SBF Coordinator shall be responsible for ensuring timely activation of the SBF.
- 4.3 The SRET is responsible for taking the steps necessary to activate the SBF.

## 5.0 Procedure

- 5.1 Report to the SBF when directed by the RPD. Site Boundary Facility activation is required upon declaration of an Alert or higher per EPIP-AD-04.
- 5.2 Source check the portable survey instruments.
- 5.3 Perform background efficiency checks on the counting equipment not including SAM-2.
- 5.4 Turn on and source check the PPM-1 Portal Monitor as follows:
- 5.4.1 Obtain check source indicated on the PPM-1 calibration sticker.
- 5.4.2 With PPM-1 in "NORMAL" mode, hold the check source in the center of the frame. Do not touch the sides, top, or bottom.
- 5.4.3 Cover the photoelectric sensor to start a count.

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5.4.4 WHEN alarm sounds, remove the source. The PPM-1 is operating properly.

5.4.5 IF the PPM-1 fails to respond properly, THEN notify the RPD and use RM-14's for frisking until the PPM-1 is returned to service.

5.5 Ensure all personnel at the SBF have proper dosimetry. SRET and ENV team members arriving at the SBF with their regular plant issued dosimetry shall continue to use their regular dosimetry and should not use the emergency dosimetry located at the SBF.

5.6 Perform radiation and contamination surveys at the SBF to establish base line radiological conditions.

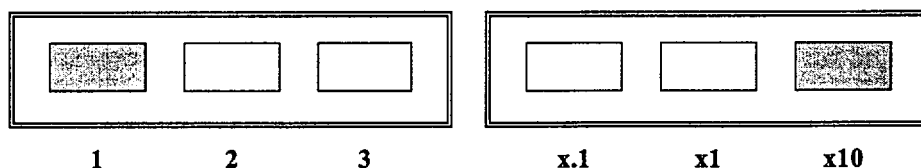
5.7 Notify the RPD that the SBF is activated. Also update the RPD with the status of radiological conditions at the SBF.

5.8 Following SBF activation, set up the SAM-2 stabilized assay meter for use. The SAM-2 is used as a back-up counting system (single channel analyzer) in case the MCA in the plant is not operable or silver zeolite cartridges can not be counted by MCA at the Point Beach Nuclear Plant to determine Iodine-131 concentrations.

5.8.1 The SAM-2 is maintained in a "powered-up" and "ready-to-go" condition at all times with only the display turned off. (IF this is not the case, THEN refer to HP-06.007.)

5.8.2 Turn DISPLAY ON-OFF to ON.

5.8.3 Set time controls for a 10-minute count as follows:



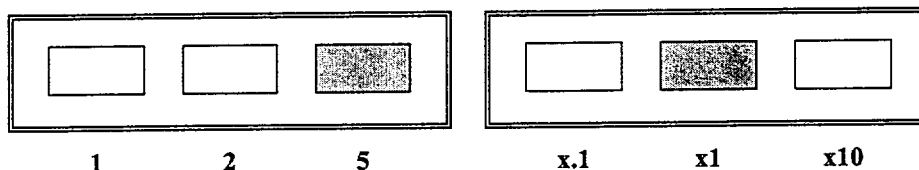
5.8.4 Ensure there is no sample cartridge in the detector shield.

5.8.5 Depress the RESET/START switch to commence a ten-minute background count.

5.8.6 Upon completion of the background count, log background data on Form EPIPF-RET-04-01.

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5.8.7 Reset time controls for a 5-minute count as follows:



5.8.8 Place the Barium-133 source (S/N 0355/RS) under the detector, insure the proper side is facing up.

5.8.9 Depress the RESET/START switch to commence a 5-minute source count.

5.8.10 Upon completion of the source count, log data on Form EPIP-RET-04-01 and determine an efficiency factor as follows:

$$\frac{(source\ dmp) (1.19)}{(source\ cpm) - (bkg\ cpm)} = / -131\ efficiency\ factor$$

**Note**

*The factor of 1.19 is derived in HP-06.007.*

## 6.0 Final Conditions

6.1 Plant Emergency has been Terminated or Recovery actions have begun and the Emergency Response Manager has suspended the use of EIPs.

## 7.0 References

7.1 EPIP-AD-04, KNPP Response to Alert or Higher

7.2 HP-06.007, Instrument Operating Procedure - SAM-2

7.3 EPIP-SEC-04, Security Force Actions for Dosimetry Issue

<b>WISCONSIN PUBLIC SERVICE CORP.</b>  <b>Kewaunee Nuclear Power Plant</b>  <i>Emergency Plan Implementing Procedure</i>	<b>No.</b>	EPIP-RET-04	<b>Rev.</b>	Q
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## 8.0 Records

- 8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

### 8.1.1 QA Records

- SAM-2 Counting Equipment Worksheet, Form EPIPF-RET-04-01

### 8.1.2 Non-QA Records

None

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Reviewed By <i>Jeannette M. Ferris</i>		Approved By <i>W. M. Barth</i>			
Nuclear Safety Related	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	PORC Review Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	SRO Approval Of Temporary Changes Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No

## 1.0 Purpose

- 1.1 This procedure provides instruction for the activities of the Site Radiation Emergency Team (SRET) and the Site Boundary Facility (SBF) Coordinator during operation and relocation of the SBF.

## 2.0 General Notes

- 2.1 None

## 3.0 Precautions and Limitations

- 3.1 None

## 4.0 Initial Conditions

- 4.1 Personnel exposure monitoring at the SBF will be initiated at the levels of **Alert, Site Emergency, and General Emergency**, and as directed by the Emergency Director (ED) or Radiological Protection Director (RPD).
- 4.2 Contamination Control measures will be initiated at the levels of **Site Emergency, General Emergency**, or as directed by the ED or RPD.

## 5.0 Procedure

- 5.1 Following SBF activation, the SBF Coordinator shall take overall charge at the SBF and ensure that the following Radiological Controls are established.

### 5.1.1 Personnel Exposure Monitoring

- 5.1.1.1 Ensure that all personnel have proper dosimetry. SRET and Environmental (ENV) team members arriving at the SBF with their regular plant issued dosimetry shall continue to use their regular dosimetry and should not use the emergency dosimetry located at the SBF.

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5.1.1.2 Instruct all personnel to periodically monitor their pocket dosimeters and to report to the SBF or Radiation Protection Office (RPO) if the dosimeter reaches  $\frac{3}{4}$  of full scale.

5.1.1.3 Any dosimeters reading  $\frac{3}{4}$  scale or greater should be rezeroed and returned to the wearer. Dose totals are to be logged on Form EPIPF-SEC-04-01 and reported to the RPD.

#### 5.1.2 Direct Radiation Surveys

5.1.2.1 Perform direct radiation surveys of the site at intervals as directed by the RPD but not to exceed 4 hours. Initial survey routes are:

- SBF to STH-42
- STH-42 to Security Building
- Sewage Plant to Training Building
- Training Building to Lake along North Fenceline
- Training Building to STH-42
- STH-42 from North Access Road to Nuclear Road

5.1.2.2 Surveys should be made approximately 3 feet above ground level. They may be made from a slowly moving vehicle.

5.1.2.3 Record all radiation survey results on a site map and report all results to the RPD. Survey locations can be modified by the RPD as required.

#### 5.1.3 Ground Level Contamination Surveys

5.1.3.1 Contact the Radiological Analysis Facility (RAF) or RPD for an estimate of plume path.

5.1.3.2 Survey site roads, walkways, and probable traffic routes for contamination as directed by the RPD.

5.1.3.3 Take surface swipes of the area. Return the swipes to the SBF for counting.

5.1.3.4 Report all results to the RPD.

5.1.3.5 IF the contamination is localized, THEN barricade or restrict traffic through the area to prevent spread of contamination.



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5.1.3.6 Per EPIP-AD-11, establish positive control over any areas which exceed:

- 2.0 mrem/hr
- 2,000 DPM/100 cm<sup>2</sup> Beta-Gamma
- 200 DPM/100 cm<sup>2</sup> Alpha

This includes barriers, ropes, signs, protective clothing, etc.

#### 5.1.4 Sample Counting Utilizing the SAM-2

5.1.4.1 Each silver zeolite cartridge, when received at the SBF, should arrive as part of a RAP sample delivered by the Environmental Monitoring Team. The cartridge should be individually bagged along with a RAP particulate filter. The bag containing these two samples should be labeled with the following information:

- Location (by grid coordinates)
- Type of Sample
- Date
- Time Sample Started
- Time Sample Ended
- Flowrate of Sampler from Calibration Sticker or Total Sample Volume
- Initials

5.1.4.2 These samples should be taken immediately to the RAF or the Point Beach Nuclear Plant for counting on the MCA. IF neither MCA is available, THEN count the silver zeolite cartridge at the SBF using the following steps:

5.1.4.3 Remove the silver zeolite cartridge from its bag and place it into the SAM-2 detector shield. Orient the cartridge such that flow arrows are pointing downward.

5.1.4.4 The SAM-2 should already be set for a 5-minute count time. Depress the RESET/START switch to commence sample counting.

5.1.4.5 At the completion of the count, log all data on Form EIPF-RET-04-01 and calculate the Iodine-131 activity.

5.1.4.6 Remove the cartridge from the SAM-2 detector shield and rebag it, ensuring that the original label information is included with the samples.

5.1.4.7 Route the samples (particulate filter, silver zeolite cartridge, and gas marinelli) to the RAF count room for recounting. Call the RAF to have them pick up the samples as soon as they are delivered to the Security Building.

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5.1.4.8 IF no more counting will be required, THEN return Barium-133 source (S/N 0355-RS) to the source locker in the SBF and turn DISPLAY ON-OFF to OFF.

#### 5.1.5 Relocation

5.1.5.1 The SBF shall be relocated if:

- External radiation levels reach 100 mrem/hr and are expected to remain at that level or higher for a minimum of 3 hours, OR
- Airborne radioactivity levels reach 40 times DAC and are expected to remain at that level or higher for a minimum of 3 hours.

5.1.5.2 IF a significant radiological release is occurring, THEN contact the ED or RPD to discuss whether the SBF should be relocated or not.

5.1.5.3 Choose a location that will facilitate performance of SRET activities.

5.1.5.4 Prior to evacuation of the SBF, collect the following:

- Portable Radiation Survey Instruments
- Portable Air Samplers
- TLD Issue Sheets
- Status Board Sheets
- Portable 2-Way Radios
- Radio Antennas
- EPIP Procedures
- Emergency Plan
- Maps
- Potassium Iodide Tablets
- Dosimeters and Chargers
- Phone Lists
- Calculator
- Swipe Packets
- Step-off Pads
- Rad Bags
- Lab Coats
- Gloves
- Shoe Covers
- PPM-1 Portal Monitor

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- 5.1.5.5 Inform the RPD of the position of the relocated SBF and request that the new location be announced on the Gai-tronics.

## 6.0 Final Conditions

- 6.1 Plant Emergency has been Terminated or Recovery actions have begun and the responsible director has suspended the use of EIPs.

## 7.0 References

- 7.1 EPIP-AD-11, Emergency Radiation Controls
- 7.2 HP-03.007, Personnel Decontamination
- 7.3 EPIP-SEC-04, Security Force Actions for Dosimetry Issue
- 7.4 HP-06.007, Instrument Operating Procedure - SAM 2
- 7.5 EPIP-ENV-04B, Air Sampling and Analysis
- 7.6 EPIP Appendix B, Forms

## 8.0 Records

- 8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

### 8.1.1 QA Records

- Emergency Dosimeter Log, Form EPIPF-SEC-04-01
- SAM-2 Counting Equipment Worksheet, Form EPIPF-RET-04-01

### 8.1.2 Non-QA Records

None

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Reviewed By <i>[Signature]</i>		Approved By <i>[Signature]</i>			
Nuclear Safety Related	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	PORC Review Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	SRO Approval Of Temporary Changes Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No

## 1.0 Purpose

- 1.1 This procedure provides instruction for checklists for the Data Coordinator or other Technical Support Center (TSC) staff members to maintain an awareness of plant parameters, equipment availability, and radiological conditions.

## 2.0 General Notes

- 2.1 None

## 3.0 Precautions and Limitations

- 3.1 None

## 4.0 Initial Conditions

- 4.1 This procedure shall be implemented upon declaration of an **Alert, Site Emergency, General Emergency**, or when directed by the Shift Manager or Emergency Director.

## 5.0 Procedure

- 5.1 Start the trend recorders in the TSC (Safety Parameter Display System (SPDS) and meteorological) using instructions on Form EPIPF-TSC-02-04.
- 5.2 Ensure Technical Support Center (TSC) instrumentation and parameter displays are activated and functional.
- 5.2.1 IF a display is not functional, THEN contact the KNPP Computer Group.
- 5.3 Contact the Technical Support Center Director (TSCD) to determine if trends of critical parameters are needed.
- 5.4 If directed, initiate the following trends for the event in progress (i.e., LOCA: RWST Level versus Containment Sump Level):
- 5.4.1 Safety Assessment System
- 5.4.2 Digital Display (#3)
- 5.4.3 Honeywell Trend Recorders

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5.5 Obtain Data from the Plant Process Computer.

5.5.1 Call up *Graphic Display #51*, Plant System Status (Form EPIPF-TSC-03-01) on the system.

5.5.2 Obtain a hard copy by pressing the *Print TSC Screen* function key.

5.5.3 Call up *Graphic Display #52*, Environmental Status Board (Form EPIPF-TSC-03-03) on the Honeywell system.

5.5.4 Obtain a hard copy by pressing the *Print TSC Screen* function key.

5.5.5 Call up *Group Outputs #45* and *#46*, Radiation Monitors on the System, and copy data onto Form EPIPF-TSC-03-04.

5.6 Direct the Operations Communicator to obtain the following information from the Control Room to complete Forms EPIPF-TSC-03-01 and EPIPF-TSC-03-02:

5.6.1 S/G PORV/STM Dump, A/B, Cond/ATM

5.6.2 SI Acc Level 1A/B

5.6.3 Containment Humidity

5.6.4 RXCP Status A/B

5.7 Coordinate with the Operations Communicator to determine plant equipment status and record on Form EPIPF-TSC-03-02.

5.8 Data not available on the SPDS for Environmental Status Board (Form EPIPF-TSC-03-03) should be obtained and filled in by hand using the following guidelines:

a. Meteorological

1. Meteorological strip chart printers in the TSC
2. Point Beach Control Room
3. National Weather Service

b. Radiological Release

1. Radiological Protection Director (RPD)
2. Environmental Protection Director (EPD)

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c. Protective Action Recommendation

1. Emergency Director (ED)
2. Technical Support Center Director (TSCD)

5.9 Update the status boards periodically from the completed forms based on:

- 5.9.1 The magnitude and pace of the event.
- 5.9.2 When a significant change in parameters or equipment status occurs.
- 5.9.3 As a mean guide, update approximately every 15 minutes.

5.10 Review data for trends or significant changes.

5.11 Notify the ED or TSCD of any critical items.

5.12 Return to Step 5.4.

## 6.0 Final Conditions

- 6.1 Plant Emergency has been Terminated or Recovery actions have begun and the Emergency Response Manager has suspended the use of EPIPs.

## 7.0 References

- 7.1 COMTRAK 84-177
- 7.2 COMTRAK 87-156
- 7.3 EPIP Appendix B, Forms

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## 8.0 Records

- 8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

### 8.1.1 QA Records

- TSC Chart Recorder Operation Checklist, Form EPIPF-TSC-02-04
- Plant System Status, Form EPIPF-TSC-03-01
- Plant Equipment Status, Form EPIPF-TSC-03-02
- Environmental Status Board, Form EPIPF-TSC-03-03
- Radiation Monitors, Form EPIPF-TSC-03-04

### 8.1.2 Non-QA Records

None

## SAM-2 COUNTING EQUIPMENT WORKSHEET

### EFFICIENCY FACTOR

$$\frac{(\text{known check source dpm})}{(\text{net check source cpm})} \times 1.19 = \underline{\hspace{2cm}}$$
[illegible]



# PLANT SYSTEM STATUS

PRIMARY SYSTEM CONDITIONS				MITIGATING SYSTEM CONDITIONS			
Time/Date				Time/Date			
C O R E	S/R (N31)		CPS	C O N T A I N M E N T	PRES		psig
	NI		pct/cps		TEMP		°F
	MAX TC		°F		HUMIDITY		pct
	RCS PRES		psig		WR SMP LVL		feet
	PZR LVL		pct		H <sub>2</sub> CONC		pct
	SUBCOOL		°F		HIGH RANGE R-40		r/hr
	HP SI FLW		gpm				
	LP SI FLW		gpm		RWST WR LVL		pct
	RVLIS		pct		SPRAY FLOW		gpm
L O O P S	A	WR T <sub>h</sub>	°F	SG WR LVL		pct	
		WR T <sub>c</sub>	°F	SG PRESSURE		psig	
			DUMP TO		A / C / X		
	B	WR T <sub>h</sub>	°F	SG WR LVL		pct	
		WR T <sub>c</sub>	°F	SG PRESSURE		psig	
			DUMP TO		A / C / X		
NOTES				NOTES			

TRENDS:     ↑ = Increasing     ↓ = Decreasing     ↔ = Stable     ⇕ = Fluctuating

# PLANT EQUIPMENT STATUS

UPDATED (TIME/DATE)	EQUIPMENT STATUS			MISCELLANEOUS STATUS		
	TRAIN					
MAIN AUX (supplies)	a	b		RX TRIP TIME _____		
TERT AUX (supplies)	a	b		EVENT CLASS _____ @ _____		
RES AUX (supplies)	a	b		AUTO SI = NO / YES / RESET / BLOCKED		
DG	A	B	TSC	CORE DAMAGE: CLAD / OVERHEAT / MELT _____ %		
AFW	A	B	TD	UPDATED (TIME)		
SI	A	B		REACTOR COOLANT PUMP	A	B
RHR	A	B		ACCUMULATOR LEVEL	_____ %	_____ %
SWP	A1/A2	B1/B2		BORIC ACID ST. TANK LEVEL	_____ %	_____ %
CFCU	A/B	C/D		ASV	A	B
ICS	A	B		SBV	A	B
CC	A	B				
CHARGING PUMPS	A	B	C			
NOTES						

X = NOT AVAILABLE

O = OPERATING

# ENVIRONMENTAL STATUS BOARD

<b>TIME/DATE:</b>			
<b>WIND DIRECTION</b>	15 min. avg.	10 m _____	deg.
	15 min. avg.	60 m _____	deg.
<b>WIND SPEED</b>	15 min. avg.	10 m _____	mph
	15 min. avg.	60 m _____	mph
<b>DELTA</b>	15 min. avg. _____ °F	<b>Sigma Theta</b>	_____ °
<b>STABILITY CLASS</b>	A B C D E F G		
<b>RELEASE:</b> <input type="checkbox"/> YES <input type="checkbox"/> NO		<b>Release Start Time</b> _____	
		(Est) <b>Release Stop Time</b> _____ *	
		<b>Release Duration</b> _____ Hrs	
<b>RELEASE FLOW RATE</b> _____ cc/sec			
<b>RELEASE ACTIVITY LEVEL</b> _____ <input type="checkbox"/> mR/hr <input type="checkbox"/> R/hr <input type="checkbox"/> CPM (check one)			
<b>FORECAST (from NWS Green Bay):</b>			
<b>PROTECTIVE ACTION RECOMMENDATIONS:</b>			

\* **NOTE:** IF release stop time is unavailable, THEN use an exposure time of Current Time + 4 Hours as your estimated release stop time.

# RADIATION MONITORS

TIME/DATE \_\_\_\_\_

## CONTAINMENT

				(OFF SCALE) HIGH
R-2	Containment Area	_____	mr/hr	(10 <sup>4</sup> )
R-40	Containment High Range	_____	R/hr	(10 <sup>8</sup> )
R-41	Containment High Range	_____	R/hr	(10 <sup>8</sup> )
R-11	Cont Air Particulate	_____	cpm	(10 <sup>6</sup> )
R-12	Cont Gas	_____	cpm	(10 <sup>6</sup> )
R-37	Containment Vent Low	_____	mr/hr	(10 <sup>4</sup> )
R-38	Containment Vent High	_____	R/hr	(10 <sup>4</sup> )
SPING	Containment Vent Noble Medium (02-07)	_____	cpm	(10 <sup>6</sup> )
SPING	Containment Vent Noble High (02-09)	_____	cpm	(10 <sup>6</sup> )

## AUXILIARY BUILDING

R-4	Charging Pump Room Area (Control Room)	_____	mr/hr	(10 <sup>4</sup> )
R-5	Fuel Handling Area	_____	mr/hr	(10 <sup>4</sup> )
R-9	Letdown Line Area	_____	mr/hr	(10 <sup>2</sup> )
R-13	Aux Bldg Vent (Control Room)	_____	cpm	(10 <sup>6</sup> )
R-14	Aux Bldg Vent	_____	cpm	(10 <sup>6</sup> )
R-35	Aux Bldg Vent Low	_____	mr/hr	(10 <sup>4</sup> )
R-36	Aux Bldg Vent High	_____	R/hr	(10 <sup>4</sup> )
SPING	Aux Bldg Stack Noble Medium (01-07)	_____	cpm	(10 <sup>6</sup> )
SPING	Aux Bldg Stack Noble High (01-09)	_____	cpm	(10 <sup>6</sup> )

## STEAM GENERATORS

R-15	Condenser Air Ejector Gas	_____	cpm	(10 <sup>6</sup> )
R-19	S/G Blowdown	_____	cpm	(10 <sup>6</sup> )
R-31	A Stm Line Monitor 1A Low	_____	mr/hr	(10 <sup>4</sup> )
R-32	A Stm Line Monitor 1A High	_____	R/hr	(10 <sup>4</sup> )
R-33	B Stm Line Monitor 1B Low	_____	mr/hr	(10 <sup>4</sup> )
R-34	B Stm Line Monitor 1B High	_____	R/hr	(10 <sup>4</sup> )