

May 13, 1985

Docket Nos. 50-272
and 50-311

Mr. C. A. McNeill, Jr., Vice President -
Nuclear
Public Service Electric and Gas Company
Post Office Box 236
Hancocks Bridge, New Jersey 08038

Distribution
ORB#1 RDG
L PDR
HThompson
DFischer
LHarmon
BGrimes
TBarhart (8)
EButcher
OPA, CMiles

Docket file
NRC PDR
Gray file (4)
CParrish
OELD
EJordan
JPartlow
WJones
ACRS (10)
RDiggs

Dear Mr. McNeill:

The Commission has issued the enclosed Amendment No. 62 to Facility Operating License No. DPR-70 and Amendment No. 33 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated October 26, 1984 and supplemented November 6, 1984.

These amendments update the Public Service Electric and Gas Company's Nuclear Department organization, SORC responsibilities, membership, and quorum requirements.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,
/s/DCFischer

Donald Fischer, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 62 to DPR-70
2. Amendment No. 33 to DPR-75
3. Safety Evaluation

cc: w/enclosures
See next page

*SEE PREVIOUS WHITE FOR CONCURRENCE

ORB#1:DL*	ORB#1:DL*	BC-ORB#1:DL*
CParrish	DFischer/ts	SVarga
04/23/85	04/23/85	04/24/85

OELD	AD:DL	A-D:DHFS
GLainas	WTRussell	
04/8/85	04/13/85	04/25/85

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04/23/85

BC-ORB#1:DL
SVarco
04/23/85

OELD
04/23/85
AD:OR:DL
GLainas
04/23/85

Mr. C. A. McNeill
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Salem Nuclear Generating Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-70

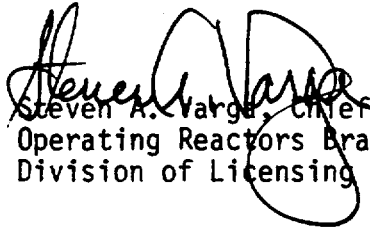
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated October 26, 1984 and supplemented November 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 6 months after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Vargo, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

XVIII
XIX
6-1
6-2
6-3
6-4
6-5
6-6
6-7
6-8
6-9
6-9a
6-10
6-11
6-12
6-12a
6-13

Insert Pages

XVIII
XIX
6-1
6-2
6-3
6-4
6-5
6-6
6-7
6-8
6-9
6-9a
6-10
6-11
6-12
6-12a
6-13

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
<u>6.1 RESPONSIBILITY</u>	6-1
<u>6.2 ORGANIZATION</u>	
Offsite	6-1
Facility Staff	6-1
Shift Technical Advisor	6-6
<u>6.3 FACILITY STAFF QUALIFICATIONS</u>	6-6
<u>6.4 TRAINING</u>	6-6
<u>6.5 REVIEW AND AUDIT</u>	
6.5.1 STATION OPERATIONS REVIEW COMMITTEE	
Function	6-7
Composition	6-7
Alternates	6-7
Meeting Frequency	6-7
Quorum	6-8
Responsibilities	6-9
Review Process	6-9
Authority	6-9
Records	6-9
6.5.2 NUCLEAR SAFETY REVIEW	
Function	6-9
Composition	6-9
Consultants	6-9a
Off-Site Review Group	6-9a
Review	6-9a
Audits	6-10
On-Site Safety Review Group	6-11
Responsibilities	6-11
Authority	6-11

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
6.5.3 TECHNICAL REVIEW AND CONTROL	6-12
<u>6.6 REPORTABLE EVENT ACTION</u>	6-12a
<u>6.7 SAFETY LIMIT VIOLATION</u>	6-12a
<u>6.8 PROCEDURES AND PROGRAMS</u>	6-13
<u>6.9 REPORTING REQUIREMENTS</u>	
6.9.1 ROUTINE REPORTS	6-15
6.9.2 SPECIAL REPORTS	6-19
<u>6.10 RECORD RETENTION</u>	6-19
<u>6.11 RADIATION PROTECTION PROGRAM</u>	6-21
<u>6.12 HIGH RADIATION AREA</u>	6-21
<u>6.13 PROCESS CONTROL PROGRAM</u>	6-22
<u>6.14 OFFSITE DOSE CALCULATION MANUAL</u>	6-22
<u>6.15 MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS</u>	6-23
<u>6.16 ENVIRONMENTAL QUALIFICATION</u>	6-24

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The General Manager - Salem Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Senior Nuclear Shift Supervisor or, during his absence from the Control Room, a designated individual shall be responsible for the Control Room command function. A management directive to this effect, signed by the Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, at least one licensed Senior Reactor Operator shall be in the Control Room area at all times.
- c. A health physics technician# shall be on site when fuel is in the reactor.
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by a licensed Senior Reactor Operator who has no other concurrent responsibilities during this operation.
- e. A site Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include 4 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.
- f. The amount of overtime worked by plant staff members performing safety-related functions must be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

#The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of the health physics technician and/or Fire Brigade members provided immediate action is taken to restore the health physics technician and/or Fire Brigade to within the minimum requirements.

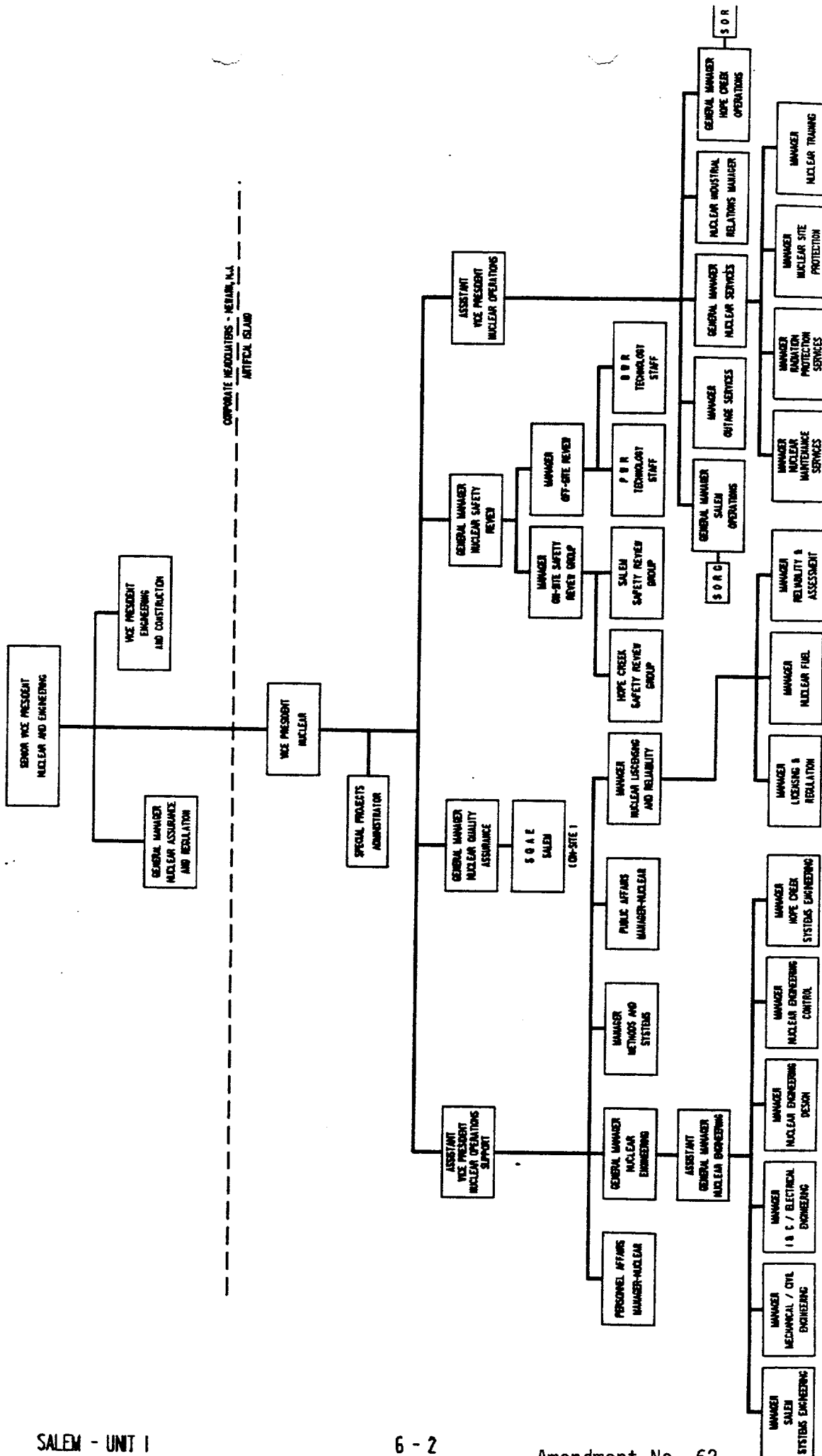
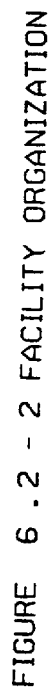


FIGURE 6.2-1 CORPORATE HEADQUARTERS AND OFF-SITE ORGANIZATION FOR MANAGEMENT AND TECHNICAL SUPPORT



Amendment No. 62

140 STA QUALIFIED AND FULFILLS STA REQUIREMENTS OF TABLE 6.2-1 IF SRO
 141 COUNSED WILL BE ASSIGNED NUCLEAR SHIFT SUPERVISORY DUTIES.

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

SALEM UNIT 1

WITH UNIT 2 IN MODES 5 OR 6 OR DE-FUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SNSS	1a	1a
SRO	1b	none
STA	1b	none
NCO	2	1
EO/UO	3	2c
Maintenance Electrician	1	none

WITH UNIT 2 IN MODES 1, 2, 3 OR 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SNSS	1a	1a
SRO	1b	none
STA	1b	none
NCO	2	1
EO/UO	3d	1
Maintenance Electrician	1a	none

- a/ Individual may fill the same position on Unit 2
- b/ Individual who fulfills the STA requirement may fill the same position on Unit 2. The STA, if a licensed SRO, may concurrently fill the SRO position on one unit; the other unit also requires a qualified SRO on shift.
- c/ One of the two required individuals may fill the position on Unit 2, such that there are a total of three EO/UO's for both units.
- d/ One of the three required individuals may fill the same position of Unit 2, such that there are a total of five EO/UO's for both units.

TABLE 6.2-1 (Continued)

- SNSS - **Senior Nuclear Shift Supervisor** with a Senior Reactor Operator License on both units.
- SRO - Individual with a Senior Reactor Operator License on both units (normally, a **Nuclear Shift Supervisor**).
- NCO - **Nuclear Control Operator** with a Reactor Operator License on both units.
- STA - **Shift Technical Advisor** (if licensed as SRO, may be assigned duties as a Nuclear Shift Supervisor).
- EO/UO - **Equipment Operator or Utility Operator.**

Except for the Senior Nuclear Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate the unexpected absence of on-duty shift crew members provided that immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewperson's being late or absent.

During any absence of the Senior Nuclear Shift Supervisor from the Control Room area while the unit is in any MODE, an individual with a valid SRO License shall be designated to assume the Control Room command function.

ADMINISTRATIVE CONTROLS

6.2.3 SHIFT TECHNICAL ADVISOR

6.2.3.1 The Shift Technical Advisor shall serve in an advisory capacity to the Shift Supervisor on matters pertaining to the engineering aspects assuring safe operation of the unit.

6.2.3.2 The Shift Technical Advisor shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Manager - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager - Nuclear Training and shall meet or exceed the requirements of Section 27 of NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the General Manager - Salem Operations on operational matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee (SORC) shall be composed of:

Chairman:	General Manager - Salem Operations
Member and Vice Chairman:	Assistant General Manager - Salem Operations
Member and Vice Chairman:	Maintenance Manager
Member and Vice Chairman:	Operations Manager
Member and Vice Chairman:	Technical Manager
Member:	Operating Engineer
Member:	I & C Engineer
Member:	Senior Nuclear Shift Supervisor
Member:	Technical Engineer
Member:	Maintenance Engineer
Member:	Radiation Protection Engineer
Member:	Chemistry Engineer
Member:	Manager - On Site Safety Review Group or his designee.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman.

- a. Vice Chairmen shall be members of Station management.
- b. No more than two alternates to members shall participate as voting members in SORC activities at any one meeting.
- c. Alternate appointees will only represent their respective department.
- d. Alternates for members will not make up part of the voting quorum when the members the alternate represents is also present.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and five members including alternates.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of: (1) Station Administrative Procedures and changes thereto and (2) Newly created procedures or changes to existing procedures that involve a significant safety issue as described in Section 6.5.3.2.d.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review of the safety evaluations that have been completed under the provisions of 10CFR50.59.
- f. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear and the General Manager - Nuclear Safety Review.
- g. Review of all REPORTABLE EVENTS.
- h. Review of facility operations to detect potential nuclear safety hazards.
- i. Performance of special reviews, investigations or analyses and reports thereon as requested by the General Manager - Salem Operations or General Manager - Nuclear Safety Review.
- j. Review of the Facility Security Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- k. Review of the Facility Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- l. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- m. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear and the General Manager - Nuclear Safety Review.
- n. Review of changes to the PROCESS CONTROL MANUAL and the OFF-SITE DOSE CALCULATION MANUAL.

ADMINISTRATIVE CONTROLS

SORC REVIEW PROCESS

6.5.1.7 A technical review and control system utilizing qualified reviewers shall function to perform the periodic or routine review of procedures and changes thereto. Details of this technical review process are provided in Section 6.5.3.

AUTHORITY

6.5.1.8 The Station Operations Review Committee shall:

- a. Recommend to the General Manager - Salem Operations written approval or disapproval of items considered under 6.5.1.6 (a) through (e) above.
- b. Provide written notification within 24 hours to the Vice President - Nuclear and the General Manager - Nuclear Safety Review of disagreement between the SORC and the General Manager - Salem Operations; however, the General Manager - Salem Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.9 The Station Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear, the General Manager - Nuclear Safety Review and the Manager - Off-Site Review.

6.5.2 NUCLEAR SAFETY REVIEW

FUNCTION

6.5.2.1 The Nuclear Safety Review Department (NSR) shall function to provide the independent safety review program and audit of designated activities.

COMPOSITION

6.5.2.2 NSR shall consist of the General Manager - Nuclear Safety Review, the Manager - On-Site Safety Review Group (SRG), who is supported by at least four dedicated, full-time engineers located on-site, and the Manager Off-Site Review Group (OSR) who is supported by at least four dedicated, full-time engineers located off-site.* The OSR staff shall possess experience and competence in the general areas listed in Section 6.5.2.4. The General Manager and Managers shall determine when additional technical experts should assist in reviews of complex problems.

NSR shall utilize a system of qualified reviewers from other technical organizations to augment its expertise in the disciplines of Section 6.5.2.4. Such qualified reviewers shall meet the same qualification requirements as the NSR staff, and will not have been involved with performance of the original work.

*Since the Nuclear Department is located on Artificial Island site, the terms "on-site" and "off-site" convey the distinction between inside and outside the station fence, respectively

ADMINISTRATIVE CONTROLS

The Manager - Off-Site Review and Staff shall meet or exceed the qualifications described in Section 4.7 of ANS 3.1(1981) and shall be guided by the provisions for independent review described in Section 4.3 of ANSI N18.7 (1976) (ANS 3.2). The Manager - On Site Review and staff will meet or exceed the qualifications described in Section 4.4 of ANS 3.1 (1981).

CONSULTANTS

6.5.2.3 Consultants shall be utilized as determined by the General Manager - Nuclear Safety Review to provide expert advice to the NSR.

OFF-SITE REVIEW GROUP

6.5.2.4 The Off-Site Review Group (OSR) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry
- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical Engineering
- h. Electrical Engineering
- i. Quality Assurance
- j. Nondestructive Testing
- k. Emergency Preparedness

The OSR shall also function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

REVIEW

6.5.2.4.1 The OSR shall review:

- a. Safety Evaluations for
 - 1) Changes to procedures, equipment, or systems and
 - 2) Tests or experiments completed under the provisions of Section 50.59, 10CFR, to verify that such actions did not constitute an unreviewed safety question.

ADMINISTRATIVE CONTROLS

- b. Proposed changes to procedures, equipment, or systems that involve an unreviewed safety question as defined in Section 50.59, 10CFR.
- c. Proposed tests or experiments that involve an unreviewed safety question as defined in Section 50.59, 10CFR.
- d. Proposed changes to Technical Specifications or to the Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems or components.
- i. Reports and meeting minutes of the Station Operations Review Committee.

AUDITS

6.5.2.4.2 Audits of facility activities that are required to be performed under the cognizance of OSR are listed below:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training, and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the Criteria of Appendix "B", 10CFR50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the General Manager - Nuclear Safety Review or the Vice President - Nuclear.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.

ADMINISTRATIVE CONTROLS

- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.

The above audits shall be conducted by the Nuclear Quality Assurance Department or an independent consultant as required. Audit results and recommendations shall be reviewed by OSR.

ON-SITE SAFETY REVIEW GROUP

6.5.2.5 The On-Site Safety Review Group (SRG) shall function to provide: the review of plant design and operating experience for potential opportunities to improve plant safety; the evaluation of plant operations and maintenance activities; and advice to management on the overall quality and safety of plant operations.

The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

RESPONSIBILITIES

6.5.2.5.1 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics, NRC issuances, industry advisories, and other appropriate sources of plant design and operating experience information that may indicate areas for improving plant safety.
- b. Review of selected facility features, equipment, and systems.
- c. Review of selected procedures and plant activities including maintenance, modification, operational problems, and operational analysis.
- d. Surveillance of selected plant operations and maintenance activities to provide independent verification* that they are performed correctly and that human errors are reduced to as low as reasonably achievable.

NSR AUTHORITY

6.5.2.6 NSR shall report to and advise the Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.4 and 6.5.2.5.

*Not responsible for sign-off function

ADMINISTRATIVE CONTROLS

6.5.2.7 Records of NSR activities shall be prepared and maintained. Reports of reviews and audits shall be distributed as follows:

- a. Reports of reviews encompassed by Section 6.5.2.4.1 above, shall be prepared, approved and forwarded to the Vice President - Nuclear, within 14 days following completion of the review.
- b. Audit reports encompassed by Section 6.5.2.4.2 above, shall be forwarded to the Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.5.3 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.3.1 Programs required by Technical Specification 6.8 and other procedures which affect nuclear safety as determined by the General Manager - Salem Operations, other than editorial or typographical changes, shall be reviewed as follows:

PROCEDURE RELATED DOCUMENTS

6.5.3.2 Procedures, Program and changes thereto shall be reviewed as follows:

- a. Each newly created procedure, program or change thereto shall be independently reviewed by an individual knowledgeable in the subject area other than the individual who prepared the procedure, program or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Station Administrative procedures will be approved by the appropriate Station Department Manager or by the Assistant General Manager - Salem Operations. Each Station Department Manager shall be responsible for a predesignated class of procedures. The General Manager - Salem Operations shall approve Station Administrative Procedures, Security Plan implementing procedures and Emergency Plan implementing procedures.
- b. On-the-spot changes to procedures which clearly do not change the intent of the approved procedures shall be approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator License. Revisions to procedures which may involve a change in intent of the approved procedures, shall be reviewed in accordance with Section 6.5.3.2.a above.
- c. Individuals responsible for reviews performed in accordance with item 6.5.3.2.a above shall be approved by the SORC Chairman and designated as Station Qualified Reviewers. A system of Station Qualified Reviewers, each of whom shall possess qualifications that meet or exceed the requirements of Section 4.4 of ANSI N18.1 - 1971, shall be maintained by the SORC Chairman. Each review shall include a written determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

ADMINISTRATIVE CONTROLS

- d. If the Department Manager determines that the documents involved contain significant safety issues, the documents shall be forwarded for SORC review and also to NSR for an independent review to determine whether or not an unreviewed safety question is involved, Pursuant to 10CFR50.59, NRC approval of items involving unreviewed safety questions or Technical Specification changes shall be obtained prior to implementation.

NON-PROCEDURE RELATED DOCUMENTS

6.5.3.3 Tests or experiments, changes to Technical Specifications, and changes to equipment or systems shall be forwarded for SORC review and also to NSR for an independent review to determine whether or not an unreviewed safety question is involved. The results of NSR reviews will be provided to SORC. Recommendations for approval are made by SORC to the General Manager - Salem Operations. Pursuant to 10CFR50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

RECORDS

6.5.3.4 Written records of reviews performed in accordance with item 6.5.3.2a above, including recommendations for approval or disapproval, shall be maintained. Copies shall be provided to the General Manager - Salem Operations, SORC, NSR and/or NRC as necessary when their reviews are required.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Section 50.73 to 10CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the resultant Licensee Event Report submitted to NSR and the Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President - Nuclear and General Manager - Nuclear Safety Review shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the General Manager - Nuclear Safety Review and the Vice President - Nuclear within 14 days of the violation.

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved in accordance with Specification 6.5.1.6 or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 On-the-spot changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented and receives the same level of review and approval as the original procedure under Specification 6.5.3.2a within 14 days of implementation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 33
License No. DPR-75


1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated October 26, 1984 and supplemented November 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 33, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 6 months after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 13, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 33

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

XVIII
XIX
6-1
6-2
6-3
6-4
6-5
6-6
6-7
6-8
6-9
6-9a
6-10
6-11
6-12
6-12a
6-13

Insert Pages

XVIII
XIX
6-1
6-2
6-3
6-4
6-5
6-6
6-7
6-8
6-9
6-9a
6-10
6-11
6-12
6-12a
6-13

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
<u>6.1 RESPONSIBILITY</u>	6-1
<u>6.2 ORGANIZATION</u>	
Offsite	6-1
Facility Staff	6-1
Shift Technical Advisor	6-6
<u>6.3 FACILITY STAFF QUALIFICATIONS</u>	6-6
<u>6.4 TRAINING</u>	6-6
<u>6.5 REVIEW AND AUDIT</u>	
6.5.1 STATION OPERATIONS REVIEW COMMITTEE	
Function	6-7
Composition	6-7
Alternates	6-7
Meeting Frequency	6-7
Quorum	6-7
Responsibilities	6-8
Review Process	6-9
Authority	6-9
Records	6-9
6.5.2 NUCLEAR SAFETY REVIEW	
Function	6-9
Composition	6-9
Consultants	6-9a
Off-Site Review Group	6-9a
Review	6-9a
Audits	6-10
On-Site Safety Review Group	6-11
Responsibilities	6-11
Authority	6-11

INDEX

ADMINISTRATIVE CONTROLS

<u>SECTION</u>	<u>PAGE</u>
6.5.3 TECHNICAL REVIEW AND CONTROL	6-12
<u>6.6 REPORTABLE EVENT ACTION</u>	6-12a
<u>6.7 SAFETY LIMIT VIOLATION</u>	6-12a
<u>6.8 PROCEDURES AND PROGRAMS</u>	6-13
<u>6.9 REPORTING REQUIREMENTS</u>	
6.9.1 ROUTINE REPORTS	6-15
6.9.2 SPECIAL REPORTS	6-19
<u>6.10 RECORD RETENTION</u>	6-19
<u>6.11 RADIATION PROTECTION PROGRAM</u>	6-21
<u>6.12 HIGH RADIATION AREA</u>	6-21
<u>6.13 PROCESS CONTROL PROGRAM</u>	6-22
<u>6.14 OFFSITE DOSE CALCULATION MANUAL</u>	6-22
<u>6.15 MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS</u>	6-23

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The General Manager - Salem Operations shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Senior Nuclear Shift Supervisor or, during his absence from the Control Room, a designated individual shall be responsible for the Control Room command function. A management directive to this effect, signed by the Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, at least one licensed Senior Reactor Operator shall be in the Control Room area at all times.
- c. A health physics technician# shall be on site when fuel is in the reactor.
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by a licensed Senior Reactor Operator who has no other concurrent responsibilities during this operation.
- e. A site Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include 4 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.
- f. The amount of overtime worked by plant staff members performing safety-related functions must be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

#The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of the health physics technician and/or Fire Brigade members provided immediate action is taken to restore the health physics technician and/or Fire Brigade to within the minimum requirements.

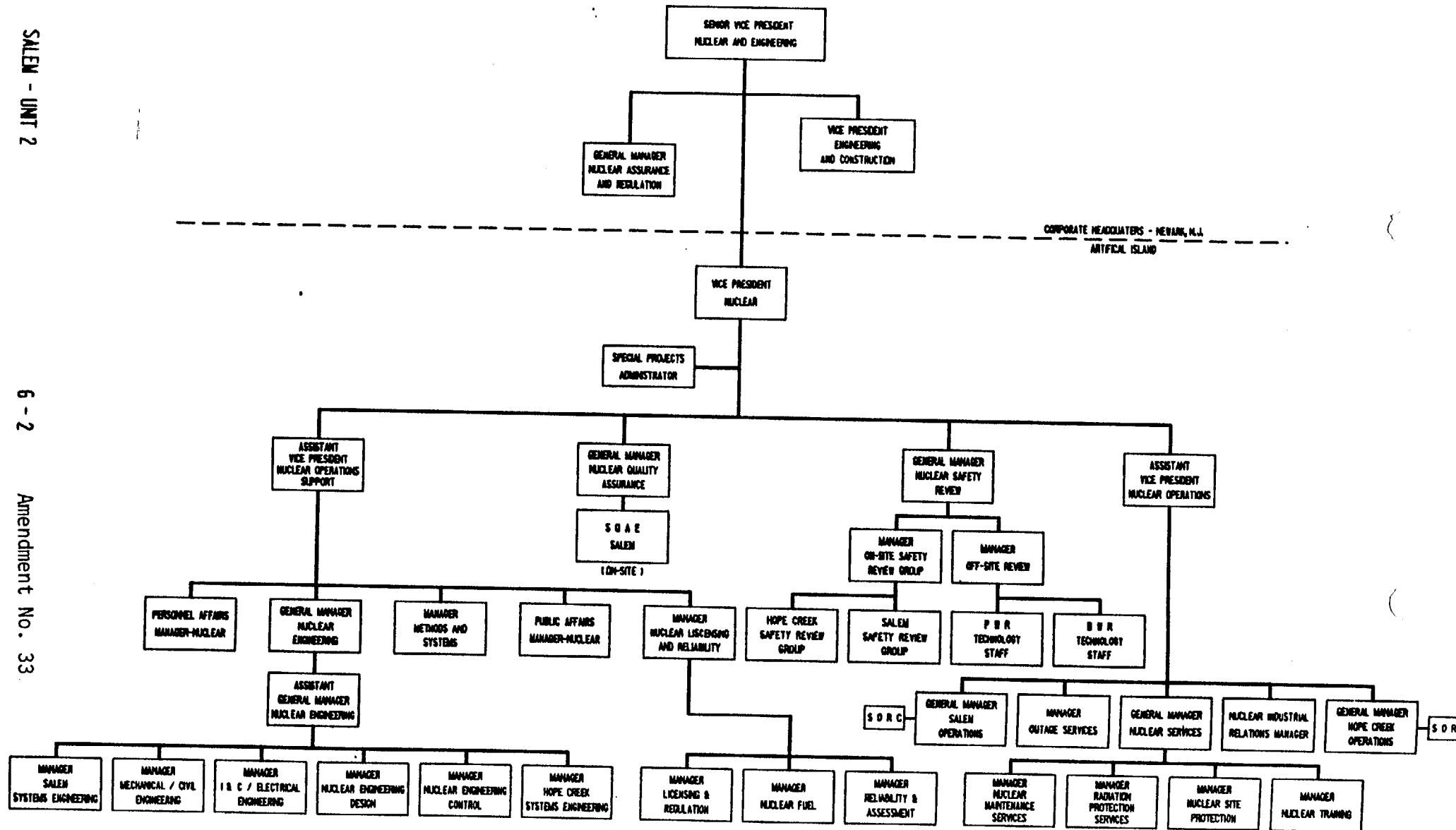


FIGURE 6.2-1 CORPORATE HEADQUARTERS AND OFF-SITE ORGANIZATION FOR MANAGEMENT AND TECHNICAL SUPPORT

▲ EITHER THE OPERATIONS MANAGER OR ASSISTANT FOR THE POSITION WILL MEET THE QUALIFICATIONS OF ANS 101-171 FOR THE POSITION OF OPERATIONS MANAGER. ALL SENIOR SHIFT SUPERVISORS SHALL REPORT TO AN INDIVIDUAL WHO SHALL HOLD A SENIOR REACTOR OPERATORS LICENSE AND HAVE SIX YEARS OF RESPONSIBLE POWER PLANT EXPERIENCE OF WHICH A MINIMUM OF TWO YEARS SHALL BE IN A POWER PLANT OPERATOR'S POSITION. A MINIMUM OF THE REMAINING TWO OF THE REMAINING FOUR YEARS OF POWER PLANT EXPERIENCE MAY BE FULFILLED BY SATISFACTORY COMPLETION OF ACADEMIC OR RELATED TECHNICAL TRAINING ON A PER TIME BASIS.

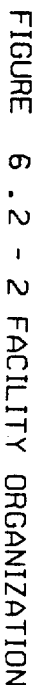


TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITIONSALEM UNIT 2

WITH UNIT 1 IN MODES 5 OR 6 OR DE-FUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SNSS	1a	1a
SRO	1b	none
STA	1b	none
NCO	2	1
EO/UO	3	2c
Maintenance Electrician	1	none

WITH UNIT 1 IN MODES 1, 2, 3 OR 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SNSS	1a	1a
SRO	1b	none
STA	1b	none
NCO	2	1
EO/UO	3d	1
Maintenance Electrician	1a	none

- a/ Individual may fill the same position on Unit 1
- b/ Individual who fulfills the STA requirement may fill the same position on Unit 1. The STA, if a licensed SRO, may concurrently fill the SRO position on one unit; the other unit also requires a qualified SRO on shift.
- c/ One of the two required individuals may fill the position on Unit 1, such that there are a total of three EO/UO's for both units.
- d/ One of the three required individuals may fill the same position of Unit 1, such that there are a total of five EO/UO's for both units.

TABLE 6.2-1 (Continued)

- SNSS - **Senior Nuclear Shift Supervisor** with a Senior Reactor Operator License on both units.
- SRO - Individual with a Senior Reactor Operator License on both units (normally, a **Nuclear Shift Supervisor**).
- NCO - **Nuclear Control Operator** with a Reactor Operator License on both units.
- STA - **Shift Technical Advisor** (if licensed as SRO, may be assigned duties as a Nuclear Shift Supervisor).
- EO/UO - **Equipment Operator or Utility Operator.**

Except for the Senior Nuclear Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate the unexpected absence of on-duty shift crew members provided that immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewperson's being late or absent.

During any absence of the Senior Nuclear Shift Supervisor from the Control Room area while the unit is in any MODE, an individual with a valid SRO License shall be designated to assume the Control Room command function.

ADMINISTRATIVE CONTROLS

6.2.3 SHIFT TECHNICAL ADVISOR

6.2.3.1 The Shift Technical Advisor shall serve in an advisory capacity to the Shift Supervisor on matters pertaining to the engineering aspects—assuring safe operation of the unit.

6.2.3.2 The Shift Technical Advisor shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Manager - Nuclear Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Manager - Nuclear Training and shall meet or exceed the requirements of Section 27 of NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the General Manager - Salem Operations on operational matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee (SORC) shall be composed of:

Chairman:	General Manager - Salem Operations
Member and Vice Chairman:	Assistant General Manager - Salem Operations
Member and Vice Chairman:	Maintenance Manager
Member and Vice Chairman:	Operations Manager
Member and Vice Chairman:	Technical Manager
Member:	Operating Engineer
Member:	I & C Engineer
Member:	Senior Nuclear Shift Supervisor
Member:	Technical Engineer
Member:	Maintenance Engineer
Member:	Radiation Protection Engineer
Member:	Chemistry Engineer
Member:	Manager - On Site Safety Review Group or his designee.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman.

- a. Vice Chairmen shall be members of Station management.
- b. No more than two alternates to members shall participate as voting members in SORC activities at any one meeting.
- c. Alternate appointees will only represent their respective department.
- d. Alternates for members will not make up part of the voting quorum when the members the alternate represents is also present.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

QUORUM

6.5.1.5 The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and five members including alternates.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of: (1) Station Administrative Procedures and changes thereto and (2) Newly created procedures or changes to existing procedures that involve a significant safety issue as described in Section 6.5.3.2.d.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review of the safety evaluations that have been completed under the provisions of 10CFR50.59.
- f. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear and the General Manager - Nuclear Safety Review.
- g. Review of all REPORTABLE EVENTS.
- h. Review of facility operations to detect potential nuclear safety hazards.
- i. Performance of special reviews, investigations or analyses and reports thereon as requested by the General Manager - Salem Operations or General Manager - Nuclear Safety Review.
- j. Review of the Facility Security Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- k. Review of the Facility Emergency Plan and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- l. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the General Manager - Nuclear Safety Review.
- m. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear and the General Manager - Nuclear Safety Review.
- n. Review of changes to the PROCESS CONTROL MANUAL and the OFF-SITE DOSE CALCULATION MANUAL.

ADMINISTRATIVE CONTROLS

SORC REVIEW PROCESS

6.5.1.7 A technical review and control system utilizing qualified reviewers shall function to perform the periodic or routine review of procedures and changes thereto. Details of this technical review process are provided in Section 6.5.3.

AUTHORITY

6.5.1.8 The Station Operations Review Committee shall:

- a. Recommend to the General Manager - Salem Operations written approval or disapproval of items considered under 6.5.1.6 (a) through (e) above.
- b. Provide written notification within 24 hours to the Vice President - Nuclear and the General Manager - Nuclear Safety Review of disagreement between the SORC and the General Manager - Salem Operations; however, the General Manager - Salem Operations shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.9 The Station Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear, the General Manager - Nuclear Safety Review and the Manager - Off-Site Review.

6.5.2 NUCLEAR SAFETY REVIEW

FUNCTION

6.5.2.1 The Nuclear Safety Review Department (NSR) shall function to provide the independent safety review program and audit of designated activities.

COMPOSITION

6.5.2.2 NSR shall consist of the General Manager - Nuclear Safety Review, the Manager - On-Site Safety Review Group (SRG), who is supported by at least four dedicated, full-time engineers located on-site, and the Manager Off-Site Review Group (OSR) who is supported by at least four dedicated, full-time engineers located off-site.* The OSR staff shall possess experience and competence in the general areas listed in Section 6.5.2.4. The General Manager and Managers shall determine when additional technical experts should assist in reviews of complex problems.

NSR shall utilize a system of qualified reviewers from other technical organizations to augment its expertise in the disciplines of Section 6.5.2.4. Such qualified reviewers shall meet the same qualification requirements as the NSR staff, and will not have been involved with performance of the original work.

*Since the Nuclear Department is located on Artificial Island site, the terms "on-site" and "off-site" convey the distinction between inside and outside the station fence, respectively.

ADMINISTRATIVE CONTROLS

The Manager - Off-Site Review and Staff shall meet or exceed the qualifications described in Section 4.7 of ANS 3.1(1981) and shall be guided by the provisions for independent review described in Section 4.3 of ANSI N18.7 (1976) (ANS 3.2). The Manager - On Site Review and staff will meet or exceed the qualifications described in Section 4.4 of ANS 3.1 (1981).

CONSULTANTS

6.5.2.3 Consultants shall be utilized as determined by the General Manager - Nuclear Safety Review to provide expert advice to the NSR.

OFF-SITE REVIEW GROUP

6.5.2.4 The Off-Site Review Group (OSR) shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear Power Plant Operations
- b. Nuclear Engineering
- c. Chemistry and Radiochemistry
- d. Metallurgy
- e. Instrumentation and Control
- f. Radiological Safety
- g. Mechanical Engineering
- h. Electrical Engineering
- i. Quality Assurance
- j. Nondestructive Testing
- k. Emergency Preparedness

The OSR shall also function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

REVIEW

6.5.2.4.1 The OSR shall review:

- a. Safety Evaluations for
 - 1) Changes to procedures, equipment, or systems and
 - 2) Tests or experiments completed under the provisions of Section 50.59, 10CFR, to verify that such actions did not constitute an unreviewed safety question.

ADMINISTRATIVE CONTROLS

- b. Proposed changes to procedures, equipment, or systems that involve an unreviewed safety question as defined in Section 50.59, 10CFR.
- c. Proposed tests or experiments that involve an unreviewed safety question as defined in Section 50.59, 10CFR.
- d. Proposed changes to Technical Specifications or to the Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems or components.
- i. Reports and meeting minutes of the Station Operations Review Committee.

AUDITS

6.5.2.4.2 Audits of facility activities that are required to be performed under the cognizance of OSR are listed below:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training, and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the Criteria of Appendix "B", 10CFR50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the General Manager - Nuclear Safety Review or the Vice President - Nuclear.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.

Amendment No. 33

ADMINISTRATIVE CONTROLS

- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.

The above audits shall be conducted by the Nuclear Quality Assurance Department or an independent consultant as required. Audit results and recommendations shall be reviewed by OSR.

ON-SITE SAFETY REVIEW GROUP

6.5.2.5 The On-Site Safety Review Group (SRG) shall function to provide: the review of plant design and operating experience for potential opportunities to improve plant safety; the evaluation of plant operations and maintenance activities; and advice to management on the overall quality and safety of plant operations.

The SRG shall make recommendations for revised procedures, equipment modifications, or other means of improving plant safety to appropriate station/corporate management.

RESPONSIBILITIES

6.5.2.5.1 The SRG shall be responsible for:

- a. Review of selected plant operating characteristics, NRC issuances, industry advisories, and other appropriate sources of plant design and operating experience information that may indicate areas for improving plant safety.
- b. Review of selected facility features, equipment, and systems.
- c. Review of selected procedures and plant activities including maintenance, modification, operational problems, and operational analysis.
- d. Surveillance of selected plant operations and maintenance activities to provide independent verification* that they are performed correctly and that human errors are reduced to as low as reasonably achievable.

NSR AUTHORITY

6.5.2.6 NSR shall report to and advise the Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.4 and 6.5.2.5.

*Not responsible for sign-off function

ADMINISTRATIVE CONTROLS

6.5.2.7 Records of NSR activities shall be prepared and maintained. Reports of reviews and audits shall be distributed as follows:

- a. Reports of reviews encompassed by Section 6.5.2.4.1 above, shall be prepared, approved and forwarded to the Vice President - Nuclear, within 14 days following completion of the review.
- b. Audit reports encompassed by Section 6.5.2.4.2 above, shall be forwarded to the Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.5.3 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.3.1 Programs required by Technical Specification 6.8 and other procedures which affect nuclear safety as determined by the General Manager - Salem Operations, other than editorial or typographical changes, shall be reviewed as follows:

PROCEDURE RELATED DOCUMENTS

6.5.3.2 Procedures, Program and changes thereto shall be reviewed as follows:

- a. Each newly created procedure, program or change thereto shall be independently reviewed by an individual knowledgeable in the subject area other than the individual who prepared the procedure, program or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Station Administrative procedures will be approved by the appropriate Station Department Manager or by the Assistant General Manager - Salem Operations. Each Station Department Manager shall be responsible for a predesignated class of procedures. The General Manager - Salem Operations shall approve Station Administrative Procedures, Security Plan implementing procedures and Emergency Plan implementing procedures.
- b. On-the-spot changes to procedures which clearly do not change the intent of the approved procedures shall be approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator License. Revisions to procedures which may involve a change in intent of the approved procedures, shall be reviewed in accordance with Section 6.5.3.2.a above.
- c. Individuals responsible for reviews performed in accordance with item 6.5.3.2a above shall be approved by the SORC Chairman and designated as Station Qualified Reviewers. A system of Station Qualified Reviewers, each of whom shall possess qualifications that meet or exceed the requirements of Section 4.4 of ANSI N18.1 - 1971, shall be maintained by the SORC Chairman. Each review shall include a written determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

ADMINISTRATIVE CONTROLS

- d. If the Department Manager determines that the documents involved contain significant safety issues, the documents shall be forwarded for SORC review and also to NSR for an independent review to determine whether or not an unreviewed safety question is involved, Pursuant to 10CFR50.59, NRC approval of items involving unreviewed safety questions or Technical Specification changes shall be obtained prior to implementation.

NON-PROCEDURE RELATED DOCUMENTS

6.5.3.3 Tests or experiments, changes to Technical Specifications, and changes to equipment or systems shall be forwarded for SORC review and also to NSR for an independent review to determine whether or not an unreviewed safety question is involved. The results of NSR reviews will be provided to SORC. Recommendations for approval are made by SORC to the General Manager - Salem Operations. Pursuant to 10CFR50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

RECORDS

6.5.3.4 Written records of reviews performed in accordance with item 6.5.3.2a above, including recommendations for approval or disapproval, shall be maintained. Copies shall be provided to the General Manager - Salem Operations, SORC, NSR and/or NRC as necessary when their reviews are required.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Section 50.73 to 10CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SORC and the resultant Licensee Event Report submitted to NSR and the Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President - Nuclear and General Manager - Nuclear Safety Review shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the General Manager - Nuclear Safety Review and the Vice President - Nuclear within 14 days of the violation.

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved in accordance with Specification 6.5.1.6 or 6.5.3, as appropriate, prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 On-the-spot changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented and receives the same level of review and approval as the original procedure under Specification 6.5.3.2a within 14 days of implementation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. DPR-70
AND AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATION STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

Introduction

Public Service Electric and Gas Company (PSE&G) requested by letter dated October 26, 1984, and supplemented November 6, 1984, changes to the Technical Specifications for the Salem Generating Station, Units 1 and 2. The amendment request consisted of an update to their Nuclear Department organization, SORC responsibilities, membership, and quorum requirements. Technical Review and Control, Nuclear Safety Review and a clarification of shift composition requirements were also included as a part of the request.

Evaluation and Summary

The following 9 areas were reviewed for acceptability, and are summarized below:

1. Figure 6.2.1 (Corporate Headquarters and Off-site Organization for Management and Technical Support). This organization has been revised to establish the new positions of Assistant Vice President Nuclear Operations Support, General Manager Nuclear Quality Assurance, General Manager Nuclear Safety Review, and Assistant Vice President Nuclear Operations, reporting to the Vice President Nuclear. This change does not delete any technical support provided to the Salem units by the

previous organization, but reduces the span of control of the Vice President and provides increased management direction to the support of the Salem units. This organization has also been reviewed during the Hope Creek Operating License review and further details can be found in the Hope Creek Safety Evaluation Report (NUREG-1048).

We have reviewed this change and find it meets the guidance of draft NUREG-0731, and the acceptance criteria of Section 13.1.1 of NUREG-0800, the Standard Review Plan, and is acceptable.

2. Figure 6.2-2 (Facility Organization). The Licensee has deleted the position of Station Planning Engineer, a discipline that we do not require for the safe operation of nuclear power plants.

The Licensee has added to Figure 6.2-2 a Nuclear Fire Protection Supervisor who reports offsite to the Nuclear Fire and Safety Administrator who in turn reports to the Manager-Nuclear Site Protection. The Manager - Nuclear Site Protection is responsible for the fire protection program implementation, a responsibility formerly held by the General Manager Salem Operations.

We find these changes acceptable as they do not delete the technical support we require for the Salem Units 1 and 2, and they meet the acceptance criteria of Section 13.1.2 of NUREG-0800, the Standard Review Plan.

3. Table 6.2-1 (Minimum Shift Crew Composition)

PSE&G has clarified Table 6.2-1 relative to the use of the STA as a Senior Licensed Operator on one unit, provided there are two other SROs on shift and the individual holds a Senior Operator's License. PSE&G normally operates with three SRO's on shift, one more than we require.

We find this change acceptable as it does not reduce the number of Senior Licensed Operators on shift and meets the requirements of 10 CFR Part 50.54(m).

4. Section 6.5.1 (Station Operations Review Committee)

The licensee has revised the composition of the Station Operations Review Committee (SORC) to delete the positions of Operations Manager, Technical Manager and Senior Radiation Protection Supervisor, and add the position of Manager On-Site Safety Review Group. In addition, PSE&G has revised the provisions for alternates for SORC members and increased the quorum requirements to constitute a majority of the members.

The licensee has revised the responsibilities of the SORC to delete the review of procedures that do not involve significant safety issues and the periodic or routine review of procedures and changes thereto. These items will be reviewed by a separate technical review and control function as described under Section 6.5.3.

We find these changes acceptable as they meet the staff requirements described in Section 13.4 of NUREG-0800, the Standard Review Plan.

5. Section 6.5.2 (Nuclear Review Board)

The licensee has abolished the Nuclear Review Board and established the Nuclear Safety Review Department (NSR) under the General Manager - Nuclear Safety Review. Reporting to the General Manager - Nuclear Safety Review is a Manager Off-Site Review Group (OSR) who is supported by at least four full time engineers. The OSR function will include the review of all items formerly reviewed by the NRB.

We find these changes acceptable as they meet the staff requirements described in Section 13.4 of NUREG-0800, the Standard Review Plan.

6. Section 6.5.2.5 (On-Site Safety Review Group [SRG], formerly the Safety Review Group).

The SRG, which meets the staff's requirements for an Independent Safety Engineering Group, remains intact except that it now reports to the General Manager-Nuclear Safety Review.

We find this change acceptable as it meets the staff requirement described in Task Action Plan Item 1.B.1.2 of NUREG-0737, Clarification of TMI Action Plan Requirements.

7. Section 6.5.3 (Technical Review and Control)

The licensee has established a technical review and control function to provide for the independent review of new procedures and programs and changes thereto as required by Section 6.8 of the Technical Specifications. Procedures will be approved by predesignated Department Managers. The reviews will include cross-disciplinary review where necessary.

Qualification of reviewers will meet or exceed the requirements of Section 4.4 of ANSI N18.1-1971. We find these changes acceptable as they meet the staff requirements described in Section 13.4 of NUREG-0800, the Standard Review Plan.

8. Section 6.8.2 - Procedures and Programs

The license has revised this section to conform to the revised program for the review and approval of procedures.

We find these changes acceptable as they meet the staff requirements described in Section 13.5.1 of NUREG-0800, the Standard Review Plan.

9. Numerous other changes have been made throughout Section 6 to reflect changed titles and reporting relationships that have been changed by the revised organization.

The licensee's submittal of November 6, 1984, was made to clarify the language of the original submittal dated October 26, 1984, and does not contain substantive changes.

Environmental Consideration

These amendments involve a change in the installation or use of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner,
- and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 13, 1985

Principal Contributor:

F. Allenspach