

Serial: RNP-RA/01-0107
JUN 1 2 2001

United States Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 DOCKET NO. 50-261/LICENSE NO. DPR-23

### MONTHLY OPERATING REPORT

Ladies and Gentlemen:

In accordance with the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 Technical Specification (TS) Section 5.6.4, "Monthly Operating Reports," and HBRSEP, Unit No. 2 Technical Requirements Manual 6.2, "Monthly Operating Report," Carolina Power & Light Company submits the enclosed report of operating statistics and shutdown experience for the month of May 2001. In accordance with TS Section 5.6.4, this report is being submitted to the NRC by the 15th day of the month following the calendar month covered by the report.

Attachments I, II, and III provide the Monthly Operating Report in accordance with TS Section 5.6.4 and TRM 6.2. Attachment IV provides the Steam Generator Tube Inspection Report information required by TS Section 5.6.8.b.

If you have any questions concerning this matter, please contact Mr. H. K. Chernoff.

Sincerely,

B. L. Fletcher III

Manager - Regulatory Affairs

Robinson Nuclear Plant 3581 West Entrance Road Hartsville, SC 29550

TEU

United States Nuclear Regulatory Commission

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## DJS/djs

### Attachments

- I. Plant Performance Data System Appendix C Operating Data Report
- II. Appendix B Average Daily Power Level
- III. Appendix D Unit Shutdowns and Power Reductions
- IV. Steam Generator Tube Inspection Report
- c: NRC Resident Inspector, HBRSEP
  - L. A. Reyes, NRC, Region II
  - R. Subbaratnam, NRC, NRR

Attachment I to Serial: RNP-RA/01-0107

CP&L CO RUN DATE 06/01/01 **RUN TIME 12:23:379** 

### PLANT PERFORMANCE DATA SYSTEM APPENDIX C - OPERATING DATA REPORT H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

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**DOCKET NO. 050-0261** COMPLETED BY TOM BARDAUSKAS TELEPHONE (843) 857-1053

#### **OPERATING STATUS**

7.

UNIT NAME: H. B. ROBINSON STEAM ELECTRIC PLANT (HBRSEP), NOTES: 1. UNIT NO. 2 REPORTING PERIOD: MAY 2001 2. 3. LICENSED THERMAL POWER (MWT): 2300 NAMEPLATE RATING (GROSS MWE): 739.0 4. DESIGN ELECTRICAL RATING (NET MWE): 700.0 5. MAX DEPENDABLE CAPACITY (GROSS MWE): 700.0 6.

- IF CHANGES OCCUR IN CAPACITY RATING (ITEMS 3 THROUGH 7) SINCE LAST REPORT, 8. **GIVE REASONS:**
- POWER LEVEL TO WHICH RESTRICTED IF ANY (NET MWE): None 9.

MAX DEPENDABLE CAPACITY (NET MWE): 683.0

REASONS FOR RESTRICTION, IF ANY: 10.

		THIS MONTH	YR TO DATE	CUMULATIVE
11.	HOURS IN REPORTING PERIOD	744.00	3623.00	265199.00
12.	NUMBER OF HOURS REACTOR CRITICAL	493.73	2798.38	198670.59
13.	REACTOR RESERVE SHUTDOWN HRS	.00	.00	3314.65
14.	HOURS GENERATOR ON LINE	479.87	2782.95	195381.26
15.	UNIT RESERVE SHUTDOWN HOURS	.00	.00	23.20
16.	GROSS THERMAL ENERGY GEN. (MWH)	1048761.36	5959397.52	411156178.80
17.	GROSS ELEC. ENERGY GEN. (MWH)	341808.00	1974185.00	133777674.00
18.	NET ELEC. ENERGY GENERATED (MWH)	322819.00	1874354.00	126690664.00
19.	UNIT SERVICE FACTOR	64.50	76.81	73.67
20.	UNIT AVAILABILITY FACTOR	64.50	76.81	73.68
21.	UNIT CAP. FACTOR (USING MDC NET)	63.53	75.75	71.17
22.	UNIT CAP. FACTOR (USING DER NET)	61.99	73.91	68.25
23.	UNIT FORCED OUTAGE RATE	.00	.00	11.83
24.	SHUTDOWNS SCHED. OVER NEXT 6 MONTHS (7	TYPE DATE AND DURAT	ION OF EACH):	

IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF START UP: UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): FORECAST ACHIEVED 25. 26.

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION Attachment II to Serial: RNP-RA/01-0107

CP&L CO RUN DATE 06/01/01 RUN TIME 12:23:37 APPENDIX B - AVERAGE DAILY POWER LEVEL H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

RPD39-000

DOCKET NO. 050-0261 COMPLETED BY: THOMAS BARDAUSKAS TELEPHONE (843) 857-1053

### **MAY 2001**

DAY	AVG. DAILY POWER LEVEL (MWE-NET)	DAY	AVG. DAILY POWER LEVEL (MWE-NET)
1	o	17	711*
2	o	18	714*
3	o	19	716*
4	o	20	715*
5	o	21	716*
6	0	22	715*
7	o	23	711*
8	o	24	711*
9	o	25	714*
10	. 0	26	712*
11	o	27	712*
12	221	28	712*
13	513	29	711*
14	708*	30	710*
15	713*	31	710*
16	714*		

<sup>\*</sup>The maximum dependable net capacity is 683 MWE. The Plant operated above the maximum dependable capacity due to the condenser water inlet temperature  $<95^{\circ}F$ .

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# APPENDIX D UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-0261
UNIT NAME HBRSEP, UNIT NO. 2
DATE 06/01/01
COMPLETED BY THOMAS BARDAUSKAS
TELEPHONE (843) 857-1053

REPORT MONTH: MAY 2001

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
01.001	4/7/01	S	264.13	С	1	N/A			Scheduled Shutdown: RO-20

F: Forced
S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method: Exhibit G - Instructions

1-Manual For Preparation of Data

2-Manual Scram. Entry Sheets for Licensee 3-Automatic Scram. Event Report (LER) File

(NUREG-0161)

<sup>5</sup> Exhibit 1 - Same Source

SUMMARY: Continuation of refueling outage (RO) 20. The outage ended on 5/12/01. There were no challenges to the pressurizer power operated relief valves or the pressurizer safety valves during the month. A steam generator inspection occurred during this refueling outage. Steam generator tube inspections performed during RO-20 are summarized in Attachment IV to this letter.

4-Other (Explain)

### Steam Generator Tube Inspection Report

As required by the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 Technical Specification Section 5.6.8, "Steam Generator Tube Inspection Report," this document summarizes the Steam Generator (S/G) inspection program and the results of the examination that was performed at the HBRSEP, Unit No. 2 during Refueling Outage (RO) 20.

### 1. Tubes Inspected

The S/G examination program included the use of multi-frequency Bobbin Coil, and Motorized Rotating Pancake Coil or Plus Point Coil (RPC) probes. RPC examinations were performed in S/Gs B & C on approximately 20% of the inlet (hot leg) side historical manufacturing buff marks, dents, and on suspect bobbin indications (diagnostics). RPC examinations were also performed on 50% of the u-bend region of row 1 and 2 and 50% of the hot leg top-of-tubesheet (TTS) area. S/Gs B & C had approximately 50% of all open tubes tested with a Bobbin Coil probe.

S/G	Test	# Of Tubes
В	Bobbin Cold Leg R1-2, R6+	1470
В	Bobbin Cold Leg R3-5	149
В	Bobbin Hot Leg R1-2 (rows 1-2 tubes are the same tubes from the hot	89
	and cold leg)	
В	Bobbin Total Tubes	1619
В	RPC Cold Leg (Wrapper MOD tubes)	3
В	RPC Cold Leg (Diagnostics of bobbin indications)	12
В	RPC Cold Leg Row 1-2 U-Bend	89
В	RPC Hot Leg (Dent sample)	46
В	RPC Hot Leg (Benign indication sample)	47
В	RPC Hot Leg (Wrapper MOD tubes)	3
В	RPC Hot Leg (Diagnostics of bobbin indications)	2
В	RPC Hot Leg TTS	1714
C	Bobbin Cold Leg R1-2, R6+	1573
C	Bobbin Cold Leg R3-5	124
C	Bobbin Hot Leg R1-2 (Row 1-2 tubes are the same tubes from the hot	98
	and cold leg)	
C	Bobbin Total Tubes	1697
C	RPC Cold Leg (Wrapper MOD tube)	1
C	RPC Cold Leg (Diagnostics of Bobbin indication)	1
C	RPC Cold Leg R1-2 U-Bend	98
C	RPC Hot Leg (Dent Sample)	22
C	RPC Hot Leg (Benign indication sample)	9
C	RPC Hot Leg (Wrapper MOD tubes)	2
C	RPC Hot Leg (Potential PLP bounding tubes)	4
C	RPC Hot Leg (Diagnostics of bobbin indications)	5
C	RPC Hot Leg TTS	1697

### 2. Wall Thickness Penetration

	Tube	Tube	<u></u>	
SG	Row	Column	Location	% Depth
В	1	1	CTS +13.07	17
			CTS +13.10	18
			CTS +16.20	18
			CTS +09.62	18
В	7	1	CTS +01.15	19
В	40	25	CTS +00.59	21
В	5	48	CTS +01.46	28
			CTS +01.71	29
В	5	49	CTS +01.77	24
			CTS +01.78	27
В	33	78	CTS +00.62	20
			CTS +00.63	20
В	26	84	CTS +00.69	20
В	1	92	CTS +10.92	12
			CTS +11.86	12
			CTS +08.55	12
		:	CTS +09.62	13
			HTS +01.94	14
			HTS +14.12	15
			HTS +02.82	19
В	2	92	CTS +10.66	10
С	32	26	HTS +00.28	32
С	44	38	HTS +00.48	18
С	37	45	03A +00.00	6
			04A -00.06	3
С	44	56	FBH +00.45	43
С	44	57	HTS +00.58	17
			HTS +01.18	17
С	35	61	02A +00.41	5 5
			03A +00.48	5
С	38	62	02A -00.05	4
			03A -00.13	13
С	33	78	HTS +00.71	15
С	3	91	HTS +00.28	28

## Location definitions:

CTS Cold leg top of tubesheet HTS Hot leg top of tubesheet

FBH Flow distribution baffle on hot leg side

02A, 03A, and 04A Anti-Vibration Bars 2, 3, and 4

The facility had zero primary to secondary leakage at the time the refueling outage was entered.

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## 3. Tubes Plugged

Steam Generator	Tube Plugged	Comments
В	R43C55	A dent was identified in this tube. The dent prevented an examination by a qualified bobbin probe. Use of a Rotating Pancake Coil probe (Plus Point) resulted in poor quality data as a result of the configuration and location of the dent. A preventive plug was installed.
С	R44C56	A wear indication was identified at the flow distribution baffle on the hot leg side of the steam generator. A preventive plug was installed.
С	R33C34	An obstruction identified in the tube above the sixth tube support plate on the hot leg side of the steam generator. A preventive plug was installed.
С	R32C26	A wear indication was identified near the tube sheet on the hot leg side of the steam generator. A preventive plug was installed.