

August 13, 1999

Mr. Robert J. Wanczyk
Acting Director of Operations
Vermont Yankee Nuclear Power Corporation
185 Old Ferry Road
Brattleboro, VT 05301

SUBJECT: VERMONT YANKEE NUCLEAR POWER STATION - ISSUANCE OF
AMENDMENT RE: CLARIFICATION OF INSERVICE INSPECTION
REQUIREMENTS (TAC NO. MA5281)

Dear Mr. Wanczyk:

The Commission has issued the enclosed Amendment No. 172 to Facility Operating License DPR-28 for the Vermont Yankee Nuclear Power Station, in response to your application dated April 16, 1999, as supplemented June 9, 1999.

The amendment clarifies the inservice inspection requirements for the Vermont Yankee Nuclear Power Station regarding the granting of relief from ASME Code requirements by the NRC. The amendment also makes changes to reflect the previous NRC approval of the use of ASME Code Case N-560 at the Vermont Yankee Nuclear Power Station. In a separate matter, the NRC is assessing the licensee's classification of certain Category A welds at Vermont Yankee.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Richard P. Croteau, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosures: 1. Amendment No.172 to
License No. DPR-28
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink, appearing to read "R. Croteau".

Richard P. Croteau, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosures: 1. Amendment No. 172 to
License No. DPR-28
2. Safety Evaluation

cc w/encls: See next page

Vermont Yankee Nuclear Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 172
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Vermont Yankee Nuclear Power Corporation (the licensee) dated April 16, 1999, as supplemented by letter dated June 9, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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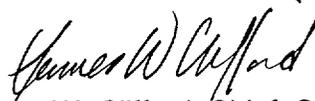
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-28 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 172 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 13, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 172

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Insert

120

120

3.6 LIMITING CONDITIONS FOR OPERATION

D. Safety and Relief Valves

1. During reactor power operating conditions and whenever the reactor coolant pressure is greater than 120 psig and temperature greater than 350°F, both safety valves and at least three of the four relief valves shall be operable.
2. If Specification 3.6.D.1 is not met, initiate an orderly shutdown and the reactor coolant pressure shall be below 120 psig and 350°F within 24 hours.

E. Structural Integrity and Operability Testing

The structural integrity and the operability of the safety-related systems and components shall be maintained at the level required by the original acceptance standards throughout the life of the plant.

4.6 SURVEILLANCE REQUIREMENTS

D. Safety and Relief Valves

1. Operability testing of Safety and Relief Valves shall be in accordance with Specification 4.6.E. The lift point of the safety and relief valves shall be set as specified in Specification 2.2.B.

E. Structural Integrity and Operability Testing

1. Inservice inspection of safety-related components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC. Inservice inspection of piping, identified in NRC Generic Letter 88-01, shall be performed in accordance with the staff positions on schedule, methods, and personnel and sample expansion included in the Generic Letter, except that sample selection for the scope of Category A welds may be in accordance with ASME Code Case N-560.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 172 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letter dated April 16, 1999, as supplemented by letter dated June 9, 1999, the Vermont Yankee Nuclear Power Corporation (the licensee) submitted a request to amend the Vermont Yankee Nuclear Power Station (Vermont Yankee) Technical Specifications (TSs). The proposed amendment would revise the TSs to clarify the inservice inspection requirements for Vermont Yankee regarding the granting of relief from American Society of Mechanical Engineers (ASME) Code requirements by the NRC. The licensee also proposed changes to reflect the previous NRC approval of the use of ASME Code Case N-560 at Vermont Yankee.

2.0 EVALUATION

The licensee proposed to change TS 4.6.E.1 as follows:

TS 4.6.E.1 states, in part, that "Inservice inspection of piping, identified in GL 88-01, shall be performed in accordance with the staff positions on schedule, methods, and personnel and sample expansion included in the Generic Letter." The licensee proposes to add a caveat to the end of that sentence, stating, "except that sample selection for the scope of Category A welds may be in accordance with ASME Code Case N-560."

TS 4.6.E.1 also states that inservice inspection of safety-related components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda ... except where written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i). The licensee proposed deleting "pursuant to 10 CFR 50, Section 50.55a(g)(6)(i)" from this statement.

By letter dated November 9, 1998, the licensee received NRC approval to use ASME Code Case N-560 in association with inservice inspection of Class 1, Category B-J, piping welds under ASME Section XI. The licensee stated that all of Vermont Yankee's ASME Category B-J piping welds are also Category A piping welds as defined in GL 88-01. Code Case N-560 reduces the 25 percent inspection sample required by both ASME Section XI and GL 88-01 to 10 percent, while stipulating selection of that sample in accordance with a risk-informed analytical methodology. The TSs specifically reference the staff's positions on schedule, methods, personnel, and sample expansion in GL 88-01. Approval of Code Case N-560 affects the sampling requirements of GL 88-01 by allowing alternative sample selection criteria for

Class 1 Category B-J (GL 88-01 Category A) piping welds. Therefore, a TS change is necessary to reflect the November 9, 1998, NRC approval of Code Case N-560 for Vermont Yankee. Since the staff has previously authorized the use of Code Case N-560 for Vermont Yankee (see NRC safety evaluation dated November 9, 1998), the proposed change to administratively reflect the effect of this approval in the Vermont Yankee TS associated with NRC Generic Letter 88-01 is acceptable to the staff.

With respect to the other change, the licensee stated that TS 4.6.E.1 allows relief from ASME Section XI requirements to be granted by the NRC only under 50.55a(g)(6)(i) and that it is inappropriate to reference only one paragraph from 10 CFR 50.55a when other paragraphs may be applicable to the granting of relief. In accordance with 10 CFR 50.55a, relief requests may also be "granted" pursuant to paragraph (f)(6)(i). Alternatives to code requirements may be "authorized" pursuant to paragraphs (a)(3)(i) and (a)(3)(ii). The staff finds the proposed change to be acceptable because the various paragraphs of 10 CFR 50.55a adequately describe the process for obtaining relief from the ASME Section XI requirements and a general reference in the TSs to granting written relief by the NRC is adequate.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Vermont State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in amounts, and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (64 FR 38037). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Croteau

Date: August 13, 1999