

April 24, 1989

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger  
Vice President and Chief Nuclear  
Officer  
Public Service Electric & Gas Company  
Post Office Box 236  
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: CONTAINMENT ISOLATION VALVES (TAC NOS. 60673/60674)

RE: SALEM GENERATING STATION, UNIT NOS. 1 AND 2

The Commission has issued the enclosed Amendment Nos. 92 and 67 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated November 27, 1985, as modified in your February 15, 1989 submittal.

These amendments would modify the containment isolation valve table, Technical Specification Table 3.6-1.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

James C. Stone, Project Manager  
Project Directorate I-2  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 92 to License No. DPR-70
- 2. Amendment No. 67 to License No. DPR-75
- 3. Safety Evaluation

*DFOI*  
*1/1*

cc w/enclosures:  
See next page

DISTRIBUTION w/enclosures:

Docket File	MO'Brien (2)	Wanda Jones	Brent Clayton
NRC PDR	OGC	EButcher	EWenzinger
Local PDR	DHagan	DFischer	
PDI-2 Reading	EJordan	ACRS (10)	
WButler	BGrimes	CMiles, GPA/PA	
JStone/MThadani	TMeek (8)	RDiggs, ARM/LFMB	

\*Previously concurred

PDI-2/LA*	PDI-2/PM*	OGC*	PDI-2/D*	SPLB*	[SALEM 1/2]
MO'Brien	JStone:mr	EReis	WButler	JCraig	
04/19/89	04/19/89	04/20/89	04/20/89	04/20/89	

8905020428 890424  
PDR ADOCK 05000272  
P PDC

*CP-1*  
*cc*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 24, 1989

Docket Nos. 50-272/311

Mr. Steven E. Miltenberger  
Vice President and Chief Nuclear  
Officer  
Public Service Electric & Gas Company  
Post Office Box 236  
Hancocks Bridge, New Jersey 08038

Dear Mr. Miltenberger:

SUBJECT: CONTAINMENT ISOLATION VALVES (TAC NOS. 60673/60674)

RE: SALEM GENERATING STATION, UNIT NOS. 1 AND 2

The Commission has issued the enclosed Amendment Nos. 92 and 67 to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your applications dated November 27, 1985, as modified in your February 15, 1989 submittal.

These amendments would modify the containment isolation valve table, Technical Specification Table 3.6-1.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Project Manager  
Project Directorate I-2  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 92 to  
License No. DPR-70
2. Amendment No. 67 to  
License No. DPR-75
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. Steven E. Miltenberger  
Public Service Electric & Gas Company

Salem Nuclear Generating Station

cc:

Mark J. Wetterhahn, Esquire  
Conner and Wetterhahn  
Suite 1050  
1747 Pennsylvania Avenue, NW  
Washington, DC 20006

Richard B. McGlynn, Commission  
Department of Public Utilities  
State of New Jersey  
101 Commerce Street  
Newark, NJ 07102

Richard Fryling, Jr., Esquire  
Law Department - Tower 5E  
80 Park Place  
Newark, NJ 07101

Regional Administrator, Region I  
U. S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. L. K. Miller  
General Manager - Salem Operations  
Salem Generating Station  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Lower Alloways Creek Township  
c/o Mary O. Henderson, Clerk  
Municipal Building, P.O. Box 157  
Hancocks Bridge, NJ 08038

Mr. S. LaBruna  
Vice President - Nuclear Operations  
Nuclear Department  
P.O. Box 236  
Hancocks Bridge, New Jersey 08038

Mr. Bruce A. Preston, Manager  
Licensing and Regulation  
Nuclear Department  
P.O. Box 236  
Hancocks Bridge, NJ 08038

Robert Traee, Mayor  
Lower Alloways Creek Township  
Municipal Hall  
Hancocks Bridge, NJ 08038

Mr. David Wersan  
Assistant Consumer Advocate  
Office of Consumer Advocate  
1425 Strawberry Square  
Harrisburg, PA 17120

Kathy Halvey Gibson, Resident Inspector  
Salem Nuclear Generating Station  
U.S. Nuclear Regulatory Commission  
Drawer I  
Hancocks Bridge, NJ 08038

Scott B. Ungerer  
MGR. - Joint Generation Projects  
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1199 Black Horse Pike  
Pleasantville, NJ 08232

Richard F. Engel  
Deputy Attorney General  
Department of Law and Public Safety  
CN-112  
State House Annex  
Trenton, NJ 08625

Delmarva Power & Light Company  
c/o Jack Urban  
General Manager, Fuel Supply  
800 King Street  
P.O. Box 231  
Wilmington, DE 19899

Mr. David M. Scott, Chief  
Bureau of Nuclear Engineering  
Department of Environmental Protection  
State of New Jersey  
CN 411  
Trenton, NJ 08625



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92  
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated November 27, 1985, as modified on February 15, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

8905020437 890424  
PDR ADOCK 05000272  
P PDC

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1989

*Handwritten:* PDI-2/DA  
MONB:men  
4/19/89

*Handwritten:* Jem  
PDI-2/PM  
JStone:mr  
4/19/89

*Handwritten:* OGC  
as modified  
4/20/89

*Handwritten:* PDI-2/D  
WButler  
4/21/89

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 6-14	3/4 6-14
-	3/4 6-14a
3/4 6-15	3/4 6-15
3/4 6-16	3/4 6-16
3/4 6-17	3/4 6-17
3/4 6-17a	3/4 6-17a

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>		<u>FUNCTION</u>	<u>ISOLATION TIME (Second)</u>
<b>A. PHASE "A" ISOLATION</b>			
1.	1 PR 17*	Pressurizer Relief Tk.-Gas Analyzer Conn.	< 10 Sec.
2.	1 PR 18*	Pressurizer Relief Tk.-Gas Analyzer Conn.	< 10 Sec.
3.	1 NT 25*	Pressurizer Relief Tk.-N <sub>2</sub> Connection	< 10 Sec.
4.	1 MR 80*	Pressurizer Relief Tk.-Primary Water Conn.	< 10 Sec.
5.	1 CV 3	CVCS - Letdown Line	< 10 Sec.
6.	1 CV 4	CVCS - Letdown Line	< 10 Sec.
7.	1 CV 5	CVCS - Letdown Line	< 10 Sec.
8.	1 CV 7	CVCS - Letdown Line	< 10 Sec.
9.	1 CV 68**	CVCS - Charging Line	< 10 Sec.
10.	1 CV 69**	CVCS - Charging Line	< 10 Sec.
11.	1 CV 284	CVCS - RCP Seals	< 10 Sec.
12.	1 CV 116	CVCS - RCP Seals	< 10 Sec.
13.	1 CC 215	Comp. Cooling to Excess Letdown Hx	< 10 Sec.
14.	1 CC 113	Comp. Cooling to Excess Letdown Hx	< 10 Sec.
15.	1 WL 96*	RC Drain Tk. - Gas Analyzer Conn.	< 10 Sec.
16.	1 WL 97*	RC Drain Tk. - Gas Analyzer Conn.	< 10 Sec.
17.	1 WL 98*	RC Drain Tk. - Vent Header Conn.	< 10 Sec.
18.	1 WL 99*	RC Drain Tk. - Vent Header Conn.	< 10 Sec.
19.	1 WL 108*	RC Drain Tk. - N <sub>2</sub> Connection	< 10 Sec.
20.	1 WL 12*	RC Drain Tank Pumps	< 10 Sec.
21.	1 WL 13*	RC Drain Tank Pumps	< 10 Sec.
22.	1 NT 32*	Accumulator N <sub>2</sub> Connection	< 10 Sec.
23.	1 SJ 123*	SI Test Line	< 10 Sec.
24.	1 SJ 60*	SI Test Line	< 10 Sec.
25.	1 SJ 53*	SI Test Line	< 10 Sec.
26.	1 SS 103*	Accumulator Sampling	< 10 Sec.
27.	1 SS 27*	Accumulator Sampling	< 10 Sec.
28.	1 SS 104*	RC Sampling	< 10 Sec.
29.	1 SS 33*	RC Sampling	< 10 Sec.

TABLE 3.6-1 (Contd.)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>A. PHASE "A" ISOLATION (Contd.)</b>		
30. 1 SS 107*	Pressurizer Liquid Sampling	< 10 Sec.
31. 1 SS 49*	Pressurizer Liquid Sampling	< 10 Sec.
32. 1 SS 110*	Pressurizer Steam Sampling	< 10 Sec.
33. 1 SS 64*	Pressurizer Steam Sampling	< 10 Sec.
34. 1 VC 7	Containment Radiation Sampling	< 10 Sec.
35. 1 VC 8	Containment Radiation Sampling	< 10 Sec.
36. 1 VC 11	Containment Radiation Sampling	< 10 Sec.
37. 1 VC 12	Containment Radiation Sampling	< 10 Sec.
38. 11 CA 330	Instrument Air Supply	< 10 Sec.
39. 12 CA 330	Instrument Air Supply	< 10 Sec.
40. 1 DR 29	Demineralized Water Supply	< 10 Sec.
41. 1 ML 16	Containment Sump Discharge	< 10 Sec.
42. 1 ML 17	Containment Sump Discharge	< 10 Sec.
43. 1 FP 147*	Fire Protection System	< 10 Sec.
<b>B. PHASE "B" ISOLATION</b>		
1. 1 CC 118	Component Cooling to RCP	< 10 Sec.
2. 1 CC 187	Component Cooling to RCP	< 10 Sec.
3. 1 CC 136	Component Cooling to RCP	< 10 Sec.
4. 1 CC 190	Component Cooling to RCP	< 10 Sec.
5. 1 CC 131	Component Cooling to RCP	< 10 Sec.
<b>C. MAIN STEAM ISOLATION</b>		
1. 11 MS 7#	Main Steam Drain	< 10 Sec.
2. 12 MS 7#	Main Steam Drain	< 10 Sec.
3. 13 MS 7#	Main Steam Drain	< 10 Sec.
4. 14 MS 7#	Main Steam Drain	< 10 Sec.
5. 11 MS 18#	Main Steam Bypass	< 10 Sec.
6. 12 MS 18#	Main Steam Bypass	< 10 Sec.
7. 13 MS 18#	Main Steam Bypass	< 10 Sec.
8. 14 MS 18#	Main Steam Bypass	< 10 Sec.

TABLE 3.6-1 (Contd.)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>D. FEEDWATER ISOLATION</b>		
1. 11 BF 19#	Main Feedwater Isolation	< 8 Sec.
2. 12 BF 19#	Main Feedwater Isolation	< 8 Sec.
3. 13 BF 19#	Main Feedwater Isolation	< 8 Sec.
4. 14 BF 19#	Main Feedwater Isolation	< 8 Sec.
5. 11 BF 40#	Main Feedwater Isolation	< 8 Sec.
6. 12 BF 40#	Main Feedwater Isolation	< 8 Sec.
7. 13 BF 40#	Main Feedwater Isolation	< 8 Sec.
8. 14 BF 40#	Main Feedwater Isolation	< 8 Sec.
<b>E. STEAM GENERATOR BLOWDOWN ISOLATION</b>		
1. 11 GB 4#	Steam Generator Blowdown	< 10 Sec.
2. 12 GB 4#	Steam Generator Blowdown	< 10 Sec.
3. 13 GB 4#	Steam Generator Blowdown	< 10 Sec.
4. 14 GB 4#	Steam Generator Blowdown	< 10 Sec.
5. 11 SS 94#	SG Blowdown Sampling	< 10 Sec.
6. 12 SS 94#	SG Blowdown Sampling	< 10 Sec.
7. 13 SS 94#	SG Blowdown Sampling	< 10 Sec.
8. 14 SS 94#	SG Blowdown Sampling	< 10 Sec.
<b>F. CONTAINMENT PURGE AND PRESSURE-VACUUM RELIEF</b>		
1. 1 VC 1	Purge Supply	< 2 Sec.
2. 1 VC 2	Purge Supply	< 2 Sec.
3. 1 VC 3	Purge Exhaust	< 2 Sec.
4. 1 VC 4	Purge Exhaust	< 2 Sec.
5. 1 VC 5*	Pressure-Vacuum Relief	< 2 Sec.
6. 1 VC 6*	Pressure-Vacuum Relief	< 2 Sec.

TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>G. MANUAL</b>		
1. 1 SS 900#	Pressurizer Dead-Weight Calibrator	Not Applicable
2. 1 SS 901#	Pressurizer Dead-Weight Calibrator	Not Applicable
3. 11 CV 98#	CVCS - RCP Seals	Not Applicable
4. 12 CV 98#	CVCS - RCP Seals	Not Applicable
5. 13 CV 98#	CVCS - RCP Seals	Not Applicable
6. 14 CV 98#	CVCS - RCP Seals	Not Applicable
7. 1 SJ 71#	CVCS Flushing Connection	Not Applicable
8. 1 SA 118	Compressed Air Supply	Not Applicable
9. 1 ML 190	Refueling Canal Supply	Not Applicable
10. 1 SF 36	Refueling Canal Supply	Not Applicable
11. 1 ML 191	Refueling Canal Discharge	Not Applicable
12. 1 SF 22	Refueling Canal Discharge	Not Applicable
13. 1 VC 9*	Containment Radiation Sampling	Not Applicable
14. 1 VC 10*	Containment Radiation Sampling	Not Applicable
15. 1 VC 13*	Containment Radiation Sampling	Not Applicable
16. 1 VC 14*	Containment Radiation Sampling	Not Applicable
17. - #	Fuel Transfer Tube	Not Applicable
18. 1 CS 900	Containment Instrumentation Fill Conn.	Not Applicable
19. 1 CS 901	Containment Instrumentation Fill Conn.	Not Applicable
20. 1 CS 902	Containment Instrumentation Fill Conn.	Not Applicable
21. 1 CS 903	Containment Instrumentation Fill Conn.	Not Applicable
22. 1 SA 262	Containment Pressure Test Conn.	Not Applicable
23. 1 SA 264	Containment Pressure Test Conn.	Not Applicable
24. 1 SA 265	Containment Pressure Test Conn.	Not Applicable
25. 1 SA 267	Containment Pressure Test Conn.	Not Applicable
26. 1 SA 268	Containment Pressure Test Conn.	Not Applicable
27. 1 SA 270	Containment Pressure Test Conn.	Not Applicable
28. 1 CA 1714	Containment Air Lock Instrument Conn.	Not Applicable
29. 1 CA 1715	Containment Air Lock Instrument Conn.	Not Applicable

TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>G. MANUAL</b>		
30. 11 SS 181*	Post LOCA Sampling	Not Applicable
31. 11 SS 182*	Post LOCA Sampling	Not Applicable
32. 11 SS 188*	Post LOCA Sampling	Not Applicable
33. 11 SS 189*	Post LOCA Sampling	Not Applicable
34. 13 SS 181*	Post LOCA Sampling	Not Applicable
35. 13 SS 182*	Post LOCA Sampling	Not Applicable
36. 13 SS 184*	Post LOCA Sampling	Not Applicable
37. 13 SS 185*	Post LOCA Sampling	Not Applicable
38. 11 VC 17*	Post LOCA Sampling	Not Applicable
39. 11 VC 18*	Post LOCA Sampling	Not Applicable
40. 11 VC 19*	Post LOCA Sampling	Not Applicable
41. 11 SF 20*	Post LOCA Sampling	Not Applicable
42. 12 VC 17*	Post LOCA Sampling	Not Applicable
43. 12 VC 18*	Post LOCA Sampling	Not Applicable
44. 12 VC 19*	Post LOCA Sampling	Not Applicable
45. 12 VC 20*	Post LOCA Sampling	Not Applicable
46. 11 CS 2†	Containment Spray	Not Applicable
47. 12 CS 2†	Containment Spray	Not Applicable
<b>H. CHECK</b>		
1. 1 CV 74	CVCS-Charging Line	Not Applicable
2. 1 CV 296	CVCS-RCP Seals	Not Applicable
3. 1 CC 119	Component Cooling to RCP	Not Applicable
4. 1 CC 186	Component Cooling to RCP	Not Applicable
5. 1 CC 208	Component Cooling to RCP	Not Applicable
6. 11 CS 48	Containment Spray	Not Applicable
7. 12 CS 48	Containment Spray	Not Applicable
8. 1 DR 30	Deminerlized Water System	Not Applicable
9. 1 FP 148	Fire Protection System	Not Applicable
10. 1 NT 26	Pressurizer Relief Tk. -Nitrogen Conn.	Not Applicable
11. 1 NT 34	Accumulator Nitrogen Supply	Not Applicable
12. 1 SA 119	Compressed Air Supply	Not Applicable

TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>H. CHECK (Contd.)</b>		
13. 1 MR 81	Pressurizer Relief Tk.-Primary Water Conn.	Not Applicable
14. 11 CA 360	Instrument Air Supply	Not Applicable
15. 12 CA 360	Instrument Air	Not Applicable

<sup>4</sup>May be opened on an intermittent basis under administrative control.  
<sup>#</sup>Not subject to Type C leakage tests.  
<sup>#</sup>Either valve 1 CV 68 or 1 CV 69 must be OPERABLE.  
<sup>†</sup>Normally closed motor operated containment isolation valve. Valve opens on Phase B isolation



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67  
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Public Service Electric & Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated November 27, 1985, as modified on February 15, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective as of its date of issuance and to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1989

*[Handwritten signature]*  
PDI-2/D  
WButler  
4/19/89

*[Handwritten signature]*  
PDI-2/PM  
JStone:mr  
4/19/89

*[Handwritten signature]*  
OGC  
4/20/89

*[Handwritten signature]*  
PDI-2/D  
WButler  
4/21/89

*[Handwritten signature]*

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 6-16	3/4 6-16
-	3/4 6-16a
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 6-19	3/4 6-19
3/4 6-20	3/4 6-20

TABLE 3.6-1  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seco( )</u>
<b>A. PHASE "A" ISOLATION</b>		
1. 2 PR 17*	Pressurizer Relief Tk.-Gas Analyzer Conn.	< 10 Sec.
2. 2 PR 18*	Pressurizer Relief Tk.-Gas Analyzer Conn.	< 10 Sec.
3. 2 NT 25*	Pressurizer Relief Tk.-N <sub>2</sub> Connection	< 10 Sec.
4. 2 WR 80*	Pressurizer Relief Tk.-Primary Water Conn.	< 10 Sec.
5. 2 CV 3	CVCS - Letdown Line	< 10 Sec.
6. 2 CV 4	CVCS - Letdown Line	< 10 Sec.
7. 2 CV 5	CVCS - Letdown Line	< 10 Sec.
8. 2 CV 7	CVCS - Letdown Line	< 10 Sec.
9. 2 CV 68##	CVCS - Charging Line	< 10 Sec.
10. 2 CV 69##	CVCS - Charging Line	< 10 Sec.
11. 2 CV 284	CVCS - RCP Seals	< 10 Sec.
12. 2 CV 116	CVCS - RCP Seals	< 10 Sec.
13. 2 CC 215	Comp. Cooling to Excess Letdown Hx	< 10 Sec.
14. 2 CC 113	Comp. Cooling to Excess Letdown Hx	< 10 Sec.
15. 2 WL 96*	RC Drain Tk. - Gas Analyzer Conn.	< 10 Sec.
16. 2 WL 97*	RC Drain Tk. - Gas Analyzer Conn.	< 10 Sec.
17. 2 WL 98*	RC Drain Tk. - Vent Header Conn.	< 10 Sec.
18. 2 WL 99*	RC Drain Tk. - Vent Header Conn.	< 10 Sec.
19. 2 WL 108*	RC Drain Tk. - N <sub>2</sub> Connection	< 10 Sec.
20. 2 WL 12*	RC Drain Tank Pumps	< 10 Sec.
21. 2 WL 13*	RC Drain Tank Pumps	< 10 Sec.
22. 2 NT 32*	Accumulator N <sub>2</sub> Connection	< 10 Sec.
23. 2 SJ 123*	SI Test Line	< 10 Sec.
24. 2 SJ 60*	SI Test Line	< 10 Sec.
25. 2 SJ 53*	SI Test Line	< 10 Sec.
26. 2 SS 103*	Accumulator Sampling	< 10 Sec.
27. 2 SS 27*	Accumulator Sampling	< 10 Sec.
28. 2 SS 104*	RC Sampling	< 10 Sec.
29. 2 SS 33*	RC Sampling	< 10 Sec.

TABLE 3.6-1 (Contd.)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>A. PHASE "A" ISOLATION (Contd.)</b>		
30. 2 SS 107*	Pressurizer Liquid Sampling	< 10 Sec.
31. 2 SS 49*	Pressurizer Liquid Sampling	< 10 Sec.
32. 2 SS 110*	Pressurizer Steam Sampling	< 10 Sec.
33. 2 SS 64*	Pressurizer Steam Sampling	< 10 Sec.
34. 2 VC 7	Containment Radiation Sampling	< 10 Sec.
35. 2 VC 8	Containment Radiation Sampling	< 10 Sec.
36. 2 VC 11	Containment Radiation Sampling	< 10 Sec.
37. 2 VC 12	Containment Radiation Sampling	< 10 Sec.
38. 21 CA 330	Instrument Air Supply	< 10 Sec.
39. 22 CA 330	Instrument Air Supply	< 10 Sec.
40. 2 DR 29	Demineralized Water Supply	< 10 Sec.
41. 2 WL 16	Containment Sump Discharge	< 10 Sec.
42. 2 WL 17	Containment Sump Discharge	< 10 Sec.
43. 2 FP 147*	Fire Protection System	< 10 Sec.
<b>B. PHASE "B" ISOLATION</b>		
1. 2 CC 118	Component Cooling to RCP	< 10 Sec.
2. 2 CC 187	Component Cooling to RCP	< 10 Sec.
3. 2 CC 136	Component Cooling to RCP	< 10 Sec.
4. 2 CC 190	Component Cooling to RCP	< 10 Sec.
5. 2 CC 131	Component Cooling to RCP	< 10 Sec.
<b>C. MAIN STEAM ISOLATION</b>		
1. 21 MS 7#	Main Steam Drain	< 10 Sec.
2. 22 MS 7#	Main Steam Drain	< 10 Sec.
3. 23 MS 7#	Main Steam Drain	< 10 Sec.
4. 24 MS 7#	Main Steam Drain	< 10 Sec.
5. 21 MS 18#	Main Steam Bypass	< 10 Sec.
6. 22 MS 18#	Main Steam Bypass	< 10 Sec.
7. 23 MS 18#	Main Steam Bypass	< 10 Sec.
8. 24 MS 18#	Main Steam Bypass	< 10 Sec.

TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Second)</u>
<b>D. FEEDWATER ISOLATION</b>		
1. 21 BF 19#	Main Feedwater Isolation	< 5 Sec.
2. 22 BF 19#	Main Feedwater Isolation	< 5 Sec.
3. 23 BF 19#	Main Feedwater Isolation	< 5 Sec.
4. 24 BF 19#	Main Feedwater Isolation	< 5 Sec.
5. 21 BF 40#	Main Feedwater Isolation	< 5 Sec.
6. 22 BF 40#	Main Feedwater Isolation	< 5 Sec.
7. 23 BF 40#	Main Feedwater Isolation	< 5 Sec.
8. 24 BF 40#	Main Feedwater Isolation	< 5 Sec.
<b>E. STEAM GENERATOR BLOWDOWN ISOLATION</b>		
1. 21 GB 4#	Steam Generator Blowdown	< 10 Sec.
2. 22 GB 4#	Steam Generator Blowdown	< 10 Sec.
3. 23 GB 4#	Steam Generator Blowdown	< 10 Sec.
4. 24 GB 4#	Steam Generator Blowdown	< 10 Sec.
5. 21 SS 94#	SG Blowdown Sampling	< 10 Sec.
6. 22 SS 94#	SG Blowdown Sampling	< 10 Sec.
7. 23 SS 94#	SG Blowdown Sampling	< 10 Sec.
8. 24 SS 94#	SG Blowdown Sampling	< 10 Sec.
<b>F. CONTAINMENT PURGE AND PRESSURE-VACUUM RELIEF</b>		
1. 2 VC 1	Purge Supply	< 2 Sec.
2. 2 VC 2	Purge Supply	< 2 Sec.
3. 2 VC 3	Purge Exhaust	< 2 Sec.
4. 2 VC 4	Purge Exhaust	< 2 Sec.
5. 2 VC 5*	Pressure-Vacuum Relief	< 2 Sec.
6. 2 VC 6*	Pressure-Vacuum Relief	< 2 Sec.

TABLE 3.6-1 (Contd.)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>G. MANUAL</b>		
1. 2 SS 900#	Pressurizer Dead-Weight Calibrator	Not Applicable
2. 2 SS 901#	Pressurizer Dead-Weight Calibrator	Not Applicable
3. 21 CV 98#	CVCS - RCP Seals	Not Applicable
4. 22 CV 98#	CVCS - RCP Seals	Not Applicable
5. 23 CV 98#	CVCS - RCP Seals	Not Applicable
6. 24 CV 98#	CVCS - RCP Seals	Not Applicable
7. 2 SJ 71#	CVCS Flushing Connection	Not Applicable
8. 2 SA 118	Compressed Air Supply	Not Applicable
9. 2 ML 190	Refueling Canal Supply	Not Applicable
10. 2 SF 36	Refueling Canal Supply	Not Applicable
11. 2 ML 191	Refueling Canal Discharge	Not Applicable
12. 2 SF 22	Refueling Canal Discharge	Not Applicable
13. 2 VC 9*	Containment Radiation Sampling	Not Applicable
14. 2 VC 10*	Containment Radiation Sampling	Not Applicable
15. 2 VC 13*	Containment Radiation Sampling	Not Applicable
16. 2 VC 14*	Containment Radiation Sampling	Not Applicable
17. - #	Fuel Transfer Tube	Not Applicable
18. 2 CS 900	Containment Instrumentation Fill Conn.	Not Applicable
19. 2 CS 901	Containment Instrumentation Fill Conn.	Not Applicable
20. 2 CS 902	Containment Instrumentation Fill Conn.	Not Applicable
21. 2 CS 903	Containment Instrumentation Fill Conn.	Not Applicable
22. 2 SA 262	Containment Pressure Test Conn.	Not Applicable
23. 2 SA 264	Containment Pressure Test Conn.	Not Applicable
24. 2 SA 265	Containment Pressure Test Conn.	Not Applicable
25. 2 SA 267	Containment Pressure Test Conn.	Not Applicable
26. 2 SA 268	Containment Pressure Test Conn.	Not Applicable
27. 2 SA 270	Containment Pressure Test Conn.	Not Applicable
28. 2 CA 1714	Containment Air Lock Instrument Conn.	Not Applicable
29. 2 CA 1715	Containment Air Lock Instrument Conn.	Not Applicable

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TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>G. MANUAL</b>		
30. 21 SS 181*	Post LOCA Sampling	Not Applicable
31. 21 SS 182*	Post LOCA Sampling	Not Applicable
32. 21 SS 188*	Post LOCA Sampling	Not Applicable
33. 21 SS 189*	Post LOCA Sampling	Not Applicable
34. 23 SS 181*	Post LOCA Sampling	Not Applicable
35. 23 SS 182*	Post LOCA Sampling	Not Applicable
36. 23 SS 184*	Post LOCA Sampling	Not Applicable
37. 23 SS 185*	Post LOCA Sampling	Not Applicable
38. 21 VC 17*	Post LOCA Sampling	Not Applicable
39. 21 VC 18*	Post LOCA Sampling	Not Applicable
40. 21 VC 19*	Post LOCA Sampling	Not Applicable
41. 21 SF 20*	Post LOCA Sampling	Not Applicable
42. 22 VC 17*	Post LOCA Sampling	Not Applicable
43. 22 VC 18*	Post LOCA Sampling	Not Applicable
44. 22 VC 19*	Post LOCA Sampling	Not Applicable
45. 22 VC 20*	Post LOCA Sampling	Not Applicable
46. 21 CS 2†	Post LOCA Sampling	Not Applicable
47. 22 CS 2†	Containment Spray	Not Applicable
	Containment Spray	Not Applicable
<b>H. CHECK</b>		
1. 2 CV 74	CVCS-Charging Line	Not Applicable
2. 2 CV 296	CVCS-RCP Seals	Not Applicable
3. 2 CC 119	Component Cooling to RCP	Not Applicable
4. 2 CC 186	Component Cooling to RCP	Not Applicable
5. 2 CC 208	Component Cooling to RCP	Not Applicable
6. 21 CS 48	Containment Spray	Not Applicable
7. 22 CS 48	Containment Spray	Not Applicable
8. 2 DR 30	Demineralized Water System	Not Applicable
9. 2 FP 148	Fire Protection System	Not Applicable
10. 2 NT 26	Pressurizer Relief Tk. -Nitrogen Conn.	Not Applicable
11. 2 NT 34	Accumulator Nitrogen Supply	Not Applicable
12. 2 SA 119	Compressed Air Supply	Not Applicable

TABLE 3.6-1 (Contd.)  
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (Seconds)</u>
<b>H. CHECK (Contd.)</b>		
13. 2 MR 81	Pressurizer Relief Tk.-Primary Water Conn.	Not Applicable
14. 21 CA 360	Instrument Air Supply	Not Applicable
15. 22 CA 360	Instrument Air	Not Applicable

\*May be opened on an intermittent basis under administrative control.

#Not subject to Type C leakage tests.

##Either valve 1 CV 68 or 1 CV 69 must be OPERABLE.

†Normally closed motor operated containment isolation valve. Valve opens on Phase B isolation



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 92 AND 67 TO FACILITY OPERATING

LICENSE NOS. DPR-70 AND DPR-75

PUBLIC SERVICE ELECTRIC & GAS COMPANY

PHILADELPHIA ELECTRIC COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

SALEM GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated November 27, 1985, Public Service Electric & Gas Company (PSE&G) requested an amendment to Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Generating Station, Unit Nos. 1 and 2. The proposed amendments would amend Technical Specification 3.6.3, Containment Isolation Valves, for Salem Units 1 and 2. The proposed amendment sought a modification of the wording in the Limiting Condition for Operation (LCO) as well as changes in the containment isolation valve table (Tech. Spec. Table 3.6-1).

After an initial staff review, the staff concluded that the LCO wording change was unacceptable and that the containment isolation valve table modification was acceptable with one exception. PSE&G had included two containment outboard series isolation valves in the table (CC117 and CC118, Component Cooling to the Reactor Coolant Pumps) with the intention of using either one or the other of these outboard valves as the isolation valve for a particular containment penetration. The staff stated that only the valve which is closest to the containment (CC118) should be included in Tech. Spec. Table 3.6-1.

In order to gain approval of this license change request, PSE&G on February 15, 1989 resubmitted the requested change removing valve CC117 from Tech Spec Table 3.6-1 for both Salem Units 1 and 2, and withdrawing their proposed change of the wording in the LCO.

## 2.0 EVALUATION

The original November 27, 1985 request contained the following proposed changes to TS 3.6.3 and associated ACTION statements.

- (a) Description of change: The change would allow an inoperable isolation valve to satisfy its OPERABILITY requirements if the valve is maintained closed to isolate the affected penetration.

Staff evaluation: The proposed TS change would consider an inoperable containment isolation valve to be OPERABLE, i.e., the limiting condition for operation to be satisfied, if the valve is administratively maintained closed. Consequently, fluid lines could be isolated, and administratively maintained closed, without ever having to declare that a limiting condition for operation was not being met or that an ACTION statement has to be entered. Although, isolating a fluid line and administratively maintaining that condition is an acceptable approach for meeting the current LCO, having to enter an ACTION statement is preferable from an operational safety standpoint. Operations personnel would be more aware of potential safety concerns by having to take remedial action, and documenting it, to reestablish compliance with a LCO. Consequently, the staff found the proposed changes to TS 3.6.3 unacceptable. PSE&G, in their February 15, 1989 resubmittal, withdrew this proposed change.

The original November 27, 1985 request contained the following proposed changes to Technical Specifications (TS) Table 3.6-1, containment Isolation Valves.

- (a) Description of change: Adding valves (as listed in the submittal) to TS Table 3.6-1 because they are considered to be containment isolation valves and subject to Type C (local leakage rate) testing.

Staff evaluation: These valves were inadvertently omitted from Table 3.6-1 and the Type C testing program. Adding these valves to the Type C testing program is acceptable.

- (b) Description of change: Changing the classification of certain valves (as listed in the submittal) such that they are required to be Type C tested, rather than being excepted from Type C testing.

Staff evaluation: Adding valves to the Type C testing program is acceptable.

- (c) Description of change: Designating certain valves (as listed in the submittal) in the steam generator blowdown and steam generator blowdown sampling lines as not being subject to Type C testing.

Staff evaluation: These valves are connected to the secondary side of the steam generator, which constitutes a closed system inside containment which is not postulated to rupture during a LOCA. Thus, the subject valves will not be exposed to containment atmosphere during a LOCA and do not constitute potential containment atmosphere leak paths. Although Appendix J to 10 CFR Part 50 requires main steam system (e.g., main steam and feedwater) isolation valves in boiling water reactors to be Type C tested (Section II.H.4 of Appendix J), it does not require such valves in pressurized water reactors, as the Salem facility, to be Type C tested. Therefore, it is acceptable to delete the subject valves from the Type C testing program.

- (d) Description of change: Deleting valves 11(21) SS93, 12(22) SS93, 13(23) SS93, and 14(24) SS93 from TS Table 3.6-1.

Staff evaluation: The staff determined that these valves are not required to be containment isolation valves and may be deleted from the table. Since they are not containment isolation valves, they are not subject to Type C testing.

Further, the staff found that valve CC117 is not a containment isolation valve. As a further clarification to that SER, the staff finds that the valve CC117 may be deleted from TS Table 3.6-1 and removed from the Type C testing program. The valve was deleted from table in the February 15, 1989, resubmittal. Therefore, the staff finds the proposed changes as described above to be acceptable.

In addition, administrative clarifications and typographical corrections were made to the licensee's incoming technical specifications pages and bars were added to those pages to show the changes.

### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (51 FR 24261) on July 2, 1986 and consulted with the State of New Jersey. No public comments were received and the State of New Jersey did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Donald Fischer

Dated: April 24, 1989