



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 29, 1987

Docket Nos.: 50-272

Mr. Corbin A. McNeill, Jr.  
Senior Vice President - Nuclear  
Public Service Electric and Gas  
Company  
Post Office Box 236  
Hancocks Bridge, New Jersey 08038

Dear Mr. McNeill:

SUBJECT: EXTENSION OF INTERIM RELIEF FOR  
SALEM NUCLEAR GENERATING STATION UNIT NO. 1 INSERVICE TESTING  
PROGRAM FOR PUMPS AND VALVES - SECOND 10 YEAR INTERVAL

REFERENCE: Letter W. R. Butler to C. A. McNeil, Jr. dated June 24, 1987

By letter dated September 24, 1986, you submitted a proposed revision to your inservice testing program description for pumps and valves and requests for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Based upon the results of a review by the staff and a subsequent site meeting held with your representatives on February 26, 1987, you submitted a revised second 10 year interval inservice testing program description for pumps and valves and requests for relief. This revision, dated March 30, 1987, superseded the previous submittal and includes certain changes resulting from the February 26, 1987 meeting. The purpose of this letter is to extend the interim relief granted in the referenced letter until the staff concludes its detailed review of your March 30, 1987 IST program submittal.

The IST program for pumps and valves is required by 10 CFR 50.55a(g) to comply with the ASME Boiler and Pressure Vessel Code, Section XI. For Salem Unit No. 1, the applicable version of the ASME Section XI Code is the 1983 Edition through the Summer 1983 Addenda. This code version will supersede your currently referenced program version as is required under 50.55a for pumps and valves. Pursuant to CFR 50.55a(g)(5), relief was requested from certain ASME Code testing requirements for specific pumps and valves where the Code requirements are impractical within the limits of design, geometry and system safety. The requests for relief include an explanation and justification for the relief requested and a proposal for alternative testing where practical.

Based on our preliminary review, we agree with the determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level

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of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i), we hereby grant the interim relief until the staff has completed its detailed review of those inservice testing requirements of the ASME Code that you have requested. The granting of this relief is based only on the impracticality of selected ASME Code requirements. The relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins; thus, the granting of this relief does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program (except where your current Technical Specifications are more restrictive).

During the period between now and the date we complete our detailed review of your submittal, you must comply with both your existing Technical Specifications and your proposed inservice testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter testing intervals, increased number of parameters measured). In other words, the granting of this interim relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications, other than the superseding of the outdated version of the code with the above applicable version of the code.

When our detailed review of your submittal is complete we will: (1) request a meeting with you to discuss the results of our review; (2) issue final approval of your program which may contain modifications and/or additional imposed requirements resulting from the staff's review; and (3) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval.

Sincerely,



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

cc: See next page

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Sincerely,

/s/

Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

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Mr. C. A. McNeill  
Public Service Electric & Gas Company

Salem Nuclear Generating Station

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