

Docket File

REGULATORY DOCKET FILE COPY

DECEMBER 9 1980

Docket No. 50-272

Mr. F. P. Librizzi, General Manager  
Electric Production  
Production Department  
Public Service Electric and Gas Company  
80 Park Plaza, 15A  
Newark, New Jersey 07101

US NRC  
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Dear Mr. Librizzi:

The Commission has issued the enclosed Amendment No. 28 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your requests dated March 17, August 22, and September 19, 1980.

The amendment revises the Technical Specifications related to (a) minimum voltages for individual battery cells; (b) the water level maintained in the reactor cavity during movement of spent fuel; and (c) administrative actions concerning radiation protection.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:  
S. A. Varga

Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 28 to DPR-70
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page

8101100551

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DATE						

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Docket File 50-272

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*DSI: PSB  
F. ROSA F.R.  
11/21/80 RAZ*

*concur as  
to form*

OFFICE ▶	DL:ORB1	DL:ORB1	DL:ORB1	DL:AD:OR	OEND	DL:STSG
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 9, 1980

Docket No. 50-272

Mr. F. P. Librizzi, General Manager  
Electric Production  
Production Department  
Public Service Electric and Gas Company  
80 Park Plaza, 15A  
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3. Notice of Issuance

cc: w/enclosures  
See next page

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Mr. F. P. Librizzi  
Public Service Electric and Gas Company - 2 -

December 9, 1980

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
PHILADELPHIA ELECTRIC COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28  
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated March 17, August 22, and September 19, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 28, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 9, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 28

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 8-9  
3/4 8-12  
3/4 9-10  
6-3  
6-5  
6-15  
6-16

Insert Pages

3/4 8-9  
3/4 8-12  
3/4 9-10  
6-3  
6-5  
6-15  
6-16

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is  $\geq 1.200$ .
  3. The pilot cell voltage is  $\geq 2.08$  volts.
  4. The overall battery voltage is  $\geq 125$  volts.
- b. At least once per 92 days by verifying that:
1. The voltage of each connected cell is  $\geq 2.13$  volts under float charge and has not decreased more than 0.27 volts from the value observed during the original acceptance test.
  2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is  $\geq 1.200$  and has not decreased more than 0.02 from the value observed during the previous test.
  3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration.
  2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material.
  3. The battery charger will supply at least 200 amperes at 125 volts for at least 4 hours.
- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 8 hours when the battery is subjected to a battery service test.
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.

## ELECTRICAL POWER SYSTEMS

### 125-VOLT D.C. DISTRIBUTION - SHUTDOWN

#### LIMITING CONDITION FOR OPERATION

---

3.8.2.4 As a minimum, the following D. C. electrical equipment and bus shall be energized and OPERABLE:

2 - 125-volt D.C. busses, and

2 - 125-volt batteries, each with at least one full capacity charger, associated with each of the above D.C. busses.

APPLICABILITY: MODES 5 and 6.

#### ACTION:

With less than the above complement of D.C. equipment and busses OPERABLE, establish CONTAINMENT INTEGRITY within 8 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.8.2.4.1 The above required 125-volt D.C. busses shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.4.2 The above required 125-volt batteries and chargers shall be demonstrated OPERABLE per Surveillance Requirement 4.8.2.3.2.

## ELECTRICAL POWER SYSTEMS

### 28-VOLT D.C. DISTRIBUTION - OPERATING

#### LIMITING CONDITION FOR OPERATION

---

3.8.2.5 The following D.C. bus trains shall be energized and OPERABLE:

TRAIN 1A consisting of 28-volt D.C. bus No. 1A, 28-volt D.C. battery No. 1A and at least one full capacity charger.

TRAIN 1B consisting of 28-volt D.C. bus No. 1B, 28-volt D.C. battery No. 1B, and at least one full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

- a. With one 28-volt D.C. bus inoperable, restore the inoperable bus to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one 28-volt D.C. battery and/or its charger inoperable, restore the inoperable battery and/or charger to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.8.2.5.1 Each D.C. bus train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and power availability.

4.8.2.5.2 Each 28-volt battery and above required charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of the pilot cell is between the minimum and maximum level indication marks.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

2. The pilot cell specific gravity, corrected to 77°F, and full electrolyte level, is  $\geq 1.200$ .
  3. The pilot cell voltage is  $\geq 2.08$  volts.
  4. The overall battery voltage is  $\geq 27$  volts.
- b. At least once per 92 days by verifying that:
1. The voltage of each connected cell is  $\geq 2.13$  volts under float charge and has not decreased more than 0.27 volts from the value observed during the original acceptance test.
  2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is  $\geq 1.200$  and has not decreased more than 0.02 from the value observed during the previous test.
  3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration.
  2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material.
  3. The battery charger will supply at least 150 amperes at 28 volts for at least 4 hours.
- d. At least once per 18 months, during shutdown, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 8 hours when the battery is subjected to a battery service test.
- e. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.

REFUELING OPERATIONS

CONTAINMENT PURGE AND PRESSURE-VACUUM RELIEF ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

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3.9.9 The Containment Purge and Pressure-Vacuum Relief isolation system shall be OPERABLE.

APPLICABILITY: MODE 6.

ACTION:

With the Containment Purge and Pressure-Vacuum Relief isolation system inoperable, close each of the Purge and Pressure-Vacuum Relief penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provision of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

---

4.9.9 The Containment Purge and Pressure-Vacuum Relief isolation system shall be demonstrated OPERABLE within 100 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment Purge and Pressure-Vacuum Relief isolation occurs on manual initiation and on a high radiation test signal from each of the containment radiation monitoring instrumentation channels.

## REFUELING OPERATIONS

### WATER LEVEL - REACTOR VESSEL

#### LIMITING CONDITION FOR OPERATION

---

3.9.10 At least, 23 feet of water shall be maintained over the top of the reactor pressure vessel flange.

APPLICABILITY: During movement of fuel assemblies or control rods within the reactor pressure vessel while in MODE 6.

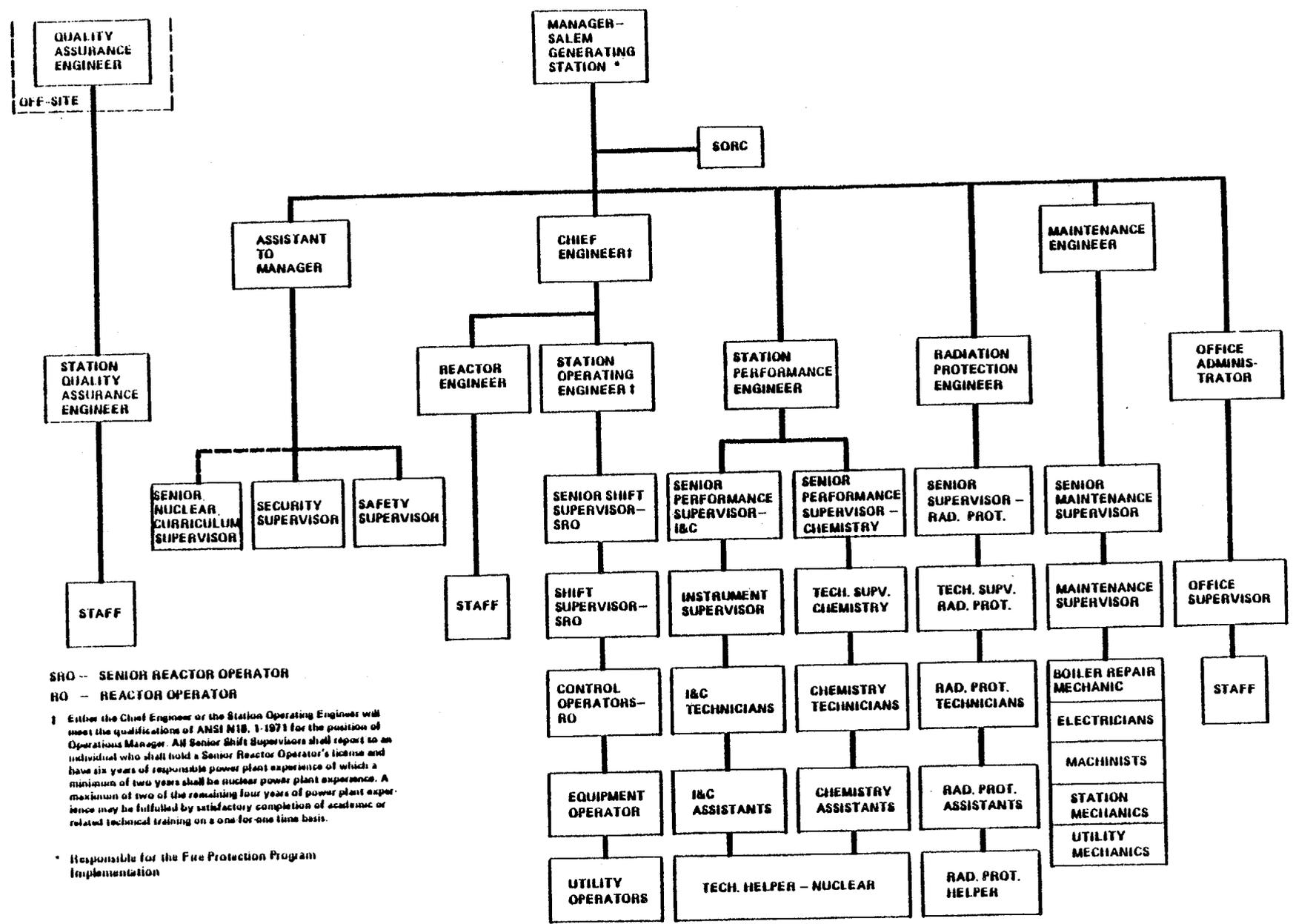
#### ACTION:

With the requirements of the above specification not satisfied, suspend all operations involving movement of fuel assemblies or control rods within the pressure vessel.. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.9.10 The water level shall be determined to be at least its minimum required depth within 2 hours prior to the start of and at least once per 24 hours thereafter during movements of fuel assemblies or control rods.



SRO -- SENIOR REACTOR OPERATOR  
 RO -- REACTOR OPERATOR

† Either the Chief Engineer or the Station Operating Engineer will meet the qualifications of ANSI N18.1-1971 for the position of Operations Manager. All Senior Shift Supervisors shall report to an individual who shall hold a Senior Reactor Operator's license and have six years of responsible power plant experience of which a minimum of two years shall be nuclear power plant experience. A maximum of two of the remaining four years of power plant experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis.

\* Responsible for the Fire Protection Program Implementation

FIGURE 6.2.2 FACILITY ORGANIZATION - SALEM GENERATING STATION

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	2	1

\*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS after the initial fuel loading.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by the Assistant to Manager and under the direction of the Training Engineer and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Safety Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Member:	Station Operating Engineer
Member:	Station Performance Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior Performance Supervisor - I&C
Member:	Senior Performance Supervisor - Chemistry
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor
Member:	Maintenance Engineer

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

## ADMINISTRATIVE CONTROLS

### MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SORC shall consist of the Chairman or his designated alternate and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Electric Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board.

## ADMINISTRATIVE CONTROLS

- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the SORC and the NRB.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Exposure Permit\*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

\*Radiation Protection personnel or personnel escorted by Radiation Protection personnel shall be exempt from the REP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

## ADMINISTRATIVE CONTROLS

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- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Senior Supervisor - Radiation Protection in the Radiation Exposure Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Senior Shift Supervisor on duty and/or the Senior Supervisor - Radiation Protection.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. DPR-70

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,  
PHILADELPHIA ELECTRIC COMPANY,  
DELMARVA POWER AND LIGHT COMPANY, AND  
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

DOCKET NO. 50-272

Introduction

By letters dated March 17, August 22, and September 19, 1980, Public Service Electric and Gas Company (the licensee) proposed that the Technical Specifications of Salem Unit No. 1 be revised. Although the applications and the revisions relate to different subjects, none have been considered to be complex or difficult to review. Consequently, we are including three safety evaluations in this amendment so as to reduce expenditure of resources.

A. Minimum Cell Voltages for 125V and 28V Batteries

By letter of March 17, 1980, the licensee requested amendment to Facility Operating License No. DPR-70 to permit a lowering of the individual cell minimum voltages on the 125 volt and 28 volt batteries from 2.17 volts to 2.13 volts. In support of the proposed Technical Specification changes, the licensee submitted a letter from C&D, Batteries Division dated July 16, 1980.

We have reviewed the information provided by the licensee concerning the lowering of the individual cell minimum voltages on the 125V and 28V batteries from 2.17 volts to 2.13 volts. To support these Technical Specification changes, the licensee submitted a letter from the battery vendor, C&D Batteries Division, which states that a cell float voltage of 2.13 is considered the minimum acceptable operating value and should this limit be exceeded, corrective action should be initiated. The licensee states that the corrective action will be the application of an equalizing charge to the cell or battery. This action is in accordance with IEEE Standard 450-1975, "IEEE Recommended Practice for Maintenance, Testing and Replacement of Large Lead Storage Batteries for Generating Stations and Substations." Based upon the C&D letter and the licensee's compliance, as noted, we find that sufficient bases and justification have been provided to grant the requested changes.

Based on our review of the above submittals, we conclude that the proposed Technical Specification changes to 125 volt and 28 volt battery individual cell minimum voltages are acceptable.

#### B. Water Level During Refueling

By letter of August 15, 1980 the staff advised the licensee of changes that had been made in Westinghouse Standard Technical Specifications 3.9.8.2, 3.9.10 and B 3/4 9.8. These changes require at least 23 feet of water over the top of the reactor pressure vessel flange during movement of fuel assemblies or control rods. This requirement assures that fuel assemblies can be transferred out of the reactor pressure vessel with sufficient water coverage to prevent exposure of fuel handlers.

The current Technical Specifications for Salem I require that the water depth be, as a minimum, 23 feet above the top of fuel assemblies rather than above the pressure vessel flange. Consequently, the licensee was required to make the necessary review and modifications to assure that exposure of fuel assemblies cannot occur. In a response of September 19, 1980, the licensee chose to substitute the wording and requirements of the revised Technical Specification 3.9.10 for the current version. This action is acceptable because the water level will be sufficiently high but is still within the design criteria of the reactor cavity.

The licensee has chosen not to accept the wording of the Technical Specification 3.9.8.2 at this time. Instead, these changes will be incorporated with others relating to residual heat removal capabilities when the plant is in various modes of operation, including refueling. This decision is acceptable because the limiting conditions of operation of T.S. 3.9.10 have not been affected. Also, the additional depth of water is not needed when the plant is in Mode 6 (refueling) other than when fuel assemblies are being moved.

#### C. Radiation Protection - Administrative Actions

By letter of August 22, 1980 the licensee proposed changes to the Salem Radiation Protection organization that provide for the separation of the radiation protection function from the Performance Department and formation of a new Radiation Protection Department. This new department will be headed by a Radiation Protection Engineer who will report directly to the station manager. It will have a Senior Supervisor - Radiation Protection (who will act as backup to the Radiation Protection Engineer), Technical Supervisors, Technicians and Technical Assistants, all of whom will be devoted to the function of radiation protection. The remainder of the Performance Department will be modified to split the Technical Assistants such that they are devoted to either the instrumentation and controls function or the chemistry function.

These proposed changes meet our positions in the draft "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 as follows:

1. The Radiation Protection Engineer (RPE - equivalent to the Radiation Protection Manager) reports directly to the Station Manager, independent of operational, technical or administrative groups. The RPE is a required member of the Station Operations Review Committee (SORC). Staff qualifications require that the RPE meet or exceed the recommendations of Regulatory Guide 1.8.

2. The newly formed Radiation Protection Department has an independent radiation protection function at all levels, and is separate from such functions as chemistry. A backup to the RPE, the Senior Supervisor-Radiation Protection has been designated. All Technical Supervisors, Technicians and Technical Assistants within the department are devoted to the radiation protection function.

3. A formal program to replace contractor radiation protection personnel with permanently assigned station radiation protection technicians has been implemented. Additionally, a qualification and retraining program conducted in accordance with ANSI 18.1, provides for qualification and training for the radiation protection department personnel. PSE&G anticipates the reorganization actions and programs to be fully complete by July 1, 1981. In the interim, a permanent staff is being recruited and all contractor radiation protection technicians are receiving class room and on the job training on systems, radiological fundamentals and procedures.

These actions and commitments by PSE&G for the Salem Station adequately meet the positions of NUREGs-0660/0694, NUREG-DRAFT "Criteria for Utility Management and Technical Competence" and Regulatory Guide 8.8 regarding Radiation Protection Organization and are therefore satisfactory. An evaluation of the Salem Radiation Protection Department will be performed during a routine inspection.

In addition, the licensee proposed to revise Technical Specification 6.12 so that the administrative control of people in high radiation areas would be the same for both Units 1 and 2. The new requirements increase the level of protection by requiring each individual or group of individuals to be provided with an integrating radiation monitoring device and to be accompanied by an individual who is qualified in radiation protection procedures. These changes upgrade the level of radiological protection and are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 9, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-272  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY,  
PHILADELPHIA ELECTRIC COMPANY,  
DELMARVA POWER AND LIGHT COMPANY, AND  
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 28 to Facility Operating License No. DPR-70, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 1 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications related to (a) minimum voltages for individual battery cells; (b) the water level maintained in the reactor cavity during movement of spent fuel; and (c) administrative actions concerning radiation protection.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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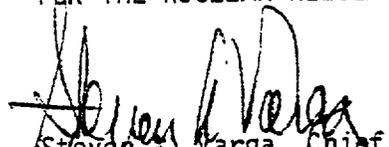
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated March 17, August 22, and September 19, 1980, (2) Amendment No. 28 to License No. DPR-70, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of December, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing