

REGULATORY BUCKET FILE COPY

January 6, 1981

Docket No. 50-272

Mr. F. P. Librizzi, General Manager
Electric Production
Production Department
Public Service Electric and Gas Company
80 Park Plaza, 15A
Newark, New Jersey 07101

Distribution

Docket File 50-272	S. Varga
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NSIC	B. Jones (4)
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D. Eisenhut	C. Miles
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	J. Wetmore

Dear Mr. Librizzi:

The Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your request dated November 20, 1980.

The amendment revises the Appendix A Technical Specifications for this unit to reflect changes in the organization that operates and supports the operation of both Unit Nos. 1 and 2.

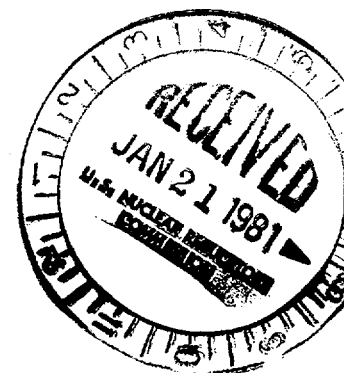
We have also taken this opportunity to revise the Administrative Control Section of Appendix A to update the requirements for the composition of the crew shifts. These changes make these requirements for Unit 1 identical to those approved for Unit 2 and did not require further review by the staff. Consistent "Administrative Controls" were also made to the Appendix B Technical Specifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing



Enclosures:

1. Amendment No. 31 to DPR-70
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures See next page

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OFFICE	DL:ORB1	DL:ORB1	DL:ORB1	DL:APROR	OELD	DL:SSS
SURNAME	WJRoss:jb	CParrish	SAVarga	TNovak	JMoose	JWetmore
DATE	12/15/80	12/16/80	12/16/80	12/20/80	12/15/81	12/20/80



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
January 6, 1981

Docket No. 50-272

Mr. F. P. Librizzi, General Manager
Electric Production
Production Department
Public Service Electric and Gas Company
80 Park Plaza, 15A
Newark, New Jersey 07101

Dear Mr. Librizzi:

The Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your request dated November 20, 1980.

The amendment revises the Appendix A Technical Specifications for this unit to reflect changes in the organization that operates and supports the operation of both Unit Nos. 1 and 2.

We have also taken this opportunity to revise the Administrative Control Section of Appendix A to update the requirements for the composition of the crew shifts. These changes make these requirements for Unit 1 identical to those approved for Unit 2 and did not require further review by the staff. Consistent "Administrative Controls" were also made to the Appendix B Technical Specifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in black ink that reads "Steven A. Varga".

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 31 to DPR-70
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

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Mr. F. P. Librizzi
Public Service Electric and Gas Company

cc: Mark J. Wetterhahn, Esquire
Conner, Moore and Corber
Suite 1050
1747 Pennsylvania Avenue, NW
Washington, D. C. 20006

Richard Fryling, Jr., Esquire
Assistant General Solicitor
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Gene Fisher, Bureau of Chief
Bureau of Radiation Protection
380 Scotch Road
Trenton, New Jersey 08628

Mr. Hank Midura, Manager
Salem Nuclear Generating Station
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Salem Free Library
112 West Broadway
Salem, New Jersey 08079

Leif J. Norrholm, Resident Inspector
Salem Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, New Jersey 08038

Attorney General
Department of Law and Public Safety
State House Annex
Trenton, New Jersey 08625

Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancocks Bridge, New Jersey 08038

Richard B. McGlynn, Commissioner
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

Deputy Attorney General
State House Annex
State of New Jersey
36 West State Street
Trenton, New Jersey 08625

Director, Criteria and Standards
Division
Office of Radiation Programs (ANW-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

Mr. E. N. Schwalje, Manager
of Quality Assurance
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Mr. R. L. Mittl, General Manager
Licensing and Environment
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Peter A. Moeller
Nuclear Licensing Engineer
Public Service Electric and Gas Company
80 Park Plaza - 15A
Newark, New Jersey 07101



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated November 20, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 31

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

6-2
6-3
6-4

6-5
6-6
6-7
6-8
6-11
6-12

Insert Pages

6-2
6-3
6-4
6-4a
6-5
6-6
6-7
6-8
6-11
6-12

Revise Appendix B as follows:

Remove Pages

5.1-1
5.2-1
5.2-2

Insert Pages

5.1-1
5.2-1
5.2-2

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

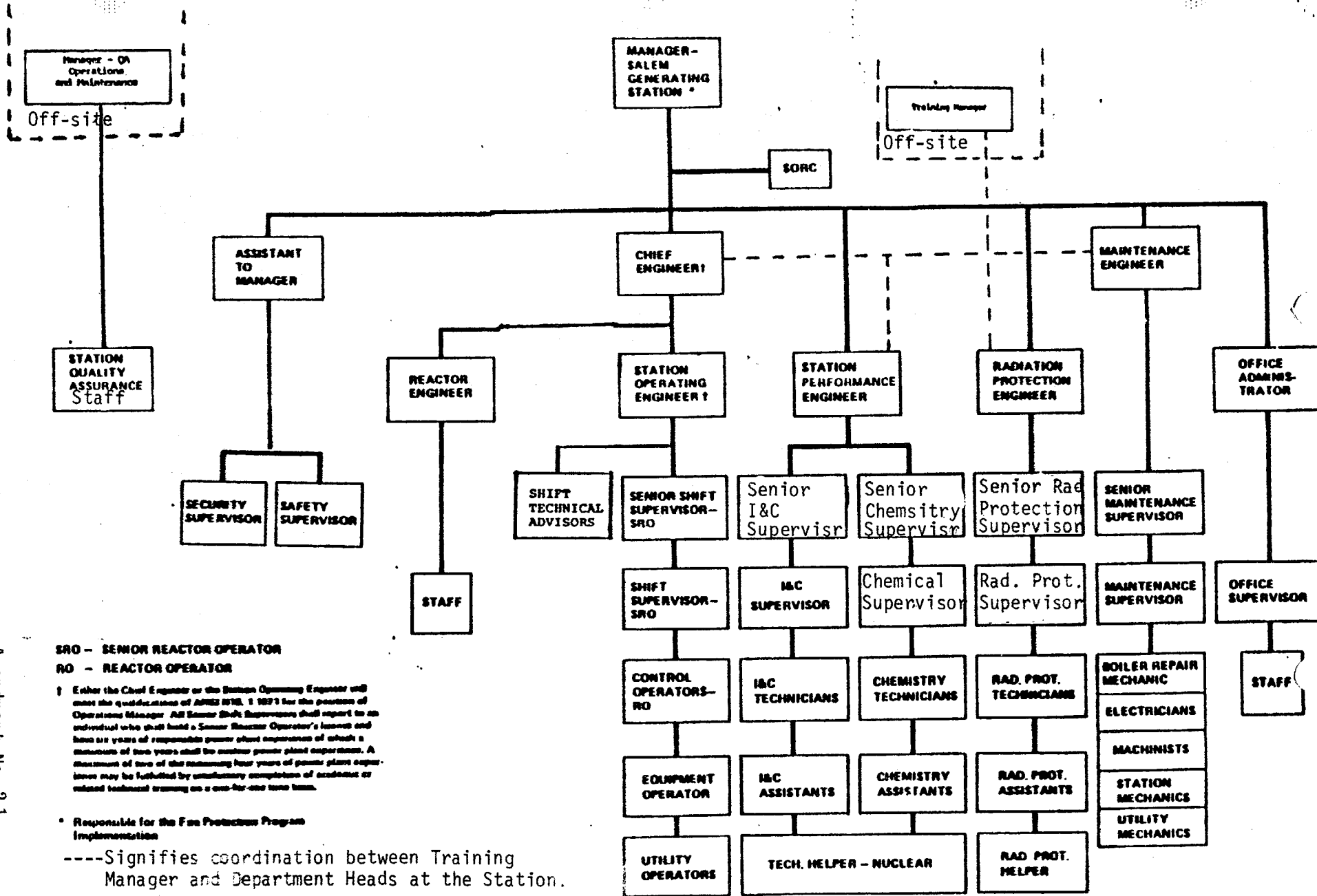
OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least 3 members shall be maintained onsite at all times. The Fire Brigade shall not include 4 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.



SRO - SENIOR REACTOR OPERATOR
 RO - REACTOR OPERATOR

1. Either the Chief Engineer or the Station Operating Engineer will meet the qualifications of APRI 1016, 1 1073 for the position of Operations Manager. All Senior Shift Supervisors shall report to an individual who shall hold a Senior Reactor Operator's license and have six years of responsible power plant experience of which a minimum of two years shall be senior power plant experience. A minimum of two of the remaining four years of power plant experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one basis.

* Responsible for the Fire Protection Program Implementation

----Signifies coordination between Training Manager and Department Heads at the Station.

FIGURE 6.2.2 FACILITY ORGANIZATION - SALEM GENERATING STATION

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

SALEM UNIT 1

WITH UNIT 2 IN MODES 5 OR 6 OR DE-FUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SS	1 ^a	1 ^a
SRO	1	none
RO	2	1
AO	2	2 ^b
STA	1	none

WITH UNIT 2 IN MODES 1, 2, 3 OR 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3 & 4	MODES 5 & 6
SS	1 ^a	1 ^a
SRO	1	none
RO	2	1
AO	2	1
STA	1 ^a	none

a/ Individual may fill the same position on Unit 2

b/ One of the two required individuals may fill the same position on Unit 2

TABLE 6.2-1 (Continued)

SS	-	Shift Supervisor with a Senior Reactor Operators License on Unit 1
SRO	-	Individual with a Senior Reactor Operators License on Unit 1
RO	-	Individual with a Reactor Operators License on Unit 1
AO	-	Auxiliary Operator
STA	-	Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2.1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Licensed operators shall not work more than:*

1. 12 hours straight,
2. 24 hours in any 48 hour period,
3. 72 hours in any 7-day period,
4. 14 consecutive days without having 2 consecutive days off.

*

Deviation from these requirements may be authorized by the station manager in accordance with established procedures and with documentation of the cause. Overtime limits do not include shift turnover time.

ADMINISTRATIVE CONTROLS

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Senior Performance Supervisor - Chemistry/HP who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (department head) at the facility and under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Safety Supervisor and shall meet or exceed the requirements of Section 27 of NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Member:	Station Operating Engineer
Vice Chairman:	Station Performance Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior I&C Supervisor
Member:	Senior Chemistry Supervisor
Member:	Radiological Protection Engineer
Member:	Senior Maintenance Supervisor
Vice Chairman:	Maintenance Engineer

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SORC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board.

ADMINISTRATIVE CONTROLS

- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.

AUTHORITY

6.5.1.7 The Station Operations Review Committee shall:

- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the General Manager- Nuclear Production and the Nuclear Review Board of disagreement between the SORC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Station Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the General Manager- Nuclear Production and Chairman of the Nuclear Review Board.

6.5.2 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.2.1 The Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering

ADMINISTRATIVE CONTROLS

- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The NRB shall be composed of the:

Chairman:	Assistant Vice President - Production
Vice Chairman:	Assistant to General Manager-Fuel Supply
Member:	General Manager-Licensing and Environment
Member:	Manager-Nuclear Operations Support
Member:	General Manager - Corporate QA
Member:	General Manager - Nuclear Production
Member:	Manager-Salem Generating Station
Member:	Principal Engineer
Member:	Manager - Hope Creek Generating Station

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.2.5 The NRB shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President-Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President-Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President-Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SORC and submitted to the NRB and the General Manager-Nuclear Production.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the General Manager- Nuclear Production and to the NRB within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the General Manager- Nuclear Production within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the SORC and approved by the Station Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

5.0 ADMINISTRATIVE CONTROLS

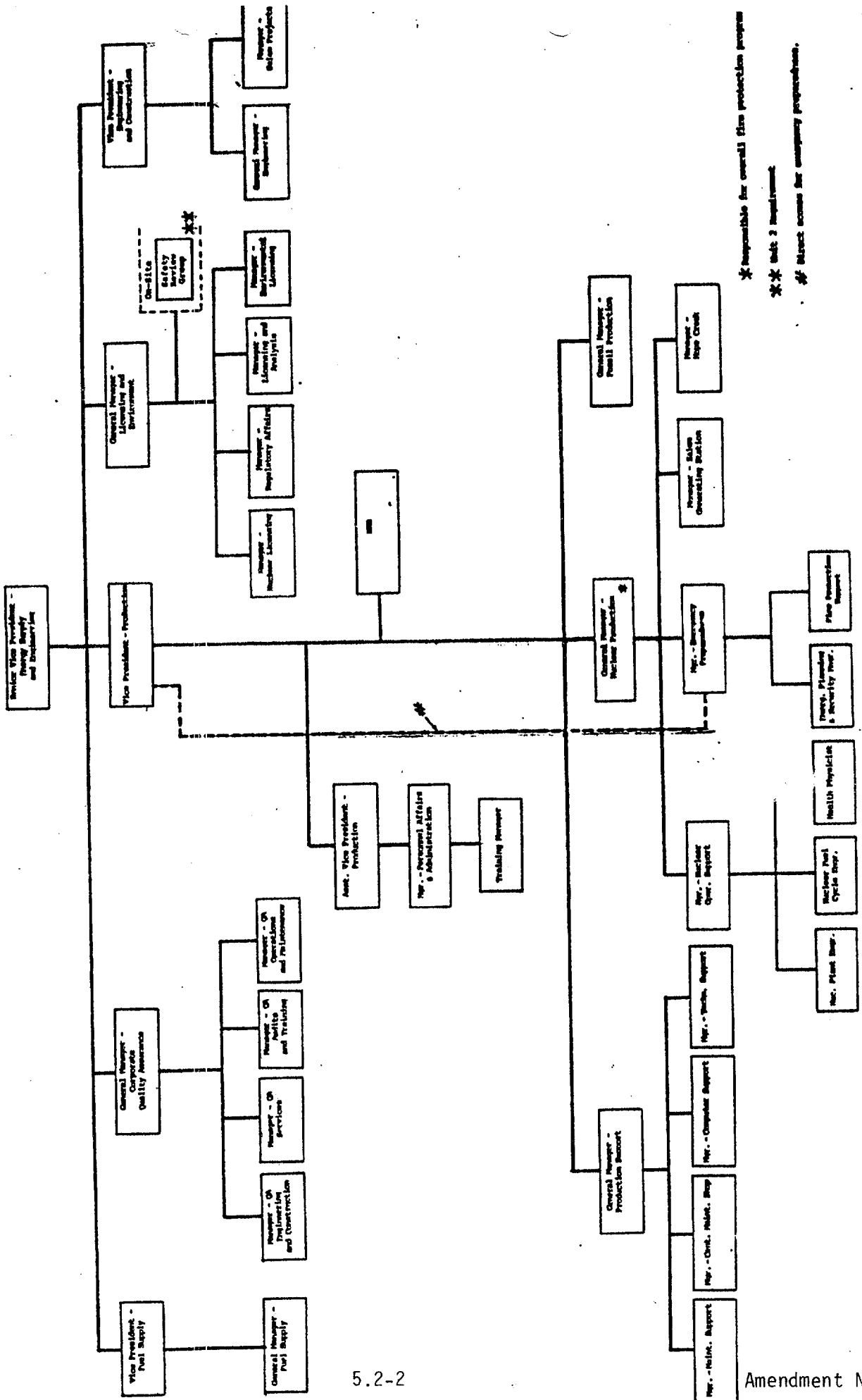
5.1 RESPONSIBILITY

5.1.1 The implementation of the surveillance programs external to the plant, including sampling, sample analysis, evaluation of results and the preparation of required reports is the responsibility of the Licensing and Environment Department. This Department is responsible for the assignment of personnel to the above functions, for assurance that appropriate written procedures, as described in Section 5.5.1, are utilized in the surveillance program activities and for assuring the quality of surveillance program results, as described in Section 5.5.3.

5.1.2 The Station Manager or his delegated alternate is responsible for operating the plant in compliance with the limiting conditions for operation as specified in the Environmental Technical Specifications and for the in-plant monitoring necessary to ensure such operation. His responsibility includes assurance that plant activities are conducted in such a manner as to provide continuing protection to the environment and that personnel performing such activities use appropriate written procedures as described in Section 5.5.

5.2 ORGANIZATION

- 5.2.1 Figure 6.2-1 of the Technical Specifications (Appendix A) identifies the corporate relationship between the Licensing and Environment Department and the Station Manager and also shows the organization of the Licensing and Environment Department. Figure 6.2.2 of the Technical Specifications (Appendix A) identifies the Production Department Station Organization.
- 5.2.2 The Nuclear Review Board (NRB) and Station Operations Review Committee (SORC) are shown in Figure 6.2-1 and 6.2-2 of the Appendix A Technical Specifications. They are advisory groups to the Vice President - Production and the Station Manager respectively.



5.2-2

* Responsible for overall site production program
 ** Unit 2 responsible
 # Direct access for emergency preparation.

FIGURE 6.2-1 OFFSITE ORGANIZATION FOR FACILITY MANAGEMENT AND TECHNICAL SUPPORT



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-70

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

DOCKET NO. 50-272

Introduction

By letter dated November 20, 1980, Public Service Electric and Gas Company (the licensee) submitted a request for amendment to License No. DPR-70 which would reflect changes in the organization that operates and supports the operation of both Unit Nos. 1 and 2. We have also revised the "Administrative Controls" Section of Appendix B.

Discussion

As a result of a proposed management reorganization for the Salem Generating Station, the licensee has requested changes to Section 6.0, "Administrative Controls", of Appendix A (Safety) of the Technical Specifications. These proposed changes also affect Section 5.0, "Administrative Controls" of Appendix B (Environmental) of the Technical Specifications.

This proposal results, to a great degree, from the increased scope of responsibilities of the licensee's management since Salem Unit No. 2 was licensed.

Under the organization in effect at the time Facility Operating License DPR-75 was issued for Salem Unit 2, all of the Company's operating plants reported directly to the General Manager - Electric Production who, in turn, reported to the Vice President - Production, and who was simultaneously responsible for both fossil and nuclear operations. We commented in our Safety Evaluation Report Supplement that this organizational arrangement could tend to dilute the attention given by corporate management to nuclear operations, but we concluded that the organization was workable due to the heavy emphasis placed on nuclear operations.

The licensee now proposes to divide the duties previously exercised by the General Manager - Electric Production, by establishing a General Manager - Nuclear Production, a General Manager - Fossil Production, and a General Manager - Production Support. The Company's nuclear plants will now report to the General Manager - Nuclear Production who will also direct the activities of a newly designated Manager - Emergency Preparedness and a Manager - Nuclear Operations Support, previously designated

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the Manager - Nuclear Operations. The new General Manager - Production Support will control most of the corporate level technical resources supporting the Company's operating plants. This proposed new organizational arrangement is shown on the attached Figure 6.2-1.

Evaluation

Our evaluation indicates that this proposed new organizational arrangement is superior to the previous arrangement since the new General Manager - Nuclear Production can devote his entire attention to the nuclear plant operation. Further, the Vice President - Production can now look to the General Manager - Production Support for most of the technical resources needed to support safe plant operation. Also, the revised organizational structure provides a Manager - Emergency Preparedness who can devote his attention to the emergency planning, security and fire protection functions at the nuclear plants. As shown in the revised Figure 6.2-1, the Manager - Emergency Preparedness will have direct access to the Vice President - Production on emergency preparedness matters.

The proposed revised organization also provides an Assistant Vice President - Production, who supervises the Manager - Personnel Affairs and Administration, and through him the corporate Training Manager. The Assistant Vice President - Production also will serve as the Chairman of the Nuclear Review Board.

Under the organization in being at the time Salem Unit 2 was licensed, the Manager - Quality Assurance reported to the Vice President - Engineering and Construction. In the new proposed organization, a General Manager - Corporate Quality Assurance has been created, on a reporting level comparable to the Vice President - Engineering and Construction and the Vice President - Production. This is shown on the attached Figure 6.2-1. This change assures the continued separation of the quality assurance function from the production function and, in fact, should result in greater corporate attention being given to quality assurance. We conclude that it is acceptable.

Figure 6.2-2, attached, depicts the proposed new station organization. There are a number of significant changes that the licensee proposes to make. In addition to a number of title changes, the Radiation Protection Engineer has now been moved such that he is free from plant operational pressures and has direct access to the Station Manager. This is in accord with the desires of the NRR Radiation Assessment Branch and is acceptable. The Senior Training Supervisor at the Station has been replaced by a corporate level Training Manager, who coordinates training activities directly with the Department Heads at the Station. This should result in increased emphasis on training and is acceptable.

At the time the license for Unit 2 was issued, the licensee had the Shift Technical Advisors reporting to the Reactor Engineer. By letter dated October 29, 1980, the licensee informed us that, in order to obtain a greater involvement of the STAs in shift operation and an improved awareness of plant conditions, the STAs had been shifted so that they would report to the Operating Engineer rather than the Reactor Engineer. This change is reflected on the attached Figure 6.2-2. This reporting arrangement maintains STA independence from the actual shift operating crew and is acceptable.

Figure 6.2-2 also shows the Station Quality Assurance Staff now reporting to the Manager - QA Operations and Maintenance at the Corporate level who, in turn, reports to the new General Manager - Corporate Quality Assurance, as shown on Figure 6.2-1. This maintains the independence of Station QA activities from operating pressures and is acceptable.

Summary

As a result of the organizational and title changes as noted above, the text of Section 6.0 of the Safety Technical Specifications and Section 5.0 of the Environmental Technical Specifications has been altered. We have reviewed these text changes and find them to be in accordance with the proposed new organization.

Overall, our review indicates that the proposed organizational changes covered by the requested Technical Specification amendment should result in strengthened management controls of the nuclear plant operations, as compared to the organization that was in effect at the time the operating license for Salem Unit 2 was issued. On this basis, we conclude that the proposed management reorganization is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 6, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-272

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 31 to Facility Operating License No. DPR-70, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Appendices A and B Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 1 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment revises the Technical Specifications for this unit to reflect changes in the organization that operates and supports the operation of both Unit Nos. 1 and 2. Consistent "Administrative Controls" were also made to the Appendix B Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.


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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 20, 1980, (2) Amendment No. 31 to License No. DPR-70, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 6th day of January 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing