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October 31, 1983

Docket No. 50-272

Mr. Richard A. Uderitz, Vice President
 Nuclear
 Public Service Electric and Gas Company
 Post Office Box 236
 Hancocks Bridge, New Jersey 08038

Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No. **54** to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your request dated July 22, 1983.

The amendment, on a one-time basis, extends the 40-~~1~~ 10 month interval of Technical Specification 4.6.1.2a during the first 10 year service period to permit the second inservice integrated leak rate test to be performed during the fifth refueling outage by adding a footnote to the specification which reads: "The second inservice Integrated Leak Rate Test shall be performed at the fifth refueling outage, but no later than May, 1984."

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Donald Fischer, Project Manager
 Operating Reactors Branch No. 1
 Division of Licensing

Enclosures:

1. Amendment No. **54** to DPR-70
2. Safety Evaluation

cc w/enclosures:
 See next page

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****See previous concurrence

OFFICE	ORB# ***	ORB#1 ***	ORB#1 ****	AD/OR:DL ***	OELD ***		
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DATE	11/ /83	11/ /83	11/ /83	11/ /83	11/ /83		

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Sincerely,

Donald Fischer, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Don't issue until after 10/21

Immediately before release check for Petition or comments to Secy and Upstream
Union: If any made come to ELD

Package de with changes discussed with SV: 10/21/83

- Enclosures
- 1. Amendment No. to DPR-70
- 2. Safety Evaluation

cc w/enclosures:
See next page

Hoppy Hallgren
Jew Shapiro for WB, CSB 10/21/83

OFFICE	ORB#L:DL CParrish	ORB#L:DL DFischer;ef	C-ORB#1:DL Svarga	AD/OP GLaffas	OELD Jm	D:DL DEisenhut
SURNAME						
DATE	10/19/83	10/20/83	10/20/83	10/20/83	10/21/83	10/21/83

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated July 22, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 6-2

Insert Pages

3/4 6-2

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of $\leq L_a$, 0.10 percent by weight of the containment air per 24 hours at design pressure, (47.0 psig).
- b. A combined leakage rate of $< 0.60 L_a$ for all penetrations and valves subject to Type B and C tests^a as identified in Table 3.6-1, when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at design pressure (47.0 psig) during each 10-year service period.* The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

* The second inservice Integrated Leak Rate Test shall be performed at the fifth refueling outage, but no later than May, 1984.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-70

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY.

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

DOCKET NO. 50-272

INTRODUCTION

In February of 1983, Salem Unit 1 experienced an extended outage that was required for the investigation and the development of a corrective action program relating to failure of the unit to automatically trip on two demands. This, in addition to other unforeseen delays in the restart of Salem Unit 1 following its fourth refueling outage, would require a four-to-five-week plant shutdown for the sole purpose of performing the Type A test originally scheduled in October 1983, after only about five months of full power operation. In view of the potential shutdown and associated cost, the Public Service Electric and Gas Company (PSE&G) submitted a request (1) for a one-time extension of the 40 ± 10 month interval between periodic Type A tests, and proposed that the required test be performed at the next (fifth) refueling outage, which will commence on May 31, 1984. On August 19, 1983, PSE&G submitted another letter(2) transmitting additional information in support of their request. The staff has reviewed these submittals and the review results are provided below.

EVALUATION

Appendix J to 10 CFR 50 requires that a set of three periodic Type A tests be performed at approximately equal intervals during each 10 year service period. The purpose of the Type A test is to provide confirmation of the overall containment integrity after a period of plant service. It also provides a measure of the effectiveness of the more frequently performed Local Leak Rate Test program. Therefore, minor alterations of the interval between two Type A tests would be considered acceptable if containment integrity has been demonstrated by previous Type A tests and verified by an effective Local Leak Rate Test Program.

We have reviewed the bases presented in submittals (1) and (2) above, to justify the one-time relief from the Technical Specifications (TS) limit of 40 ± 10 months for a periodic Type A test interval, and find the following bases acceptable for granting the one-time extension of the Type A test interval requested by PSE&G:

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- o Acceptable integrated leakage tests have been performed for both the pre-operational Type A test and the first Type A retest. The results of these tests indicate that containment integrity has been maintained within the TS limits. The first Type A retest resulted in a leakage rate (at the 95% upper confidence level) of 0.62 La; the acceptance criterion for a Type A Test is that the measured leakage rate be less than 0.75 La.
- o A complete Local Leak Rate Test Program was completed on all penetrations and valves during the most recent refueling outage. The combined test results were well within the TS limits of 0.6 La; the measured leakage rate for the local leakage test program was 0.069 La.
- o An increased surveillance program for Local Leak Rate tests will be implemented until the next refueling outage. Surveillance will be increased for those containment isolation valves that can be tested during plant operation and which have exhibited high or moderately high leakage during previous local leak rate testing. This includes valves 1VC-1, 1VC-2, 1VC-3, 1VC-4, 1VC-7, 1VC-8, 1VC-9 and 1VC-10. Valves which are not testable during plant operation will be tested when the earliest opportunity arises, e.g., unscheduled cold shutdowns with affected portions of the system drained. This includes valves 12 CS-2, 11 CA-330, 12 CA-330, 1CC-138, 1CC-215, 1CV-71, 1FP-147 and 1FP-148. Surveillance will be increased by reducing the scheduled intervals from once every 24 months for valves 1VC-7 through -10, and from once every 6 months for valves 1VC-1 through -4, to once every three months.
- o PSE&G has reviewed its records as far back as start of commercial operation and found that no repairs have been made to the concrete containment or the containment liner. Further, since start of commercial operation, they have had no indications of any kinds of problems that would, in some manner, affect a Type A Test.

From the above bases, we find that the Salem Unit 1 containment integrity has been maintained through the existing programs and that the increased surveillance program for local leak rate tests should further reduce the likelihood of a loss of containment integrity until the next refueling outage. We find that the request by PSE&G for a one-time extension of the Type A test interval to the next refueling outage on May 31, 1984 is acceptable.

SUMMARY

The containment structure is a passive mitigating feature and its primary function is to impede the release of fission-products to the environs in the event of a LOCA inside the containment. The Containment, therefore, does not influence the initiation or progression of any accident. Consequently, the proposed extension of the Type A test interval would neither increase the probability of an accident previously evaluated; nor create the possibility of a new or different kind of accident from any accident previously evaluated.

The purpose of the Type A test is to provide periodic confirmation of over all containment leakage integrity and verify the effectiveness of the Local Leak Rate Testing Program. Since containment leakage would most likely occur

through containment penetrations and isolation valves, an effective Local Leak Rate Testing Program, which is conducted at a greater frequency than the Type A test, will provide continued assurance of containment integrity. Therefore, the proposed extension of the Type A test interval, supplemented with increased surveillance of certain containment isolation valves, would not reduce the margin of safety provided by the Salem containment, and it is not likely that the consequence to the public would be increased.

On these bases, the staff concludes that the amendment does not involve a significant hazards consideration, and therefore, is acceptable.

A petition to intervene has been received from the State of Delaware relative to this amendment request. We have considered the comments contained in the petition and have concluded that the comments do not affect our determination that this amendment does not involve a significant hazards consideration.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 31, 1983

Principal Contributor:
J. Huang