April 8, 1986

Docket Nos. 50-272 and 50-311

Mr. C. A. McNeill, Jr. Vice President - Nuclear Public Service Electric and Gas Company Post Office Box 236 Hancocks Bridge, New Jersey 08038

Dear Mr. McNeill:

و المسلح الم

By letter dated March 7, 1986, we issued Amendment No. 72 to Facility Operating License No. DPR-70 and Amendment No. 46 to Facility Operating License No. 75 for Salem Nuclear Generating Station Units 1 and 2. The amendments permitted unit operation with only one service water header operable during modes 5 and 6 during a refueling outage.

An existing phrase on one page of the revised technical specifications was inadvertently omitted when the revised page was processed. The reference, identified by the symbol #, located on page 3/4 4-3b of the Unit 1 technical specifications and on page 3/4 4-4a of the Unit 2 technical specifications, did not include the phrase "may be substituted for one residual heat removal pump". The corrected pages are enclosed. Please change your technical specifications to reflect this modification.

/s/DFischer

Donald C. Fischer, Senior Project Manager PWR Project Directorate No. 3 Division of PWR Licensing-A

DISTRIBUTION

Docket File

PAD-3 Rdg

B. Grimes

D. Fischer

V. Benarova

T. Barnhart 8

ACRS 10

L PDR

OELD

L FMB

NRC PDR

Gray File H. Thompson

E. Jordan

C. Vogan

L. Harmon

W. Jones

OPA

J. Partlow

Enclosure: As stated

8604230071 860408

ADOCK

PDR

05000272

PNR

cc w/enclosure: See next page

		_ /	 		
OFC :PAD#3	:PAD#3	DIPAD#3	•		
OFC :PAD#3 NAME :CVogan V	:DFischer;ps	SVarga	 ·	• • •	•
DATE :4/ % /86	:4/8/86	:4/8/86	 :	•	•

OFFICIAL RECORD COPY

Mr. C. A. McNeill Public Service Electric & Gas Company

cc: Mark J. Wetterhahn, Esquire Conner and Wetterhahn Suite 1050 1747 Pennsylvania Avenue, NW Washington, DC 20006

Richard Fryling, Jr., Esquire Assistant General Solicitor Public Service Electric & Gas Company P. O. Box 570 - Mail Code T5E Newark, New Jersey 07101

Gene Fisher, Bureau of Chief Bureau of Radiation Protection 380 Scotch Road Trenton, New Jersey 08628

Mr. John M. Zupko, Jr. General Manager - Salem Operations Public Service Electric & Gas Company Post Office Box E Hancocks Bridge, New Jersey 08038

Robert Traae, Mayor Lower Alloways Creek Township Municipal Hall Hancocks Bridge, New Jersey 08038

Thomas Kenny, Resident Inspector Salem Nuclear Generating Station U.S. Nuclear Regulatory Commission Drawer I Hancocks Bridge, New Jersey 08038

Richard F. Engel Deputy Attorney General Department of Law and Public Safety CN-112 State House Annex Trenton, New Jersey 08625

i

Frank Casolito, Action Chief Bureau of Radiation Protection Department of Environmental Protection 380 Scotch Road Trenton, New Jersey 08628

Salem Nuclear Generating Station

Richard B. McGlynn, Commission Department of Public Utilities State of New Jersey 101 Commerce Street Newark, New Jersey 07102 ...

Mr. David Wersan Assistant Consumer Advocate Office of Consumer Advocate 1425 Strawberry Square Harrisburg, Pennsylvania 17120

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

Lower Alloways Creek Township c/o Mary O. Henderson, Clerk Municipal Building, P.O. Box 157 Hancocks Bridge, New Jersey 08038

Mr. Bruce A. Preston, Manager Nuclear Licensing & Regulation Public Service Electric & Gas Company Hancocks Bridge, New Jersey 08038

REACTOR COOLANT SYSTEM

COLD SHUT DOWN

LIMITING CONDITION FOR OPERATION

3.4.1.4 Two# residual heat removal loops shall be OPERABLE* and at least one RHR loop shall be in operation.**

....

APPLICABILITY: MODE 5.##

ACTION:

- a. With less than the above required loops operable, immediately initiate corrective action to return the required loops to OPERABLE status as soon as possible.
- b. With no RHR loop in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required RHR loop to operation.

SURVEILLANCE REQUIREMENTS

4.4.1.4 At least one residual heat removal loop shall be verified to be in operation and circulating reactor coolant at least once per 12 hours.

- # One RHR loop may be inoperable for up to two hours for surveillance testing, provided the other RHR loop is OPERABLE and in operation. Additionally, four filled reactor coolant loops, with at least two steam generators with their secondary side water levels greater than or equal to 5% (narrow range), may be substituted for one residual heat removal loop.
- ## A reactor coolant pump shall not be started with one or more of the RCS cold leg temperatures less than or equal to 312°F unless 1) the pressurizer water volume is less than 1650 cubic feet (equivalent to approximately 92% of level), or 2) the secondary water temperature of each steam generator is less than 50°F above each of the RCS cold leg temperatures.
- * Systems supporting RHR loop operability may be excepted as follows:
 - a. The normal or emergency power source may be inoperable.
 - b. One service water header may be out of service provided the equipment listed in Table 3.4-3 is OPERABLE.

** The residual heat removal pumps may be de-energized for up to 2 hours provided 1) no operations are permitted that would cause dilution of the reactor coolant system boron concentration, and 2) core outlet temperature is maintained at least 10°F below saturation temperature.

SALEM - UNIT 1

3/4 4-3b

ADOCK 05000272

PDR

8604230077 860408

PDR

REACTOR COOLANT SYSTEM

COLD SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.1.4 Two# residual heat removal loops shall be OPERABLE* and at least one RHR loop shall be in operation.**

......

APPLICABILITY: MODE 5.##

ACTION:

- a. With less than the above required loops operable, immediately initiate corrective action to return the required loops to OPERABLE status as soon as possible.
- b. With no RHR loop in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required RHR loop to operation.

SURVEILLANCE REQUIREMENTS

4.4.1.4 At least one residual heat removal loop shall be verified to be in operation and circulating reactor coolant at least once per 12 hours.

- # One RHR loop may be inoperable for up to two hours for surveillance testing, provided the other RHR loop is OPERABLE and in operation. Additionally, four filled reactor coolant loops, with at least two steam generators with their secondary side water levels greater than or equal to 5% (narrow range), may be substituted for one residual heat removal loop.
- ## A reactor coolant pump shall not be started with one or more of the RCS cold leg temperatures less than or equal to 312°F unless 1) the pressurizer water volume is less than 1650 cubic feet (equivalent to approximately 92% of level), or 2) the secondary water temperature of each steam generator is less than 50°F above each of the RCS cold leg temperatures.
- Systems supporting RHR loop operability may be excepted as follows:
 - a. The normal or emergency power source may be inoperable.
 - b. One service water header may be out of service provided the equipment listed in Table 3.4-3 is OPERABLE.
- ** The residual heat removal pumps may be de-energized for up to 2 hours provided 1) no operations are permitted that would cause dilution of the reactor coolant system boron concentration, and 2) core outlet temperature is maintained at least 10°F below saturation temperature.

ĩ