Docket Nos. 50-272 and 50-311

Mr. Richard A. Uderitz, Vice President -Nuclear Public Service Electric and Gas Company Post Office Box 236 Hancocks Bridge, New Jersey 08038

Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No.<sup>56</sup> to Facility Operating License No. DPR-70 and Amendment No.<sup>24</sup> to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letters dated March 28, 1983 and February 6, 1984.

These amendments (1) add existing manual initiation function for the auxiliary feedwater systems to the Technical Specifications, and (2) change the Containment System Air Locks Surveillance requirement in the Technical Specifications.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/DFischer

Donald Fischer, Project Manager Operating Reactors Branch #1 Division of Licensing

Enclosures: 1. Amendment No. 56 to DPR-70 2. Amendment No. 24 to DPR-75 DISTRIBUTION: 3. Safety Evaluation NRC PDR Docket File cc: w/enclosures L PDR ORB#1 Reading DEisenhut CParrish See next page DFischer **OELD** EJordan LHarmon TBarnhart, 8 JNGrace WJones DBrinkman ACRS, 10 OPA, CMiles RDiggs Gray File YHuang no legal childrin To firm pinent on FR. Potice anex OED DL ORB#1:DL ≹1:DL CParrish/di DFischer 6/ 1/84 6n~/84 8407310003 840 PDR ADOCK 05000

Mr. R. A. Uderitz Public Service Electric & Gas Company

cc: Mark J. Wetterhahn, Esquire Conner and Wetterhahn Suite 1050 1747 Pennsylvania Avenue, NW Washington, DC 20006

> Richard Fryling, Jr., Esquire Assistant General Solicitor Public Service Electric & Gas Company P. O. Box 570 - Mail Code T5E Newark, New Jersey 07101

Gene Fisher, Bureau of Chief Bureau of Radiation Protection 380 Scotch Road Trenton, New Jersey 08628

Mr. John M. Zupko, Jr. General Manager - Salem Operations Public Service Electric & Gas Company Post Office Box E Hancock Bridge, New Jersey 08038

Mr. Dale Bridenbaugh M.H.B. Technical Associates 1723 Hamilton Avenue San Jose, California 95125

James Linville, Resident Inspector Salem Nuclear Generating Station U.S. Nuclear Regulatory Commission Drawer I Hancock Bridge, New Jersey 08038

Richard F. Engel Deputy Attorney General Department of Law and Public Safety CN-112 State House Annex Trenton, New Jersey 08625

Richard B. McGlynn, Commission Department of Public Utilities State of New Jersey 101 Commerce Street Newark, New Jersey 07102 Salem Nuclear Generating Station

Regional Radiation Representative EPA Region II 26 Federal Plaza New York, New York 10007

Mr. R. L. Mittl, General Manager Nuclear Assurance and Regulation Public Service Electric & Gas Co. Mail Code T16D - P. O. Box 570 Newark, New Jersey 07101

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

Lower Alloways Creek Township c/o Mary O. Henderson, Clerk Municipal Building, P.O. Box 157 Hancock Bridge, NJ 08038

Mr. Alfred C. Coleman, Jr. Mrs. Eleanor G. Coleman 35 K Drive Pennsville, New Jersey 08070

Carl Valore, Jr., Esquire Valore, McAllister, Aron and Westmoreland, P.A. 535 Tilton Road Northfield; NJ 08225

June D. MacArtor, Esquire Deputy Attorney General Tatnall Building Post Office Box 1401 Dover, Delaware 19901

Harry M. Coleman, Mayor Lower Alloways Creek Township Municipal Hall Hancock Bridge, New Jersey **CB**038

Salem Nuclear Generating Station Units 1 and 2

- 2 -

cc: Mr. Edwin A. Liden, Manager Nuclear Licensing & Regulation Public Service Electric & Gas Company Post Office Box 236 Hancock Bridge, New Jersey 08038

> Mr. Charles P. Johnson Assistant to Vice President, Nuclear Public Service Electric & Gas Company Post Office Box 570 80 Park Plaza - 15A Newark, New Jersey 07101

Mr. David Wersan Assistant Consumer Advocate Office of Consumer Advocate 1425 Strawberry Square Harrisburg, PA 17120



### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### PUBLIC SERVICE ELECTRIC AND GAS COMPANY PHILADELPHIA ELECTRIC COMPANY DELMARVA POWER AND LIGHT COMPANY ATLANTIC CITY ELECTRIC COMPANY

### DOCKET NO. 50-272

### SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.56 License No. DPR-70

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated March 28, 1983, as supplemented February 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

8407310005 840716 PDR ADDCK 05000272

PDR

### (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief Operating Reactors Branch #1

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: July 16, 1984

# ATTACHMENT TO LICENSE AMENDMENT NO. 56 FACILITY OPERATING LICENSE NO. DPR-70 DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-20a	3/4 3-20a
3/4 3-21	3/4 3-21
3/4 3-22	3/4 3-22
3/4 3-26	3/4 3-26
3/4 3-33	3/4 3-33
3/4 3-34	3/4 3-34
3/4 6-5	3/4 6-5
3/4 6-6	3/4 6-6

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# ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNC	CTION	IAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPL ICABLE MODES	ACTION.
8.	AUX	ILIARY FEEDWATER					
	a.	Automatic Actuation Logic **	2	1	2	1, 2, 3	20 (
	b.	Manual Initiation	1/pump	1/pump	1/pump	1, 2, 3	22
	с.	Steam Generator Water LevelLow-Low					
	•	i. Start Motor Driven Pumps	3/stm. gen.	2/stm. gen.	2 stm. gen.	1, 2, 3	14*
•		ii. Start Turbine- Driven Pumps	3/stm. gen.	2/stm. gen.	2 stm. gen.	1, 2, 3	14*
	đ.	Undervoltage – RCP Start Turbine- Driven Pump	4(1/bus)	1/2 x 2	3	1, 2	19
	e.	S.I. Start Motor-Driven Pumps	See 1 above (Al	] S.I. įnitiat	ing functions an	d requirements)	
	f.	Emergency Trip of Steam Generator Feedwater Pumps Start Motor Driven Pumps	2(1/pump)	2	2(1/pump)	1	21
	g.	Station Blackout	See 6 and 7 at	 Dove (SEC and U	/V Vital Bus)	• .	

3/4 3-20a

Amendment No.

56

\*\*Applies to items c and d.

### TABLE NOTATION

#Trip function may be bypassed in this MODE below P-11.

##Trip function may be bypassed in this MODE below P-12

###Trip channel(s) associated with the protective functions derived from the out of service Reactor Coolant Loop shall be placed in the tripped mode.

\*The provisions of Specification 3.0.4 are not applicable.

### ACTION STATEMENTS

- ACTION 13 With the number of OPERABLE Channels one less than the Total Number of Channels, be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 1 hour for surveillance testing per Specification 4.3.2.1.1.
- ACTION 14 With the number of OPERABLE Channels one less than the Total Number of Channels, operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST, provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 15 With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT SHUTDOWN within the following 12 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.
- ACTION 16 With the number of OPERABLE Channels one less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the bypassed condition and the Minimum Channels OPERABLE requirement is demonstrated within 1 hour; one additional channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.
- ACTION 17 With less than the Minimum Channels OPERABLE, operation may continue provided the containment purge and exhaust valves are maintained closed.
- ACTION 18 With the number of OPERABLE Channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SALEM - UNIT 1

ACTION 19 - With the number of OPERABLE Channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

- a. The inoperable channel is placed in the tripped condition within 1 hour.
- b. The Minimum Channels OPERABLE requirements is met; however, one additional channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1

DESIGNATION	CONDITION AND SETPOINT	FUNCTION
P-11	With 2 of 3 pressurizer pressure channels <u>&gt;</u> 1925 psig.	P-11 prevents or defeats manual block of safety injection actuation on low pressurizer pressure.
P-12	With 3 of 4 T <sub>avg</sub> channels <u>&gt;</u> 545°F.	P-12 prevents or defeats manual block of safety injection actuation high steam line flow and low steam line pressure.
•	With 2 of 4 T <sub>avg</sub> channels < 541°F.	Allows manual block of safet injection actuation on high steam line flow and low steam line pressure. Causes steam line isolation on high steam flow. Affects steam dump blocks.

- ACTION 20 With the number of OPERABLE channels one less than the Total Number of Channels, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 1 hour for surveillance testing.
- ACTION 21 With the number of OPERABLE channels one less than the Minimum Number of Channels, operation may proceed provided that either:
  - a. The inoperable channel is restored to OPERABLE within 72 hours, or
  - b. If the affected Steam Generator Feedwater Pump is expected to be out of service for more than 72 hours, the inoperable channel is jumpered so as to enable the Start Circuit of the Auxiliary Feedwater Pumps upon the loss of the other Steam Generator Feedwater Pump.
- ACTION 22 With the number of OPERABLE channels relating directly with the number of OPERABLE auxiliary feedwater pumps, the ACTIONs of L.C.O. 3.7.1.2 apply.

### ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

### FUNCTIONAL UNIT

High-High

SYSTEM (SEC)

TURBINE TRIP AND FEEDWATER ISOLATION

a. Steam Generator Water Level --

SAFEGUARDS EQUIPMENT CONTROL

b. Sustained Degraded Voltage

a. Automatic Actuation Logic

Steam Generator Water Level --

UNDERVOLTAGE, VITAL BUS

a. Loss of Voltage

AUXILIARY FEEDWATER

b. Manual Initiation

d. Undervoltage - RCP

q. Station Blackout

f. Emergency Trip of Steam

**Generator Feedwater Pumps** 

Low-Low

**S.I.** 

e.

### TRIP SETPOINTS

### ALLOWABLE VALUES

< 67% of narrow range
Instrument span each
steam generator</pre>

Not Applicable

> 70% of bus voltage

> 91% of bus voltage for < 13 seconds

Not Applicable

Not Applicable

> 18% of narrow range Tnstrument span each steam generator

> 70% RCP bus voltage

See 1 above (All S.I. setpoints)

Not Applicable

Not Applicable

See 6 and 7 above (SEC and Undervoltage, Vital Bus)

< 68% of narrow range Instrument span each steam generator

Not Applicable

> 65% of bus voltage

> 90% of bus voltage for  $< \sqrt{15}$  seconds

Not Applicable

Not Applicable

> 17% of narrow range Instrument span each steam generator

> 65% RCP bus voltage

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Amendment No. 56

3/4 3-26

**~ALEM** 

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UNIT

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5.

6.

· 7.

8.

# ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS SURVEILLANCE REQUIREMENTS

FUN	CTIONA	LUNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE REQUIRED		
8.	AUXI	LIÅRY FEEDWATER				、 、		
	a.	Automatic Actuation Logic	N.A.	N.A.	M(2).	1, 2, 3		
	<b>b.</b> i	Manual Initiation	N.A.	N.A.	M(4)	1, 2, 3		
	с.	Steam Generator Water LevelLow-Low	S	R	M .	1, 2, 3		
	d.	Undervoltage - RCP	S	R	M(2)	1, 2		
	e.	S.I.	See 1 above (All S.I. surveillance requirements)					
	f.	Emergency Trip of Steam Generator Feedwater Pumps	N.A.	N.A.	R	1		
	g.	Station Blackout See 6b and 7 above (SEC and U/V Vital Bus)						

SALEM - UNIT

### TABLE NOTATION

- (1) Each logic channel shall be tested at least once per 62 days on a STAGGERED TEST BASIS. The CHANNEL FUNCTION TEST of each logic channel shall verify that its associated diesel generator automatic load sequence timer is OPERABLE with the interval between each load block within 1 second of its design interval.
- (2) Each train or logic channel shall be tested at least every 62 days on a staggered basis.
- (3) The CHANNEL FUNCTIONAL TEST shall include exercising the transmitter by applying either a vacuum or pressure to the appropriate side of the transmitter.
- (4) The CHANNEL FUNCTIONAL TEST shall be conducted in conjunction with the SURVEILLANCE REQUIREMENT OF 4.7.1.2. a

Amendment No. 56 ...

### CONTAINMENT SYSTEMS

### CONTAINMENT AIR LOCKS

### LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of  $\leq 0.05 L_a$  at design pressure (47.0 psig).

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

With an air lock inoperable, restore the air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. \*After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours by pressurizing the volume between the air lock door gaskets to  $\geq$  10.0 psig and checking for an extrapolated\*\* seal leakage rate equal to or less than 0.01 La.
- b. At least once per 6 months by conducting an overall air lock leakage test at design pressure (47.0 psig) and by verifying that the overall air lock leakage rate is within its limit, and
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

\* Exemption to Appendix "J" of 10 CFR 50.

\*\* The measured leakage at the test pressure (10 psig) shall be multiplied by an extrapolation factor of 9.1 to determine what the seal leakage flow rate would be if tested at design pressure (47.0 psig). This extrapolated seal leakage rate shall be less than or equal to 0.01 La.

ISALEM - UNIT 1

### CONTAINMENT SYSTEMS

INTERNAL PRESSURE

### LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -1.5 and +0.3 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to be within the limits at least once per 12 hours.

SALEM - UNIT 1



### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### PUBLIC SERVICE ELECTRIC AND GAS COMPANY PHILADELPHIA ELECTRIC COMPANY DELMARVA POWER AND LIGHT COMPANY ATLANTIC CITY ELECTRIC COMPANY

### DOCKET NO. 50-311

### SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.24 License No. DPR-75

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated March 28, 1983, as supplemented February 6, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

### (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. <sup>24</sup>, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

arga, Operating Reactors Branch #1 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: July 16, 1984

# ATTACHMENT TO LICENSE AMENDMENT NO. 24

# FACILITY OPERATING LICENSE NO. DPR-75

## DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-21	3/4 3-21
3/4 3-22	3/4 3-22
3/4 3-23	3/4 3-23
3/4 3-27	3/4 3-27
3/4 3-36	3/4 3-36
3/4 3-37	3/4 3-37
3/4 6-5	3/4 6-5
3/4.6-6	. 3/4 6-6

# ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

MINIMUM **APPLICABLE** CHANNELS CHANNELS TOTAL NO. **ACTION** MODES **OPERABLE** TO TRIP **OF CHANNELS** FUNCTIONAL UNIT AUXILIARY FEEDWATER 8. Automatic Actuation a. 20 1, 2, 3 2 2 1 Logic\*\* 1, 2, 3 23 1/pump 1/pump Manual Initiation 1/pump b. Stm. Gen. Water C. Level-Low-Low Start Motor i. 14\* 1, 2, 3 2/stm. gen. 2 stm. gen. **Driven Pumps** 3/stm. gen any stm. gen. ii. Start Turbine-14\* \* 1, 2, 3 3/stm. gen. 2/stm. gen. 2 stm. aen. Driven Pumps any 2 stm. gen. Undervoltage-RCP **d**. Start Turbine-19 1, 2 1/2 x 2 3 4-1/bus **Driven Pump** S. I. e. Start Motor-Driven See 1 above (All S.I. initiating functions and requirements) Pumps · f. Trip of Main Feedwater Pumps Start Motor-Driven 22\* 1/pump 1, 2 1/pump 2/Pump Pumps 1

\*\*Applies to items c and d.

SALEM - UNIT

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3/4 3-21

Amendment No. 24

1

### TABLE NOTATION

"Trip function may be bypassed in this MODE below P-11 (Pressurizer Pressure Block of Safety Injection) setpoint.

Trip function may be bypassed in this MODE below P-12 (T Block of Safety Injection) setpoint.

The provisions of Specification 3.0.4 are not applicable.

### ACTION STATEMENTS

- ACTION 13 With the number of OPERABLE Channels one less than the Total Number of Channels, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 14 With the number of OPERABLE Channels one less than the Total Number of Channels, operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST, provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 15 With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT SHUTDOWN within the following 12 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.
- ACTION 16 With the number of OPERABLE Channels one less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the bypassed condition and the Minimum Channels OPERABLE requirement is demonstrated by CHANNEL CHECK within 1 hour; one additional channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.
- ACTION 17 With less than the Minimum Channels OPERABLE, operation may continue provided the containment purge and exhaust valves are maintained closed.
- ACTION 18 With the number of OPERABLE Channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SALEM - UNIT 2

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3/4 3-22

Amendment No. 24 ...

#### ACTION 19

 With the number of OPERABLE Channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

a. The inoperable channel is placed in the tripped condition within 1 hour.

 b. The Minimum Channels OPERABLE requirements is met; however,
 one additional channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.

### ENGINEERED SAFETY FEATURES INTERLOCKS

### DESIGNATION

### CONDITION AND SETPOINT

### P-11 ·

P-12

With 2 of 3 pressurizer pressure channels ≥ 1925 psig.

With 3. of 4 T avg channels  $\geq$  545°F.

With 2 of 4 T avg Channels < 541°F.

P-11 prevents or defeats manual block of safety injection actuation on low pressurizer pressure. P-12 prevents or defeats : manual block of safety injection actuation high steam line flow and low steam line pressure. - Allows manual block of safety injection actuation on high steam line flow and low steam line pressure. Causes steam line isolation on high steam flow. Affects steam dump blocks.

FUNCTION

- ACTION 20 With the number of OPERABLE channels one less than the Total Number of Channels, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 1 hour for surveillance testing provided the other channel is OPERABLE.
- ACTION 21 With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.

ACTION 22 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE, operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST.

ACTION 23 - With the number of OPERABLE channels relating directly with the number of OPERABLE auxiliary feedwater pumps, the ACTIONs of L.C.O. 3.7.1.2 apply.

SALEH - UNIT 2

# ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

### FUNCTIONAL UNIT

# TRIP SETPOINT

# 5. TURDINE TRIP AND FEEDWATER ISOLATION

a. Steam Generator Water Level---High-High

- 6. SAFEGUARDS EQUIPMENT CONTROL SYSTEM (SEC)
- 7. UNDERVOLTAGE, VITAL BUS
  - . Love of Voltage
  - b. Gustained Degraded Voltage
- 8. AUXILIARY FEEDWATER

a. Automatic Actuation Logic

- b. Manual Initiation
- c. Steam Generator Water Level-low-low

d. Undervoltage - RCP

e. S.I.

f. Trlp of Hain Feedwater Pumps

67X of narrow range
 Instrument span each steam
 generator

Not Applicable

> 70% of bus voltage

 $\geq$  91% of bus voltage for  $r \leq 10$  seconds

Not Applicable

Not Applicable

> 18% of narrow range Instrument span each steam generator

 $\geq$  70% RCP bus voltage

See 1 Above (All S.I. setpoints)

### Not Applicable

ALLOWABLE VALUES

 68% of narrow range Instrument span each steam generator

Not Applicable

2.65% of bus voltage

£90% of bus voltage for ≤15'seconds

Not Applicable

Not Applicable

Not Applicable

> 17% of narrow range Instrument span each steam generator

≥ 65% RCP bus voltage

3/4 3-27

- UNIT 2

SALEX

## ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION SURVETILLANCE REQUIREMENTS

FUNCTIONAL UNIT		CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE 	
8.	AUXILIARY FEEDWATER					
	a.	Automatic Actuation Logic	N. A.	H. A.	H(2)	1, 2, 3
	b.	Manual Initiation	H. A.	H.A.	<b>M</b> (5)	1, 2, 3
	c. )	Steam Generator Water Level-Low-Low	S	, <b>R</b>	H	1, 2, 3
	d.	Undervoltage - RCP	<b>S</b> .	<sub>p</sub> R	м	1, 2
e. S.I. See 1 above (All S.I.			ove (All S.I. su	surveillance requirements)	irements)	
	f.	Trip of Main Feedwater Pumps	N.A. '	N. A.	S/U(4)	1, 2

SALEM - UNIT 2

### TABLE NOTATION

- (1) Each logic channel shall be tested at least once per 62 days on a STAGGERED TEST BASIS. The CHANNEL FUNCTION TEST of each logic channel shall verify that its associated diesel generator automatic load sequence timer is OPERABLE with the interval between each load block within 1 second of its design interval.
- (2) Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (3) The CHANNEL FUNCTIONAL TEST shall include exercising the transmitter by applying either a vacuum or pressure to the appropriate side of the transmitter.
- (4) If not performed in the previous 92 days.
- (5) The CHANNEL FUNCTIONAL TEST shall be conducted in conjunction with the SURVEILLANCE REQUIREMENT of 4.7.1.2.a

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### CONTAINMENT SYSTEMS

### CONTAINMENT AIR LOCKS

### SURVEILLANCE REQUIREMENTS (Continued

- a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours by pressurizing the volume between the air lock door gaskets to  $\geq$  10.0 psig and checking for an extrapolated\* seal leakage rate equal to or less than 0.01 La,
- b. Prior to establishing CONTAINMENT INTEGRITY, if opened when CONTAIN-MENT INTEGRITY was not required, and at least once per 6 months by conducting an overall air lock leakage test at design pressure (47.0 psig) and by verifying that the overall air lock leakage rate is within its limit#, and
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

SALEM - UNIT 2

<sup>\*</sup> The measured leakage at the test pressure ( $\geq$  10.0 psig) shall be multiplied by an extrapolation factor of 9.1 to determine what the seal leakage flow rate would be at design pressure (47.0 psig). This extrapolated seal leakage rate shall be equal to or less than 0.01 L<sub>a</sub>.

<sup>#</sup> The provisions of Specification 4.0.2 are not applicable.

### CONTAINMENT SYSTEMS

### INTERNAL PRESSURE

### LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -1.5 and +0.3 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

### ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to be within the limits at least once per 12 hours.



### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.<sup>56</sup> TO FACILITY OPERATING LICENSE NO. DPR-70

AND AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY PHILADELPHIA ELECTRIC COMPANY DELMARVA POWER AND LIGHT COMPANY, AND ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATION STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

### INTRODUCTION

On March 28, 1983, Public Service Electric and Gas Company (PSE&G) submitted an amendment change request which would (1) add existing manual initiation functions for the auxiliary feedwater systems to the appropriate Engineered Safety Feature Tables in the Technical Specifications, and (2) change the Containment Systems Air Locks Surveillance requirement in the Technical Specifications. This change would reduce the air lock seal testing pressure, as allowed by a change in the regulations, for those cases where the air locks are being frequently opened. Specifically, the surveillance requirement would be changed to read:

"After each opening, except when the airlock is being used for multiple entries, then at least once per 72 hours, prove gasket integrity by pressurizing the volume between door gaskets to 10 psig and checking for an extrapolated seal leakage rate equal to or less than 0.01 La."

The present test pressure is 47.0 psig, the peak accident pressure.

On February 6, 1984, PSE&G submitted two pages of Technical Specifications that were inadvertently left out of the March 28, 1983 package.

### EVALUATION AND SUMMARY

Change (1) adds the Manual Initiation function for the Auxiliary Feedwater System to the scope of the Engineered Safety Feature Actuation System instrumentation. This action satisfies our position, requirement No. 5 of NUREG-0737 Item II.E.1.2, and responds to our letter dated December 22, 1982, where we requested that the manual initiation function be added to the Technical Specifications. We have reviewed the specification changes to assure adequacy and conclude that the changes are acceptable.

8407310007 840716 PDR ADOCK 05000272 PDR PDR Change (2) would allow for air locks, a "between the seals test pressure" of 10 psig when testing for seal leakage during periods when the air locks are frequently opened. The present Technical Specifications require a test pressure of 47.0 psig which is the peak accident pressure. 10 CFR 50 Appendix J has recently been changed to allow test pressures other than the peak accident pressure to be substituted when testing air lock seal during periods of frequent openings. Since change (2) conforms to this change in the regulation, the staff has determined that the action is acceptable.

### ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation of use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (or the Commission has made a final no significant hazards consideration finding with respect to this amendment). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 50.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 16, 1984

Principal Contributors: D. Fischer

Y. Huang