DISTRIBUTION
Dockets HShaw
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GDeegan-8
DBrinkman
LSnyder
OPA
RDiggs
CParrish
GMeyer
ASLAB

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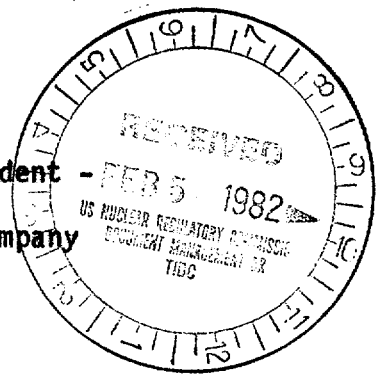
Docket Nos. 50-272
and 50-311



Mr. Richard A. Uderitz, Vice President - Nuclear
Public Service Electric and Gas Company
Mail Code T15A
Post Office Box 570
Newark, New Jersey 07101

Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No. 40 to Facility Operating License No. DPR-70 and Amendment No. 5 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated August 10, 1981.

These amendments revise the Technical Specifications related to hydraulic and mechanical snubbers.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Gary C. Meyer, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 40 to DPR-70
2. Amendment No. 5 to DPR-75
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

*Previous concurrence see next page

OFFICE	ORB#1:DL*	ORB#1:DL*	ORB#1:DL*	AD/OR:DL*	OELD*	ORAB*	
SURNAME	CParrish	GMeyer:ds	SVarga	TNovak		HShaw	
DATE	01/ /82	01/ /82	01/ /82	01/ /82	01/ /82	01/ /82	

DISTRIBUTION
Dockets HShaw
NRC PDR
L PDR
TERA
NSIC
ORB#1 Rdg
DEisenhut
OELD
IE-4
ACRS-10
GDeegan-8
DBrinkman
LSnyder
OPA
RDiggs
CParrish
GMeyer
ASLAB
Gray File

Docket Nos. 50-272
and 50-311

Mr. F. W. Schneider, Vice President
Production
Public Service Electric and Gas Company
80 Park Plaza 15A
Newark, New Jersey 07101

Dear Mr. Schneider:

The Commission has issued the enclosed Amendment No. 40 to Facility Operating License No. DPR-70 and Amendment No. 5 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated August 10, 1981.

These amendments revise the Technical Specifications related to hydraulic and mechanical snubbers.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Gary C. Meyer, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 40 to DPR-70
2. Amendment No. 5 to DPR-75
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

Concur subject to Changes

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL	AD/OR:DL	OELD	ORAB	
SURNAME	CParrish	GMeyer:ds	SVarga	TNovak	[illegible]	HShaw	
DATE	12/9/81	12/15/81	12/29/81	12/31/81	12/25/81	01/19/82	

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY USGPO: 1981—335-960

Mr. R. A. Uderitz
Public Service Electric and Gas Company

cc: Mark J. Wetterhahn, Esquire
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Public Service Electric and Gas Company
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Gene Fisher, Bureau of Chief
Bureau of Radiation Protection
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Trenton, New Jersey 08628

Mr. Henry J. Midura, General Manager - Salem Operations
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Hancocks Bridge, New Jersey 08038

Salem Free Library
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Salem, New Jersey 08079

Leif J. Norrholm, Resident Inspector
Salem Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Drawer I
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Deputy Attorney General
Department of Law and Public Safety
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Trenton, New Jersey 08625

Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancocks Bridge, New Jersey 08038

June D. MacArtor, Esquire
Deputy Attorney General
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Dover, Delware 19901

Richard B. McGlynn, Commissioner
Department of Public Utilities
State of New Jersey
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Newark, New Jersey 07102

Regional Radiation Representatives
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New York, New York 10007

Mr. R. L. Mittl, General Manager - Corporate Quality Assurance
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Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
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Mr. Alfred C. Coleman, Jr.
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Carl Valore, Jr., Esquire
Valore, McAllister, Aron and Westmoreland, P.A.
535 Tilton Road
Northfield, New Jersey 08225

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 10, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 40, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 2, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 7-28	3/4 7-28
3/4 7-29	3/4 7-29
3/4 7-30	3/4 7-30
3/4 7-31	3/4 7-31
3/4 7-32	3/4 7-32
--------	3/4 7-32a
--------	3/4 7-32b
--------	3/4 7-33 through 3/4 7-33o

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All snubbers listed in Tables 3.7-4a and 3.7-4b shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours, replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9c on the supported component or declare the supportec system inoperable and follow appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspection

An inservice visual inspection of all snubbers listed in Tables 3.7-4a and 3.7-4b shall be performed in accordance with the following schedule for each separate Table:

No. Inoperable Snubbers in a Table per Inspection Period	Subsequent Inspection Period for each Table**
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3, 4	124 days ± 25%
5, 6, 7	62 days ± 25%
8 or more	31 days ± 25%

Within each Table, the snubbers may be categorized into two groups: Those accessible and those inaccessible during reactor operation. Each group within a Table may be inspected independently in accordance with the above schedule.

*The inspection interval shall not be lenghthened more than one step at a time.

#The provisions of Specification 4.0.2 are not applicable.

SURVEILLANCE REQUIREMENTS (Continued)

b. Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not frozen up. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; and (2) the affected snubber is functionally tested in the as found condition and determined OPERABLE per Specification 4.7.9d or 4.7.9e as applicable. However, when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be declared inoperable and cannot be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval. All snubbers connected to an inoperable common hydraulic fluid reservoir shall be counted as inoperable snubbers.

c. Functional Tests **

At least once per 18 months during shutdown, a representative sample of 10% of the total of each type of snubber in use in the plant shall be functionally tested either in place or in a bench test. For each type of snubber that does not meet the functional test acceptance criteria of Specification 4.7.9d or 4.7.9e, an additional 10% of that type of snubber shall be functionally tested.

The representative sample selected for functional testing shall include the various configurations, operating environments and the range of size and capacity of snubbers. At least 25% of the snubbers in the representative sample shall include snubbers from the following three categories:

1. The first snubber away from each reactor vessel nozzle
2. Snubbers within five feet of heavy equipment (valve, pump, turbine, motor, etc.)
3. Snubbers within ten feet of the discharge from a safety relief valve

Snubbers identified in Tables 3.7-4a and 3.7-4b as "Especially Difficult to Remove" or in "High Radiation Zones During Shutdown" shall also be included in the representative sample.* Tables 3.7-4a

*Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

**Functional testing of all mechanical snubbers and any hydraulic snubber with a rated capacity of greater than 50,000 lbs. shall commence with the 4th refueling outage.

and 3.7-4b may be used jointly or separately as the basis for the sampling plan.

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

d. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

e. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force. The drag force shall not have increased more than 50% since the last functional test.

2. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

SURVEILLANCE REQUIREMENTS (Continued)

3. Snubber release rate, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

f. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.1.

Concurrent with the first inservice visual inspection and at least once per 18 months thereafter, the installation and maintenance records for each snubber listed in Tables 3.7-4a and 3.7-4b shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

TABLE 3.7-4A

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
1	No. 11 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
2	No. 11 Steam Generator. Reactor Containment, Elev. 128' 6"	I	Yes	Yes
3	No. 11 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
4	No. 11 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
5	No. 12 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
6	No. 12 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
7	No. 12 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
8	No. 12 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes

TABLE 3.7-4A(Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
9	No. 13 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
10	No. 13 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
11	No. 13 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
12	No. 13 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
13	No. 14 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
14	No. 14 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
15	No. 14 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes
16	No. 14 Steam Generator, Reactor Containment, Elev. 128' 6"	I	Yes	Yes

TABLE 3.7-4a(Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
17.	No. 11 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
18.	No. 11 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
19.	No. 12 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
20.	No. 12 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
21.	No. 13 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
22.	No. 13 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
23.	No. 14 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
24.	No. 14 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.4 7-4a provided that a revision to Table 3.4 7-4a is included with the next License Amendment request.

** Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-4a is included with the next License Amendment request.

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
1	Reactor Coolant System, Pressurizer South Centerline, Reactor Containment, Elev. 158' 0"	I	Yes	No
2	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No
3	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No
4	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No
5	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No
6	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No
7	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 158' 0"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
8	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 142' 8"	I	Yes	No
9	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 140' 3"	I	Yes	No
10	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 137' 8"	I	Yes	No
11	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 137' 8"	I	Yes	No
12	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 137' 8"	I	Yes	No
13	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 137' 8"	I	Yes	No
14	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 137' 8"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
15	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 144' 8"	I	Yes	No
16	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 141' 5"	I	Yes	No
17	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 141' 5"	I	Yes	No
18	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 139' 8"	I	Yes	No
19	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 139' 8"	I	Yes	No
20	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 146' 4"	I	Yes	No
21	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 144' 8"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
22	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 142' 8"	I	Yes	No
23	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 142' 8"	I	Yes	No
24	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 142' 6"	I	Yes	No
25	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 141' 7 1/2"	I	Yes	No
26	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 151' 5 7/8"	I	Yes	No
27	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 151' 5 7/8"	I	Yes	No
28	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 149' 0"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
29	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 149' 0"	I	Yes	No
30	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 147' 5"	I	Yes	No
31	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 146' 11"	I	Yes	No
32	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 146' 11"	I	Yes	No
33	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 151' 5 7/8"	I	Yes	No
34	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 151' 8 7/8"	I	Yes	No
35	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 147' 0"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
36	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 147' 0"	I	Yes	No
37	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 147' 1 1/2"	I	Yes	No
38	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 143' 8"	I	Yes	No
39	Reactor Coolant System, Pressurizer @ Valve 1PR-4, Reactor Containment Elev. 154' 5 1/8"	I	Yes	Yes
40	Reactor Coolant System, Pressurizer @ Valve 1PR-5, Reactor Containment Elev. 154' 5 1/8"	I	Yes	Yes
41	Reactor Coolant System, Pressurizer @ Valve 1PR-3, Reactor Containment Elev. 154' 5 1/8"	I	Yes	Yes
42	Reactor Coolant System, Pressurizer @ Valve 1PR-6, Reactor Containment Elev. 152' 1 7/8 "	I	Yes	Yes

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
43	Reactor Coolant System, Pressurizer @ Valve 1PR-7, Reactor Containment Elev. 152' 1 7/8"	I	Yes	Yes
44	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 94' 11 1/2"	I	Yes	No
45	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 94' 8 1/2"	I	Yes	No
46	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 92' 8 5/8 "	I	Yes	No
47	Reactor Coolant System, Annulus Bet. A9-3 & A9-4, Reactor Containment Elev. 92' 5 5/16"	I	No	No
48	Reactor Coolant System, Annulus Bet. A9-3 & A9-4, Reactor Containment Elev. 92' 5 5/16"	I	No	No
49	Reactor Coolant System, Annulus Bet. A8-3 & A8-4, Reactor Containment Elev. 92' 6 3/8"	I	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
50	Reactor Coolant System, Annulus Bet. A8-3 & A8-4, Reactor Containment Elev. 92' 6 3/8"	I	No	No
51	Stm. Gen. Drns. & Bldn., Bet. #13 Stm. Gen. & Crane Wall, Reactor Containment, Elev. 94' 8"	I	Yes	No
52	CVC System, Off #14 RCP Platfm. @ Valve 14CV-104, Reactor Containment Elev. 103' 11 3/4"	I	Yes	No
53	CVC System, Bet. #14 Stm. Gen. & Shield Wall, Reactor Containment Elev. 100' 4 3/8"	I	Yes	No
54	CC System, S.W. Corner of #13 RCP Platfm., Reactor Containment Elev. 109' 0"	I	Yes	No
55	CC System, Bet. W. Side of #13 RCP Platfm. & Wall, Reactor Containment Elev. 102' 4"	I	Yes	No
56	RHR & SI System, #14 Stm. Gen. Near N.S. Centerline of Containment, Reactor Containment, Elev. 98' 0"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
57	RHR & SI System, Bet. #14 Stm. Gen. & Crane Wall, Reactor Containment Elev. 98' 0"	I	Yes	No
58	RHR & SI System, 10' 4 1/8" So. of 11.2, 7' 0" East of HH, N. Pen. Area, Elev. 85' 2"	A	No	No
59	RHR & SI System, 10' 4 1/8" So. of 11.2, 7' 0" East of HH, N. Pen. Area, Elev. 85' 3 1/2"	A	No	No
60	RHR & SI System, 10' 4 1/8" So. of 11.2, 6' 9" East of HH, Pen. Area, Elev. 85' 3 1/2"	A	No	No
61	RHR & SI System, 10' 4 1/8" So. of 11.2, 6' 9" East of HH, Pen. Area, Elev. 87' 6"	A	No	No
62	RHR & SI System, 8' 1" So. of 11.8, 7' 1" East of HH, Pen. Area, Elev. 79' 6"	A	No	No
63	Cont. Spray System, 2' 6" So. of 11.8, 6' 2" West of GG, Pen. Area, Elev. 89' 0"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
64	DELETED PER DCR #1EC1260			
65				
66	Stm. Gen. Fdwtr., 23' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 128' 10"	A	No	No
67	Stm. Gen. Fdwtr., 23' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 128' 10"	A	No	No
68	Stm. Gen. Fdwtr., 23' 4" So. of 11.8, 2' 0" East of DC, Pen. Area, Elev. 130' 6"	A	No	No
69	Stm. Gen. Fdwtr., 23' 4" So. of 11.8, 2' 0" East of DC, Pen. Area, Elev. 130' 6"	A	No	No
70	Stm. Gen. Fdwtr., Near East Wall, South Pen. Area, Elev. 128' 10"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
71	Stm. Gen. Fdwtr., Near East Wall, South Pen. Area, Elev. 128' 10"	A	No	No
72	Stm. Gen. Fdwtr., Outside East Wall, South Pen. Area, Elev. 130' 6"	A	No	No
73	Stm. Gen. Fdwtr., 8' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 108' 3"	A	No	No
74	Stm. Gen. Fdwtr., 8' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 108' 9 1/8"	A	No	No
75	Stm. Gen. Fdwtr., 8' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 108' 9 1/8"	A	No	No
76	Stm. Gen. Fdwtr., 8' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 131' 2"	A	No	No
77	Stm. Gen. Fdwtr., 8' 4" So. of 11.8, 6' 6" West of DC, Pen. Area, Elev. 131' 2"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
78	Stm. Gen. Fdwtr., S.E. Corner, South Pen. Area, Elev. 108' 3"	A	No	No
79	Stm. Gen. Fdwtr., S.E. Corner, South Pen. Area, Elev. 108' 9 1/8"	A	No	No
80	Stm. Gen. Fdwtr., S.E. Corner, South Pen. Area, Elev. 131' 2"	A	No	No
81	Stm. Gen. Fdwtr., S.E. Corner, South Pen. Area, Elev. 131' 2"	A	No	No
82	M.S. SA Valve Strut, 23' 1 1/4" East of FF, South Pen. Area, Elev. 104' 3 1/2"	A	No	No
83	M.S. SA Valve Strut, 12' 0" East of FF, South Pen. Area, Elev. 106' 0"	A	No	No
84	M.S. to Aux.FWP Exhaust, 8' 7" So. of 11.8, 16' 10" West of FF, Pen. Area, Elev. 110' 5 1/2"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
85	M.S. to FWP Exhaust, 8' 7" So. of 11.8, 16' 10" West of FF, Pen. Area, Elev. 110' 5 1/2"	A	No	No
86	M.S. to FWP Exhaust, 8' 1 1/8" North of 12.3, 3' 8 3/8" East of LL, Aux. Bldg., Elev. 91' 2"	A	No	No
87	M.S. to FWP Exhaust, 8'1 1/8" North of 12.3, 3' 8 3/8" East of LL, Aux. Bldg., Elev. 91' 2"	A	No	No
88	M.S. to FWP Exhaust, 6' 4 3/8" North of 12.3, 1' 10 1/8" East of LL, Aux. Bldg., Elev. 90' 11"	A	No	No
89	M.S. to FWP Exhaust, 6' 4 3/8" North of 12.3, 1' 10 1/8" East of LL, Aux. Bldg., Elev. 90' 11"	A	No	No
90	M.S. to FWP Exhaust, 3' 3" North of 11.8, 4' 7" East of LL, Aux. Bldg., Elev. 95' 0"	A	No	No
91	M.S. to FWP Exhaust, 3' 3" North of 11.8, 4' 7" East of LL, Aux. Bldg., Elev. 95' 0"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
92	CVC System, 3' 8" South of 12.3, 1' 11" West of TT, Aux. Bldg., Elev. 97' 2"	A	No	No
93	CVC System, 8' 4" North of 12.3, 3' 5" West of NN, Aux. Bldg., Elev. 101' 11"	A	No	No
94	CC System, 6' 11' North of 12.3, 10' 9" East of GG, Aux. Bldg., Elev. 96' 0"	A	No	No
95	CC System, 6' 11" North of 12.3, 10' 9" East of GG, Aux. Bldg., Elev. 96' 0"	A	No	No
96	CC System, 3' 5" North of 11.8, 1' 3" East of JJ, Aux. Bldg., Elev. 107' 7"	A	No	No
97	RHR & SI System, 17' 10" South of 14, 5' 6 1/4" East of RR, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
98	RHR & SI System, 17' 10" South of 14, 5' 6 1/4" East of RR, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
99	RHR & SI System, 7' 9" South of 14, 4' 11" East of LL, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
100	RHR & SI System, 7' 9" South of 14, 3' 3" East of LL, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
101	RHR & SI System, 11 3/4" North of 13.2, 4 3/8" East of HH, Aux. Bldg., Elev. 86' 3"	A	No	No
102	RHR & SI System, 11 3/4" North of 13.2, 4 3/8" East of HH, Aux. Bldg., Elev. 86' 3"	A	No	No
103	RHR & SI System, 8' 9" South of 13.2, 9' 10" East of SS, Aux. Bldg., Elev. 86' 1"	A	No	No
104	RHR & SI System, 8' 9" South of 13.2, 9' 10" East of SS, Aux. Bldg., Elev. 86' 1"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
105				
106	DELETED-REPLACED WITH RIGID STRUTS			
107				
108				
109	Spent Fuel Cooling System, 5' 0" South of 12.3, 10' 2" East of LL, Aux. Bldg., Elev. 94' 7"	A	No	No

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4b provided that a revision to Table 3.7-4b is included with the next License Amendment request.

** Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-4b is included with the next License Amendment request.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 10, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 5, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 2, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 5 TO FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 7-23	3/4 7-23
3/4 7-25	3/4 7-25
3/4 7-29	3/4 7-29
3/4 7-30	3/4 7-30
--------	3/4 7-30a through 3/4 7-30l

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All snubbers listed in Tables 3.7-4a and 3.7-4b shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours, replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9c on the supported component or declare the supported system inoperable and follow appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspection

The first inservice visual inspection of snubbers shall be performed after four months but within 10 months of POWER OPERATION and shall include all snubbers listed in Tables 3.7-4a and 3.7-4b. If less than two (2) snubbers in the same Table are found inoperable during the first inservice visual inspection, the second inservice visual inspection for snubbers in that Table shall be performed 12 months ± 25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed for each separate Table in accordance with the following schedule:

No. Inoperable Snubbers in a Table per Inspection Period	Subsequent Inspection Period for each Table*#
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3, 4	124 days ± 25%
5, 6, 7	62 days ± 25%
8 or more	31 days ± 25%

Within each Table, the snubbers may be categorized into two groups: Those accessible and those inaccessible during reactor operation. Each group within a Table may be inspected independently in accordance with the above schedule.

*The inspection interval shall not be lenghthened more than one step at a time.

#The provisions of Specification 4.0.2 are not applicable.

and 3.7-4b may be used jointly or separately as the basis for the sampling plan.

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

d. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

e. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force. The drag force shall not have increased more than 50% since the last functional test.

2. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
17.	No. 21 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
18.	No. 21 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
19.	No. 22 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
20.	No. 22 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
21.	No. 23 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
22.	No. 23 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
23.	No. 24 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes
24.	No. 24 Main Steam Isolation Valve Penetration Area, Elev. 115' 5"	A	No	Yes

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4a provided that a revision to Table 3.7-4a is included with the next License Amendment request.

** Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-4a is included with the next License Amendment request.

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
1	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 141' 3"	I	Yes	No
2	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 141' 3"	I	Yes	No
3	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 141' 3"	I	Yes	No
4	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 148' 6"	I	Yes	No
5	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev.147' 0"	I	Yes	No
6	Reactor Coolant System, Pressurizer N.E. Quadrant, Reactor Containment Elev. 147' 0"	I	Yes	No
7	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 147' 0"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
8	Reactor Coolant System, Pressurizer S.W. Quadrant at valve 2PR-1, Reactor Containment, Elev. 152' 1 7/8"	I	Yes	No
9	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 145' 5 1/4 "	I	Yes	No
10	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 143' 0"	I	Yes	No
11	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 143' 0"	I	Yes	No
12	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 148' 0"	I	Yes	No
13	Reactor Coolant System, Pressurizer S.E. Quadrant at valve 2PR-47, Reactor Containment, Elev. 156' 8"	I	Yes	No
14	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 156' 8"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
15	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 145' 0"	I	Yes	No
16	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 145' 0"	I	Yes	No
17	Reactor Coolant System, Pressurizer S.E. Quadrant, Reactor Containment Elev. 145' 0"	I	Yes	No
18	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 156' 8"	I	Yes	No
19	Reactor Coolant System, Pressurizer N.W. Quadrant, Reactor Containment Elev. 141' 0"	I	Yes	No
20	Reactor Coolant System, Pressurizer S.W. Quadrant, Reactor Containment Elev. 148' 0"	I	Yes	No
21	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 94' 10 5/8"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
22	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 94' 10 5/8"	I	Yes	No
23	Reactor Coolant System, Pressurizer Support Frame, Reactor Containment Elev. 92' 8 5/8"	I	Yes	No
24	Reactor Coolant System, Annulus Bet. B9-3 & B9-4, Reactor Containment Elev. 92' 5 5/16"	I	No	No
25	Reactor Coolant System, Annulus Bet. B9-3 & B9-4, Reactor Containment Elev. 92' 5 5/16 "	I	No	No
26	Reactor Coolant System, Annulus Bet. B8-3 & B8-4, Reactor Containment Elev. 92' 6 3/8"	I	No	No
27	Reactor Coolant System, Annulus Bet. B8-3 & B8-4, Reactor Containment Elev. 92' 6 3/8"	I	No	No
28	CVC System, East Side of #22 RCP PLTFM & Wall, Reactor Containment Elev. 102' 3 1/4 "	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
29	CC System, N.E. Corner of #24 RCP Platform, Reactor Containment Elev. 101' 9 1/4"	I	Yes	No
30	CC System, Bet. West Side of #24 RCP Platform & Wall, Reactor Containment, Elev. 90' 6"	I	Yes	No
31	RHR & SI System, Annulus @ B8-3, Reactor Containment, Elev. 85' 9"	I	No	No
32	RHR & SI System, Annulus @ B6, Reactor Containment, Elev. 80' 9"	I	No	No
33	RHR & SI System, Bet. #23 Stm. Gen. Fr. & #23 RCP Fr. Reactor Containment Elev. 94' 4"	I	Yes	No
34	RHR & SI System, Bet. S.W. Side of #24 RCP Fr. & Wall, Reactor Containment Elev. 88' 7"	I	Yes	No
35	RHR & SI System, Bet. #24 Stm. Gen. Frame & Crane Wall, Reactor Containment Elev. 97' 11 1/8"	I	Yes	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
36	RHR & SI System, S.E. Corner of #23 Stm. Gen. Fr., Reactor Containment, Elev. 93' 6"	I	Yes	No
37	RHR & SI System, Bet. So. Side of Press. Frame & Wall, Reactor Containment, Elev. 93' 6"	I	Yes	No
38	Stm Gen. Drns.& Bldn, 6" North of 17.6, 19 1/2" West of JJ, S. Pen. Area, Elev. 108' 7"	A	No	No
39	Stm. Gen. Drns. & Bldn, 4' 1" North of 17.6, 6' 6" West of JJ, S. Pen. Area, Elev. 108' 7"	A	No	No
40	Stm. Gen. Drns. & Bldn., Annulus 9" West of B16-2, Reactor Containment Elev. 90' 6 1/2 "	I	No	No
41	Stm. Gen. Drns. & Bldn., S.E. Corner of #23 Stm. Gen. Fr., Reactor Containment, Elev. 99' 6"	I	Yes	No
42	Containment Spray, 2' 6" North of 16.2, 6' 2" West of GG, S. Pen. Area, Elev. 88' 3"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
43	St. Gen. Fdwtr., 23' 4" No. of 16.2 6' 6" West of DC, S. Pen. Area, Elev. 128' 10"	A	No	No
44	St. Gen. Fdwtr., 23' 4" No. of 16.2 6' 6" West of DC, S. Pen. Area, Elev. 128' 10"	A	No	No
45	St. Gen. Fdwtr., 23' 4" No. of 16.2, 2' 0" East of DC, S. Pen. Area, Elev. 130' 6"	A	No	No
46	St. Gen. Fdwtr., 23' 4" No. of 16.2, 2' 0" East of DC, S. Pen. Area, Elev. 130' 6"	A	No	No
47	St. Gen. Fdwtr., Near East Wall, N. Pen. Area, Elev. 128' 10"	A	No	No
48	St. Gen. Fdwtr., Near East Wall, N. Pen. Area, Elev. 128' 10"	A	No	No
49	St. Gen. Fdwtr,. Outside East Wall, N. Pen. Area, Elev. 130' 6"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
50	St. Gen. Fdwtr., 8' 4" No. of 16.2 6' 6" West of DC, S. Pen. Area, Elev. 108' 3"	A	No	No
51	St. Gen. Fdwtr., 8' 4" No. of 16.2, 6' 6" West of DC, S. Pen. Area, Elev. 108' 9 1/8"	A	No	No
52	St. Gen. Fdwtr., 8' 4" No. of 16.2, 6' 6" West of DC, S. Pen Area, Elev. 108' 9 1/8"	A	No	No
53	St. Gen. Fdwtr., 8' 4" No. of 16.2, 6' 6" West of DC, S. Pen. Area, Elev. 131' 2"	A	No	No
54	St. Gen. Fdwtr., 8' 4" No. of 16.2, 6' 6" West of DC, S. Pen. Area, Elev. 131' 2"	A	No	No
55	St. Gen. Fdwtr., N.E. Corner, N. Pen. Area, Elev. 108' 3"	A	No	No
56	St. Gen. Fdwtr., N.E. Corner, N. Pen. Area, Elev. 108' 9 1/8"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
57	St. Gen. Fdwtr., N.E. Corner, N. Pen. Area, Elev. 131' 2"	A	No	No
58	St. Gen. Fdwtr., N.E. Corner, N. Pen. Area, Elev. 131' 2"	A	No	No
59	Main St. Drn., 13' 4" No. of 16.2, 13' 6 3/4" West of DC, S. Pen. Area, Elev. 102' 1"	A	No	No
60	Main St. Drn., 27' 7" No. of 16.2, 8' 10" West of DC, S. Pen. Area, Elev. 112' 0"	A	No	No
61	Main St. Drn., 27' 7" No. of 16.2, 9' 8 3/4" West of DC, S. Pen. Area, Elev. 112' 0"	A	No	No
62	Main St. Drn., 24' 1" No. of 16.2, 9' 4 3/4" West of DC, S. Pen. Area, Elev. 103' 3"	A	No	No
63	RHR & SI System, 17' 10" No. of 14,5' 6 1/4" East of RR, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
64	RHR & SI System, 17' 10" No. of 14 5' 6 1/4" East of RR, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
65	RHR & SI System, 7' 9" No. of 14,4' 6" East of LL, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
66	RHR & SI System, 7' 9" No. of 14, 2' 10" East of LL, Aux. Bldg., Elev. 94' 7 1/2"	A	No	No
67	RHR & SI System, 7' 7 7/8" So. of 17.6, 7' 0" East of HH, S. Pen. Area, Elev. 87' 1"	A	No	No
68	RHR & SI System, 7' 7 7/8" So. of 17.6, 7' 0" East of HH, S. Pen. Area, Elev. 86' 2 1/4"	A	No	No
69	RHR & SI System, 7' 7 7/8" So. of 17.6, 6' 3" East of HH, S. Pen. Area, Elev. 85' 4 1/4"	A	No	No
70	RHR & SI System, 7' 7 7/8" So. of 17.6, 6' 3" East of HH, S. Pen. Area, Elev. 85' 0 3/4"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
71	RHR & SI System, 8' 0 1/2" No. of 16.2, 7' 5 3/16" East of HH, S. Pen. Area, Elev. 79' 6"	A	No	No
72	RHR & SI System, 8' 0 1/2", No. of 16.2 7' 5 3/16" East of HH, S. Pen. Area, Elev. 79' 6"	A	No	No
73	RHR & SI System, 8' 9" No. of 14.8, 9' 10" East of SS, Aux. Bldg., Elev. 86' 1"	A	No	No
74	RHR & SI System, 8' 9" No. of 14.8, 9' 10" East of SS, Aux. Bldg., Elev. 86' 1"	A	No	No
75	RHR & SI System, 5' 5 1/2" So. of 14.8, 1" West of HH, Aux. Bldg., Elev. 78' 0"	A	No	No
76	M.S. To Aux. FWP. Exhaust, 4' 0" So. of 16.2, 12' 0" West of KK, Aux. Bldg., Elev. 97' 6"	A	No	No
77	M.S. To Aux. FWP. Exhaust, 7 1/16" North of 15.7, 6' 3" East of LL, Aux. Bldg., Elev. 92' 0"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS *

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
78	M. S. To Aux. FWP. Exhaust, 7 1/16" North of 15.7, 6' 3" East of LL, Aux. Bldg., Elev. 92' 0"	A	No	No
79	M.S. To Aux. FWP. Exhaust, 7 1/16" North of 15.7, 3' 8" East of LL, Aux. Bldg., Elev. 89' 6"	A	No	No
80	M.S. To Aux. FWP. Exhaust, 7 1/16" North of 15.7, 3' 8" East of LL, Aux. Bldg., Elev. 88' 6"	A	No	No
81	M.S. To Aux. FWP. Exhaust, 7 1/16" North of 15.7, 3' 8" East of LL, Aux. Bldg., Elev. 88' 6"	A	No	No
82	M.S. To Aux. FWP. Exhaust, 11 11/16" So. of 15.7, 8' 7 5/8" East of LL, Aux. Bldg., Elev. 92' 0"	A	No	No
83	M.S. To Aux. FWP. Exhaust, 11 11/16" So. of 15.7, 5' 3 5/8" East of LL, Aux. Bldg., Elev. 92' 0"	A	No	No
84	CVC System, 8' 4" So. of 15.7, 3' 5" West of NN, Aux. Bldg., Elev. 101' 11"	A	No	No

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION DURING SHUTDOWN** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
85	CC System, 6' 11" So. of 15.7 10' 9" East of GG, Aux. Bldg., Elev. 96' 0"	A	No	No
86	CC System, 6' 11" So. of 15.7 10' 9" East of GG, Aux. Bldg., Elev. 96' 0"	A	No	No
87				
88	DELETED-REPLACED WITH RIGID STRUTS			
89				
90				
91	Spent Fuel Cooling, 4' 6" No. of 15.7, 5' 1/2" West of JJ, Aux. Bldg., Elev. 92' 3"	A	No	No

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4b provided that a revision to Table 3.7-4b is included with the next License Amendment request.

** Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-4b is included with the next License Amendment request.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-70

AND AMENDMENT NO. 5 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNITS NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

Introduction

By letter dated November 20, 1980, the NRC informed all Power Reactor Licensees (except SEP Licensees) of the staff's revised position on inservice surveillance requirements for snubbers. Attached to that letter was guidance on the content of Technical Specifications (TSs) which the staff felt appropriate to provide assurance of the operability of snubbers (both mechanical and hydraulic) during plant operation. This guidance included:

1) Addition of mechanical snubbers to the surveillance program;

2) Deletion of 50,000 lb. capacity limit on snubbers to be included on inservice testing program;

3) Deletion of requirement that seal material receive NRC approval;

4) Clarification of inservice test requirements; and

5) Provision for in-place inservice testing.

Evaluation

By letter dated August 10, 1981, Public Service Electric and Gas Company (PSE&G) responded to the November 20, 1980 NRC request and applied for changes to the Salem Units 1 and 2 Technical Specifications (TSs).

The Salem Unit 2 TSs were issued with the full power license on May 20, 1981. These TSs paralleled the guidelines of the November 20, 1980 NRC letter with the exception that a table of mechanical snubbers was not included. PSE&G was required, by license condition, to provide a complete listing of mechanical snubbers within four months of issuance of the full power license. PSE&G's amendment request of August 10, 1981 included the required listing and some minor editorial changes for clarification. The requested changes are in accordance with the staff's guidelines and, therefore, acceptable.

The proposed changes to the Salem Unit 1 TSs for snubbers provide for Limiting Conditions for Operation and Surveillance Requirements identical to those approved for Salem Unit 2 and, therefore, are acceptable. Additionally, PSE&G has requested that the functional testing portion of snubber technical specification be delayed until the 4th Refueling Outage for Unit 1. Salem Unit 1 is currently shutdown for its 3rd Refueling Outage with an estimated startup date of March 14, 1982. Its 4th Refueling Outage is scheduled for October 1982. We have reviewed PSE&G's request for delay and agree that the added functional testing requirements would be difficult to satisfy during the current refueling outage. The work scheduling difficulties are compounded by the limited availability of commercial test equipment. The requested delay would provide the time needed for the effective implementation of these requirements and, therefore, is acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

Conclusion

We have concluded, based on the considerations discussed above, that (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 2, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-272 AND 50-311

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 40 to Facility Operating License No. DPR-70 and Amendment No. 5 to Facility Operating License No. DPR-75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (the facilities) located in Salem County, New Jersey. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications related to hydraulic and mechanical snubbers. In particular, the principle changes involve the addition of mechanical snubbers to the snubber surveillance program and provide clarification of the inservice test requirements.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated August 10, 1981, (2) Amendment Nos. 40 and 5 to License Nos. DPR-70 and DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 2nd day of February, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing