

April 19, 1982

Docket Nos. 50-272  
and 50-311

Mr. Richard A. Uderitz  
Vice President - Nuclear  
Public Service Electric and Gas Company  
Mail Code T15A  
P. O. Box 570  
Newark, New Jersey 07101

Dear Mr. Uderitz:

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OPA (Clare Miles)

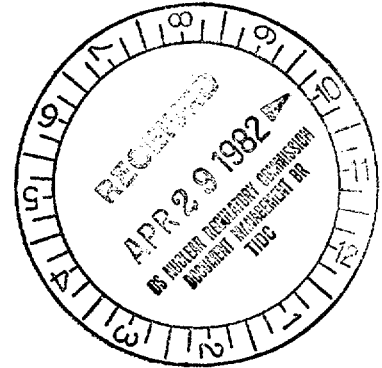
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The Commission has issued the enclosed Amendment No. 41 to Facility Operating License No. DPR-70 and Amendment No. 7 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated January 15, 1982 and supplemented by your letter of February 3, 1982.

These amendments revise the Radiological Technical Specifications to provide better control and surveillance of containment ventilation valves.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

W. J. Ross, Project Manager  
Operating Reactors Branch No. 1  
Division of Licensing

**Enclosures:**

1. Amendment No. 41 to DPR-70
2. Amendment No. 7 to DPR-75
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:

See next page

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FR NOTICE  
& AMENDMENT  
NRC  
LEAD PM 8-24 PM

OFFICE	ORB 1	ORB 1	ORB 1	AD:OR	OELD	ORB 1	
SURNAME	CParrish	WRoss/rs	SVanga	Novak	M. KARMA	E. REEVES	
DATE	3/30/82	3/30/82	4/7/82	4/8/82	4/13/82	4/6/82	

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
PHILADELPHIA ELECTRIC COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. <sup>41</sup>7  
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated January 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

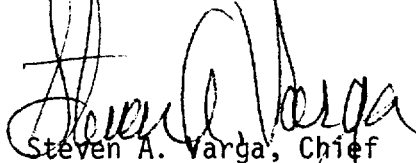
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 41, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 19, 1982



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
PHILADELPHIA ELECTRIC COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7  
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated January 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

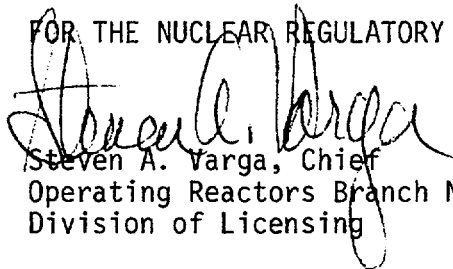
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 7, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 19, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 41

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 6-8a  
3/4 6-13  
3/4 6-16

Insert Pages

3/4 6-8a  
3/4 6-13  
3/4 6-16

## CONTAINMENT SYSTEMS

### CONTAINMENT VENTILATION SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.6.1.7 The containment purge supply and exhaust isolation valves\* shall be closed. (Valves immobilized in shut position with control air to valve operators isolated and tagged out of service).

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With one containment-purge supply and/or exhaust isolation valve open, close the open valve(s) within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.6.1.7 The containment purge supply and exhaust isolation valves shall be determined closed at least once per 31 days.

\*The containment pressure-vacuum relief isolation valves may be opened on an intermittent basis, under administrative control, as necessary to satisfy the requirement of Specification 3.6.1.4.



## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.
- c. Verifying that on a feedwater isolation test signal, each feedwater isolation valve actuates to its isolation position.
- d. Verifying that on a Containment Purge and Pressure-Vacuum Relief isolation test signal, each Purge and Pressure-Vacuum Relief valve actuates to its isolation position.

4.6.3.1.3 At least once per 18 month, verify that on a main steam isolation test signal, each main steam isolation valve specified in Table 3.6-1 actuates to its isolation position.

4.6.3.1.4 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.1.5 Each containment purge isolation valve shall be demonstrated OPERABLE within 24 hours after each closing of the valve, except when the valve is being used for multiple cyclings, then at least once per 72 hours, by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Specification 4.6.1.2d. for all other Type B and C penetrations, the combined leakage rate is less than or equal to  $0.60L_a$ .

4.6.3.1.6 A pressure drop test to identify excessive degradation of resilient valve seals shall be conducted on the:

- a. Containment Purge Supply and Exhaust Isolation Valves at least once per 6 months.
- b. Containment Pressure - Vacuum Relief Isolation Valves at least once per 3 months.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME
<b>C. MAIN STEAM ISOLATION</b>		
1. 11 MS 7#	Main Steam Drain	< 10 Sec.
2. 12 MS 7#	Main Steam Drain	< 10 Sec.
3. 13 MS 7#	Main Steam Drain	< 10 Sec.
4. 14 MS 7#	Main Steam Drain	< 10 Sec.
5. 11 MS 18#	Main Steam Bypass	< 10 Sec.
6. 12 MS 18#	Main Steam Bypass	< 10 Sec.
7. 13 MS 18#	Main Steam Bypass	< 10 Sec.
8. 14 MS 18#	Main Steam Bypass	< 10 Sec.
<b>D. FEEDWATER ISOLATION</b>		
1. 11 BF 19#	Main Feedwater Isolation	< 8 Sec.
2. 12 BF 19#	Main Feedwater Isolation	< 8 Sec.
3. 13 BF 19#	Main Feedwater Isolation	< 8 Sec.
4. 14 BF 19#	Main Feedwater Isolation	< 8 Sec.
5. 11 BF 40#	Main Feedwater Isolation	< 8 Sec.
6. 12 BF 40#	Main Feedwater Isolation	< 8 Sec.
7. 13 BF 40#	Main Feedwater Isolation	< 8 Sec.
8. 14 BF 40#	Main Feedwater Isolation	< 8 Sec.
<b>E. CONTAINMENT PURGE AND PRESSURE-VACUUM RELIEF</b>		
1. 1 VC 1	Purge Supply	< 2 Sec.
2. 1 VC 2#	Purge Supply	< 2 Sec.
3. 1 VC 3#	Purge Exhaust	< 2 Sec.
4. 1 VC 4	Purge Exhaust	< 2 Sec.
5. 1 VC 5*	Pressure-Vacuum Relief	< 2 Sec.
6. 1 VC 6#*	Pressure-Vacuum Relief	< 2 Sec.

ATTACHMENT TO LICENSE AMENDMENT NO. 7

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

3/4 6-9  
3/4 6-15

Insert Pages

3/4 6-9  
3/4 6-15

## CONTAINMENT SYSTEMS

### CONTAINMENT VENTILATION SYSTEM

#### LIMITING CONDITION FOR OPERATION

---

3.6.1.7 The containment purge supply and exhaust isolation valves\* shall be closed. (Valves immobilized in shut position with control air to valve operators isolated and tagged out of service).

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With one containment-purge supply and/or exhaust isolation valve open, close the open valve(s) within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.6.1.7 The containment purge supply and exhaust isolation valves shall be determined closed at least once per 31 days.

\*The containment pressure-vacuum relief isolation valves may be opened on an intermittent basis, under administrative control, as necessary to satisfy the requirement of Specification 3.6.1.4.

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.
- c. Verifying that on a feedwater isolation test signal, each feedwater isolation valve actuates to its isolation position.
- d. Verifying that on a Containment Purge and Pressure-Vacuum Relief isolation test signal, each Purge and Pressure-Vacuum Relief valve actuates to its isolation position.

4.6.3.3 At least once per 18 month, verify that on a main steam isolation test signal, each main steam isolation valve specified in Table 3.6-1 actuates to its isolation position.

4.6.3.4 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.5 Each containment purge isolation valve shall be demonstrated OPERABLE within 24 hours after each closing of the valve, except when the valve is being used for multiple cyclings, then at least once per 72 hours, by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Specification 4.6.1.2d. for all other Type B and C penetrations, the combined leakage rate is less than or equal to  $0.60L_a$ .

4.6.3.6 A pressure drop test to identify excessive degradation of resilient valve seals shall be conducted on the :

- a. Containment Purge Supply and Exhaust Isolation Valves at least once per 6 months.
- b. Containment Pressure - Vacuum Relief Isolation Valves at least once per 3 months.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. DPR-70  
AND AMENDMENT NO. 7 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,  
PHILADELPHIA ELECTRIC COMPANY,  
DELMARVA POWER AND LIGHT COMPANY, AND  
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

Introduction

In a letter of November 6, 1981, we informed Public Service Electric and Gas Company (the licensee) of the status of our review of the purge and vent system at Salem Unit No. 1. This review covers several concerns that were initially identified in our letter of November 28, 1978, as well as subsequent concerns that were incorporated into Item II.E.4.2 of the TMI Action Plan (NUREG-0737). The licensee's responses dated December 22, 1981, January 15, 1982 and February 3, 1982 have enabled us to close out all of the items of our long-term review for both Unit Nos. 1 and 2 except for demonstration of the operability of the Pressure-Vacuum Relief System. These responses also have enabled us to approve the licensee's commitment to seal closed unqualified valves as required by sub-item number 6 of Item II.E.4.2. This was the only unresolved issue in this Action Item.

Evaluation

A. Conformance to Standard Review Plan Section 6.2.4 Revision 1 and  
Branch Technical Position CSB 6-7 Revision 1

Our Safety Evaluation on this subject was transmitted with our November 6, 1981 letter, with four open items. Those items have been satisfied as follows:

1. The licensee in its letter of December 22, 1981 confirmed that the debris screens installed at Unit No. 1 are identical to those that were approved by the staff for Unit No. 2 in Supplement 4 to the Salem Unit No. 2 SER.
2. The licensee in its letter of December 22, 1981, committed to limit the use of the Pressure-Vacuum Relief System on Unit No. 1 to that required for safety related reasons and to a limit of 1000 hours per year. The same commitment was made for Unit 2 in a letter of February 3, 1982.

3. The licensee, in its letter of January 15, 1982, proposed revisions in the Technical Specifications for Unit Nos. 1 and 2 to immobilize the supply and exhaust valves in Modes 1, 2, 3 and 4. The staff agrees that immobility action of valves by isolating the control air to the valve operators and by tagging the valves out of service is equivalent to "locking closed" these valves. This completes action on Requirement 6 of Item II.E.4.2 in NUREG-0737. All of the requirements of this Action Item have now been met for both units.
4. The licensee, in their letter of January 15, 1982, proposed Surveillance Technical Specifications for Unit Nos. 1 and 2 to test the leakage integrity of the Purge and Exhaust Valves as well as the Vacuum-Relief Valves at least once per six months. This requirement meets the intent of Branch Technical Position (BTP) CSB 6-4 and is acceptable for both units.

B. Safety Activation Signal Override

Our Safety Evaluation Report for this subject was enclosed with our letter of November 6, 1981. This SER contained one open item; i.e., the need for adequate physical protection for the containment ventilation isolation reset push button switches located in the control room. In their letter of December 22, 1981 the licensee described the protective system installed at Unit No. 1. This system consists of plexiglass covers that have been placed over the pushbutton bezels on the main control console. These covers contain holes over pushbuttons for functions other than reset of the containment ventilation isolation to allow the operator to depress other buttons on the bezel. These protective devices are acceptable.

- C. Proposed Revision 1 to Regulatory Guide 1.141 states that the radiation monitors that initiate containment ventilation isolation shall be safety grade. As stated in our letter of November 6, 1981, the licensee need not commit to this action until Revision 1 is approved.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 19, 1982



UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 20-272 AND 50-311PUBLIC SERVICE ELECTRIC AND GAS COMPANY,  
PHILADELPHIA ELECTRIC COMPANY,  
DELMARVA POWER AND LIGHT COMPANY, AND  
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 41 to Facility Operating License No. DPR-70 and Amendment No. 7 to Facility Operating License No. 75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generation Station, Unit Nos. 1 and 2 (the facilities) located in Salem County, New Jersey. The amendments are effective as of the date of issuance.

The amendments revise the Radiological Technical Specifications to provide better control and surveillance of containment ventilation valves in both Unit No. 1 and Unit No. 2.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

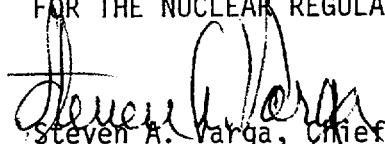
- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated January 15, 1982, (2) Amendment Nos. 41 and 7 to License Nos. DPR-70 and DPR-75, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of April 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing