

OCT 18 1982

Docket Nos. 50-272
50-311

Mr. Richard A. Uderitz
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Public Service Electric and Gas Company
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Dear Mr. Uderitz:

The Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. DPR-70 and Amendment No. 13 to Facility Operating License No. DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Appendices A and B Technical Specifications in response to your application transmitted by letter dated August 10, 1981 and modified by letter dated June 22, 1982.

These amendments revise the Radiological Safety and Environmental Technical Specifications related to process and effluent monitors to agree with existing plant conditions.

Copies of the Safety Evaluation, the Environmental Impact Appraisal and the Notice of Issuance and Negative Declaration are also enclosed.

Sincerely,

ORIGINAL SIGNED

William J. Ross, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 48 to DPR-70
2. Amendment No. 13 to DPR-75
3. Safety Evaluation
4. Environmental Impact Appraisal
5. Notice of Issuance/Negative Declaration

cc w/encs:
See next page

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FR NOTICE
& AMENDMENT

OFFICE	DL:ORB#1	DL:ORB#1	DL:ORB#1	DRP	OELD		
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DATE	10/18/82	10/14/82	10/12/82	10/5/82	10/5/82		

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 10, 1981 and modified by letter dated June 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

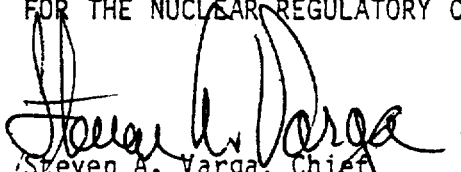
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 18, 1982

ATTACHMENT TO LICENSE AMENDMENT NO.48

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 3-18

3/4 3-36

Insert Pages

3/4 3-18

3/4 3-36

Revise Appendix B as follows:

Remove Pages

2.3-21

2.3-23

Insert Pages

2.3-21

2.3-23

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
b. Phase "B" Isolation					
1) Manual	2 sets of 2	1 set of 2	2 sets of 2	1, 2, 3, 4	18
2) Automatic Actuation Logic	2	1	2	1, 2, 3, 4	13
3) Containment Pressure--High-High	4	2	3	1, 2, 3	16
c. Purge and Exhaust Isolation					
1) Manual	2	1	2	1, 2, 3, 4	17
2) Containment Atmo- sphere Radioactivity- High	3	1	2**	1, 2, 3, 4	17
4. STEAM LINE ISOLATION					
a. Manual	1/steam line	1/steam line	1/operating steam line	1, 2, 3	18
b. Automatic Actuation Logic	2	1	2	1, 2, 3	13
c. Containment Pressure-- High-High	4	2	3	1, 2, 3	16

**

The unit vent sampling monitor may also function in this capacity, with lowered setpoints, when the purge/pressure-vacuum relief isolation valves are open.

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area	1	*	≤ 15 mR/hr	$10^{-1} - 10^4$ mR/hr	19
2. PROCESS MONITORS					
a. Containment					
1) Gaseous Activity	1#				
a) Purge & Pressure- Vacuum Relief Isolation		1, 2, 3, 4 & 6	≤ 2 x background	$10^1 - 10^6$ cpm	22
b) RCS Leakage Detection		1, 2, 3 & 4	N/A	$10^1 - 10^6$ cpm	20
2) Air Particulate Activity	1#				
a) Purge & Pressure- Vacuum Relief Isolation		1, 2, 3, 4 & 6	≤ 2 x background	$10^1 - 10^6$ cpm	22
b) RCS Leakage Detection		1, 2, 3 & 4	N/A	$10^1 - 10^6$ cpm	20
3) Fixed Filter Iodine- Purge & Pressure - Vacuum Relief Isolation	1#	1, 2, 3, 4 & 6	≤ 2 x background	$10^1 - 10^6$ cpm	22

* With fuel in the storage pool or building.

The unit vent sampling monitor may also function in this capacity, with lowered setpoints, when the purge/purge-pressure-vacuum relief isolation valves are open.

TABLE 2.3-2

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS

Gaseous Source	Sampling Frequency and Analysis	Type of Activity Analysis	Detectable Concentrations (uCi/ml) ^a
A. Waste Gas Decay Tank Releases	Each Tank to be Released	Principal Gamma Emitters	10^{-4}^b
		H-3	10^{-6}
B. Containment Purge Releases	Each Purge ^c	Principal Gamma Emitters	10^{-4}^c
		H-3	10^{-6}
C. Condenser Air Ejector	Monthly	Principal Gamma Emitters	$10^{-4}^{b,c}$
		H-3	10^{-6}
D. Environmental Release Points	Monthly (Gas Samples)	Principal Gamma Emitters	$10^{-4}^{b,c}$
		H-3	10^{-6}
	Weekly (Charcoal Sample)	I-131	10^{-12}
	Monthly (Charcoal Sample)	I-133, I-135	10^{-10}
	Weekly (Particulates) ^d	Principal Gamma Emitters (Ba-La-140, I-131 and others)	10^{-11}
	Monthly Composite ^d (Particulates)		
		Gross α	10^{-11}
	Quarterly Composite ^d (Particulates)	Sr-89, Sr-90	10^{-11}

^a The above detectability limits for activity analysis are based on technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides, lower detection limits may be readily achievable, and when nuclides are measured below the stated limits, they should also be reported.

^b For certain mixtures of gamma emitters, it may not be possible to measure radionuclides at levels near their sensitivity limits when other nuclides are present in the sample at much higher levels. Under these circumstances, it will be more appropriate to calculate the levels of such radionuclides using observed ratios with those radionuclides which are measurable.

^c Analyses shall also be performed following each refueling, startup, or similar operational occurrence which could alter the mixture of radionuclides.

^d To be representative of the average quantities and concentrations of radioactive materials in particulate form released in gaseous effluents, samples should be collected in proportion to the rate of flow of the effluent stream.

^e Not applicable to Pressure-Vacuum Relief operations

TABLE 2.3-4

SALEM STATION GASEOUS WASTE SYSTEM
LOCATION OF PROCESS AND EFFLUENT MONITORS AND SAMPLERS REQUIRED BY TECHNICAL SPECIFICATIONS

Process Stream or Release Point	Radiation Alarm	Auto Control to Isolation Valve	Continuous Monitor	Grab Sample Station	Measurement				
					Noble Gas	I	Particulate	H-3	Alpha
Waste Gas Decay Tanks				X	X	X	X	X	X
Condenser Air Removal System	X		X	X	X	X	X	X	X
Plant Vent	X	X ^b	X	X	X	X	X	X	X
Containment Purge/Pressure - Vacuum Relief				X	X	X	X	X	X
Building Ventilation Systems									
Auxiliary Building and Radwaste Area ^a				X ^c	X	X	X	X	X
Fuel Handling & Storage Building ^a				X ^c	X	X	X	X	X
Turbine Gland Seal Condenser ^a				X ^c	X	X	X	X	X

^a Since these process streams or building ventilation systems are routed to the plant vent, the need for a continuous monitor at the individual discharge point to the main exhaust duct is eliminated. One continuous monitor at the final release point is sufficient.

^b Automatically isolates Waste Gas Discharge Line on HIGH RADIATION signal from Gaseous Monitor, R41C. This monitor (R41C) along with R41A and R41B will initiate isolation of Containment Purge/Pressure-Vacuum Relief System on HIGH RADIATION also.

^c Grab sample stations from which monthly gas samples (Table 2.3-2) are to be taken. Also, grab samples should be taken and measured to determine the process stream or building ventilation system source whenever an unexplained increase is indicated by the plant vent sampler-monitors.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 10, 1981 and modified by letter dated June 22, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 13, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 18, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 13

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages

3/4 3-18

3/4 3-39

Insert Pages

3/4 3-18

3/4 3-39

Revise Appendix B as follows:

Remove Pages

2.3-21

2.3-23

Insert Pages

2.3-21

2.3-23

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
b. Phase "B" Isolation					
1) Manual	2 sets of 2	1 set of 2	2 sets of 2	1, 2, 3, 4	18
2) Automatic Actuation Logic	2	1	2	1, 2, 3, 4	13
3) Containment Pressure--High-High	4	2	3	1, 2, 3	16
c. Containment Ventilation Isolation					
1) Manual	2	1	2	1, 2, 3, 4	17
2) Automatic Actuation Logic	2	1	2	1, 2, 3, 4	13
3) Containment Atmosphere Gaseous Radioactivity-High	4**	1	1	1, 2, 3, 4	17
4. STEAM LINE ISOLATION					
a. Manual	2/steam line	1/steam line	1/operating steam line	1, 2, 3	21
b. Automatic Actuation Logic	2	1	2	1, 2, 3	20
c. Containment Pressure--High-High	4	2	3	1, 2, 3	16

** The unit vent sampling monitor may also function in this capacity, with lowered setpoints, when the purge/pressure-vacuum relief isolation valves are open.

TABLE 3.3-6
RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area	1	*	≤ 15 mR/hr	$10^{-1} - 10^4$ mR/hr	23
2. PROCESS MONITORS					
a. Containment					
1) Gaseous Activity					
a) Purge & Pressure- Vacuum Relief Isolation	1**	1, 2, 3, 4 & 6	$\leq 4.5 \times 10^{-2}$ Ci/Sec	$10^1 - 10^6$ cpm	25
b) RCS Leakage Detection	1	1, 2, 3 & 4	N/A	$10^1 - 10^6$ cpm	24
2) Particulate Activity - 1 RCS Leakage Detection	1	1, 2, 3 & 4	N/A	$10^1 - 10^6$ cpm	24

* With fuel in the storage pool or building.

** The unit vent sampling monitor may also function in this capacity, with lowered setpoints, when the purge/pressure-vacuum relief isolation valves are open.

TABLE 2.3-2

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS

Gaseous Source	Sampling Frequency and Analysis	Type of Activity Analysis	Detectable Concentrations ($\mu\text{Ci}/\text{ml}$) ^a
A. Waste Gas Decay Tank Releases	Each Tank to be Released	Principal Gamma Emitters	10^{-4b}
		H-3	10^{-6}
B. Containment Purge Releases	Each Purge ^c	Principal Gamma Emitters	10^{-4c}
		H-3	10^{-6}
C. Condenser Air Ejector	Monthly	Principal Gamma Emitters	$10^{-4b,c}$
		H-3	10^{-6}
D. Environmental Release Points	Monthly (Gas Samples)	Principal Gamma Emitters	$10^{-4b,c}$
		H-3	10^{-6}
	Weekly (Charcoal Sample)	I-131	10^{-12}
	Monthly (Charcoal Sample)	I-133, I-135	10^{-10}
	Weekly (Particulates) ^d	Principal Gamma Emitters (Ba-La-140, I-131 and others)	10^{-11}
	Monthly Composite ^d (Particulates)		
		Gross α	10^{-11}
	Quarterly Composite ^d (Particulates)	Sr-89, Sr-90	10^{-11}

^a The above detectability limits for activity analysis are based on technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides, lower detection limits may be readily achievable, and when nuclides are measured below the stated limits, they should also be reported.

^b For certain mixtures of gamma emitters, it may not be possible to measure radionuclides at levels near their sensitivity limits when other nuclides are present in the sample at much higher levels. Under these circumstances, it will be more appropriate to calculate the levels of such radionuclides using observed ratios with those radionuclides which are measurable.

^c Analyses shall also be performed following each refueling, startup, or similar operational occurrence which could alter the mixture of radionuclides.

^d To be representative of the average quantities and concentrations of radioactive materials in particulate form released in gaseous effluents, samples should be collected in proportion to the rate of flow of the effluent stream.

^e Not applicable to Pressure-Vacuum Relief operations

TABLE 2.3-4

SALEM STATION GASEOUS WASTE SYSTEM
LOCATION OF PROCESS AND EFFLUENT MONITORS AND SAMPLERS REQUIRED BY TECHNICAL SPECIFICATIONS

Process Stream or Release Point	Radiation Alarm	Auto Control to Isolation Valve	Continuous Monitor	Grab Sample Station	Measurement				
					Noble Gas	I	Particulate	H-3	Alpha
Waste Gas Decay Tanks				X	X	X	X	X	X
Condenser Air Removal System	X		X	X	X	X	X	X	X
Plant Vent	X	X ^b	X	X	X	X	X	X	X
Containment Purge/Pressure - Vacuum Relief				X	X	X	X	X	X
Building Ventilation Systems									
Auxiliary Building and Radwaste Area ^a				X ^c	X	X	X	X	X
Fuel Handling & Storage Building ^a				X ^c	X	X	X	X	X
Turbine Gland Seal Condenser ^a				X ^c	X	X	X	X	X

^a Since these process streams or building ventilation systems are routed to the plant vent, the need for a continuous monitor at the individual discharge point to the main exhaust duct is eliminated. One continuous monitor at the final release point is sufficient.

^b Automatically isolates Waste Gas Discharge Line on HIGH RADIATION signal from Gaseous Monitor, R41C. This monitor (R41C) along with R41A and R41B will initiate isolation of Containment Purge/Pressure-Vacuum Relief System on HIGH RADIATION also.

^c Grab sample stations from which monthly gas samples (Table 2.3-2) are to be taken. Also, grab samples should be taken and measured to determine the process stream or building ventilation system source whenever an unexplained increase is indicated by the plant vent sampler-monitors.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-70
AND AMENDMENT NO. 13 TO FACILITY OPERATING LICENSE NO. DPR-75

PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATION STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

Introduction

In a letter dated August 10, 1981, Public Service Electric and Gas Company (the licensee) submitted a request to modify Table 2.3-4 of Salem, Unit Nos. 1 and 2, Environmental Technical Specifications. This table indicates the location of process and effluent monitors and samplers required by technical specifications.

In a subsequent letter dated June 22, 1982, the licensee revised its proposed revision to Table 2.3-4 and, in addition, proposed modifications to Tables 3.3-3 and 3.3-6 of the Safety Technical Specifications and Table 2.3-2 of the Environmental Technical Specifications. Table 3.3-3 presents the engineered safety feature actuation system instrumentation. Table 3.3-6 presents the radiation monitoring instrumentation. Table 2.3-2 contains the various gaseous waste sampling and analysis requirements for all effluent release points and processes such as waste gas decay tank releases and containment purges. The changes to these tables are proposed to reflect the as-built monitoring system of Salem, Unit No. 2, and the modifications which are proposed for Unit No. 1.

The licensee requested that Table 2.3-4 be amended to reflect the as-built process and effluent monitoring system of Unit No. 2 and for approval of the modification to the Unit No. 1 process and effluent monitoring system which will make it identical to the Unit No. 2 system. The present table reflects the Unit No. 1 as-built condition but does not reflect Unit No. 2 as-built condition for the plant vent, reactor containment building, and waste gas discharge line.

Presently the waste gas discharge line of Unit No. 1 is monitored by gaseous monitor 1-R14 while Unit No. 2 does not have such a monitor. A high radiation signal from this monitor will automatically close the gas release valve in the Waste Disposal System of Unit No. 1. For Unit No. 2, the same function is performed by the gaseous monitor 2-R41C which is at the plant vent. Unit No. 2 does not have a separate monitor in the gas discharge line.

Flow from the containment purge of Unit No. 1 is monitored at the plant vent by air particulate, noble gas and radioiodine monitors 1-R11A, 1-R12A, and 1-R12B, respectively. These monitors also have the capability of sampling the containment atmosphere. A high radiation signal from any one of these monitors will initiate closure of the vacuum-relief line and/or the containment purge. For Unit No. 2, this monitoring function is served by plant vent monitors 2-R41A, 2-R41B, and 2-R41C. Radiation monitors 2-R11A, 2-R12A, and 2-R12B monitor exclusively the containment atmosphere at Unit No. 2.

The licensee requested that Table 3.3-3 and 3.3-6 of the Unit No. 1 Safety Technical Specifications be modified to reflect the proposed monitoring design changes. These changes would result in existing monitors 1-R11A, 1-R12A, and 1-R12B being utilized to monitor the containment atmosphere exclusively while new monitors 1-R41A, 1-R41B and 1-R41C would be utilized to monitor plant vent releases. These new monitors would also function as containment purge isolation monitors when the purge/vacuum relief isolation valves were open. However, while in this mode of operation, the licensee proposed that the monitors setpoints would be lowered. The present footnote in Tables 3.3-3 and 3.3-6 indicates that monitors 1-R11A, 1-R12A and 1-R12B can be utilized to monitor the plant vent effluent rather than monitoring containment atmosphere for up to 8 hours per 24 hour interval while either purging the containment atmosphere or venting a gas decay tank. With the proposed monitoring changes at Unit No. 1 this footnote would no longer be applicable.

For Unit No. 2, Tables 3.3-3 and 3.3-6 were proposed to be modified to clarify the footnote that the set points for the unit vent monitors (2-R41 series) would be lowered when functioning in the containment atmosphere isolation mode during purge or vacuum relief operations with these valves open.

The licensee also proposed that Table 2.3-2 of the Environmental Technical Specifications be clarified so that sampling and analysis of the containment would not be required for pressure-vacuum relief operations.

Evaluation and Findings

The staff has conducted an independent review and analysis of the potential radiological impact associated with the amending of Tables 2.3-2 and 2.3-4 of the Environmental Technical Specifications and Tables 3.3-3 and 3.3-6 of the Safety Technical Specifications.

A. Radioactive Release Considerations

The amending of Table 2.3-4 to reflect the as-built process and effluent monitoring system of Unit No. 2 will result in no additional releases of airborne effluents from Unit No. 2 since the change would only acknowledge the present system at Unit No. 2. The proposed modification to Unit No. 1 to make it identical to Unit No. 2, does present the possibility that additional airborne effluents could occur as a result of the new process and effluent monitoring system. The proposed modification to the Unit No. 1 system would eliminate monitor 1-R14. This monitor, which monitors the discharge from the gas decay tanks, presently causes the gas release valve to close automatically on a high radiation signal. In the proposed modification to Unit No. 1, a monitor 1-R41C, located at the plant vent, would close the gas release valve on a high radiation signal. The additional releases associated with the proposed modifications would occur because the monitor 1-R14 can close the valve sooner than the monitor at the plant vent. If the monitor, 1-R14, exceeded its setpoint, then the plant vent monitor 1-R41C would relay the high radiation signal to the control room less than 1 second later than if the 1-R14 monitor was still in the gas discharge line. At a maximum flow rate of 150 cfm this would result in an additional 2.5 ft³ of discharge would be an extremely small fraction of the annual anticipated release. The doses associated with such activity would be a small fraction of the annual beta and gamma air doses resulting from normal operation. Thus, the impact of this monitoring change will be minimal and is therefore acceptable.

With the acceptance of the proposed monitoring change to Unit No. 1, the changes to the footnotes in Tables 3.3-3 and 3.3-6 are also acceptable as these changes would then reflect the actual monitoring conditions. The change to the footnote to Tables 3.3-3 and 3.3-6 is acceptable because this change recognizes the lowering of the setpoint when the containment atmosphere is monitored during purge/pressure vacuum relief operations and does not impact either plant releases or the health and safety of the public.

The change to Table 2.3-2 distinguishes between sampling and analysis requirements for containment purges and pressure-vacuum relief operations. There is no need to sample and analyze the containment prior to pressure-vacuum relief operations as the discharge will be short term in nature. Therefore, the change to Table 2.3-2 is acceptable. The change to Table 2.3-4 to reflect the as-built condition of Unit No. 2 is also acceptable as it involves no increase in effluents nor a decrease in the safety margin for the plant as it presently exists.

Based upon the above evaluation, we conclude that the health and safety of the public will not be endangered by (1) amending Tables 2.3-2 and 2.3-4 of the Environmental Technical Specifications, (2) modifying Unit No. 1 process and effluent monitoring system, and (3) changing Tables 3.3-3 and 3.3-6 to reflect the changes to Unit No. 1 process and

effluent monitoring system and the as-built monitoring system for Unit No. 2. In addition, the amending of these tables and the modification to the Unit No. 1 process and effluent monitoring system will not increase the probability or consequences of accidents and does not involve a decrease in safety margin nor involve a significant hazards consideration.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 18, 1982

SER prepared by J. Hayes (ETSB)
W. Ross (ORB #1)

ENVIRONMENTAL IMPACT APPRAISAL BY
THE OFFICE OF NUCLEAR REACTOR REGULATION
REGARDING AMENDING TABLES 2.3-2 AND 2.3-4 OF THE
ENVIRONMENTAL TECHNICAL SPECIFICATIONS
AND TABLES 3.3-3 AND 3.3-6 OF THE
SAFETY TECHNICAL SPECIFICATIONS

Salem Nuclear Generating Station, Unit Nos. 1 and 2
Public Service Electric and Gas Company
Docket Nos. 50-272 and 50-311

Introduction

Public Service Electric and Gas Company is presently licensed to operate the Salem Nuclear Generating Station located on Artificial Island on the Delaware River in Lower Alloways Township, Salem County, New Jersey. There are two pressurized water reactors at the site, each reactor capable of generating 3250 MWt of power. The proposed amending of Tables 2.3-2 and 2.3-4 of the Station's Environmental Technical Specifications, Tables 3.3-3 and 3.3-6 of the Station's Safety Technical Specifications, and the modifications to the Unit No. 1 process and effluent monitoring system will not affect the reactor power level nor the fuel burnup and, therefore, not affect the benefits of the electrical power production considered in the Commission's Final Environmental Statement, Docket Nos. 50-272 and 50-311.

A. Radiological Impact

As evaluated in the associated Safety Evaluation, the proposed request does not affect the conclusions of the initial SER which were that the process and effluent monitoring system assures that releases during normal operation meet the limits of Table 2, Column 1 of Appendix B of 10 CFR Part 20.

In addition, the amending of these tables and the Unit 1 process and effluent monitoring system does not negate the fact that releases will be acceptable during normal operation and low probability accidents and within the "as low as reasonably achievable" requirements of 10 CFR Part 50, Appendix 1.

The change in monitoring location of waste gas discharges would result in no more than one second of additional flow from the waste gas decay tank. The activity released would be a very small fraction of the annual effluents from the plant and the associated doses would be a likewise small fraction of the annual beta and gamma air doses anticipated from the units.

B. Conclusion

On the basis of the foregoing evaluation, it is concluded that there would be no significant environmental impact attributable to the amending of Tables 2.3-2 and 2.3-4 of the Environmental Technical Specifications, Table 3.3-3 and 3.3-6 of the Safety Technical Specifications, and the proposed modification to the Unit No. 1 process and effluent monitoring system. As a result of this conclusion, the Commission has further concluded that no environmental impact statement for the proposed action need be prepared and that a negative declaration to this effect is appropriate.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-272 AND 50-311PUBLIC SERVICE ELECTRIC AND GAS COMPANY,
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSES
AND NEGATIVE DECLARATION

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 48 to Facility Operating License No. DPR-70 and Amendment No. 13 to Facility Operating License No. 75, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (the facilities) located in Salem County, New Jersey. The amendments are effective as of the date of issuance.

The amendments revise the Radiological Safety and Environmental Technical Specifications related to process and effluent monitors to agree with existing plant conditions.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

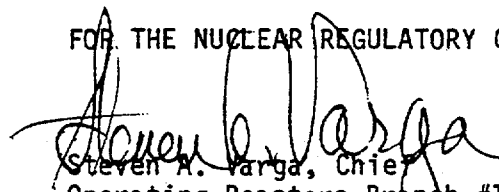
- 2 -

The Commission has prepared an environmental impact appraisal for the revised Technical Specifications and has concluded that an environmental impact statement for this particular action is not warranted because there will be no environmental impact attributable to the action other than that which has already been predicted and described in the Commission's Final Environmental Statement for the facility dated April 1973.

For further details with respect to this action, see (1) the application for amendments dated August 10, 1981, as modified by letter dated June 22, 1982, (2) Amendment Nos. 48 and 13 to License Nos. DPR-70 and DPR-75, (3) the Commission's related Safety Evaluation and (4) the Commission's Environmental Impact Appraisal. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 18th day of October, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing