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JUL 30 1981

Docket No. 50-272

Mr. F. W. Schneider, Vice President
 Production
 Public Service Electric and Gas Company
 80 Park Plaza, 15A
 Newark, New Jersey 07101

Dear Mr. Schneider:

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. DPR-70 for the Salem Nuclear Generating Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your request dated June 16, 1981.

The amendment revises the Radiological Standard Technical Specification related to the containment isolation pressure setpoint.

Copies of the Safety Evaluation, Technical Evaluation Report and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

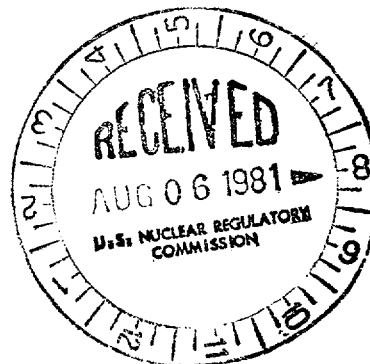
S. A. Varga

Steven A. Varga, Chief
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. 37 to DPR-70
2. Safety Evaluation
3. Technical Evaluation Report
4. Notice of Issuance

cc w/enclosures:
 See next page



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Mr. F. W. Schneider
Public Service Electric and Gas Company

cc: Mark J. Wetterhahn, Esquire
Conner, Moore and Corber
Suite 1050
1747 Pennsylvania Avenue, NW
Washington, D. C. 20006

Richard Fryling, Jr., Esquire
Assistant General Solicitor
Public Service Electric and Gas Company
80 Park Place
Newark, New Jersey 07101

Gene Fisher, Bureau of Chief
Bureau of Radiation Protection
380 Scotch Road
Trenton, New Jersey 08628

Mr. Henry J. Midura, Manager
Salem Nuclear Generating Station
Public Service Electric and Gas Company
P. O. Box 168
Hancocks Bridge, New Jersey 08038

Salem Free Library
112 West Broadway
Salem, New Jersey 08079

Leif J. Norrholm, Resident Inspector
Salem Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Drawer I
Hancocks Bridge, New Jersey 08038

Attorney General
Department of Law and Public Safety
State House Annex
Trenton, New Jersey 08625

Samuel E. Donelson, Mayor
Lower Alloways Creek Township
Municipal Hall
Hancocks Bridge, New Jersey 08038

Richard B. McGlynn, Commissioner
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

Deputy Attorney General
State House Annex
State of New Jersey
36 West State Street
Trenton, New Jersey 08625

Regional Radiation Representative
EPA Region II
26 Federal Plaza
New York, New York 10007

Mr. R. L. Mittl, General Manager
Licensing and Environment
Public Service Electric and Gas Company
80 Park Plaza
Newark, New Jersey 07101

John M. Zupko, Jr., Manager
Nuclear Operations Support
Public Service Electric and Gas Company
80 Park Plaza 15-A
Newark, New Jersey 07101

Lower Alloways Creek Township
c/o Michael C. Facemeyer, Clerk
Municipal Building
Hancocks Bridge, New Jersey 08038

Mr. Alfred C. Coleman, Jr.
Mrs. Eleanor G. Coleman
35 K Drive
Pennsville, New Jersey 08070

Mr. F. W. Schneider
Public Service Electric and Gas Company

cc: Mr. R. A. Uderitz, General Manager
Nuclear Production
Production Department
Public Service Electric and Gas
Company
80 Park Plaza 15A
Newark, New Jersey 07101

Mr. Mark L. First
Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
Environmental Protection Section
36 West State Street
Trenton, New Jersey 08625

Mr. Dale Bridenbaugh
M.H.B. Technical Associates
1723 Hamilton Avenue, Suite K
San Jose, California 95125

Mr. J. T. Boettger, General Manager
Quality Assurance I&E
Public Service Electric and Gas
Company
80 Park Plaza
Newark, New Jersey 07101

Mr. Edwin A. Liden, Manager
Nuclear Licensing
Licensing and Environment Dept.
80 Park Plaza 16D
Newark, New Jersey 07101

Carl Valore, Jr., Esquire
Valore, McAllister, Aron and
Westmoreland, P.A.
535 Tilton Road
Northfield, New Jersey 08225

June D. MacArtor, Esquire
Deputy Attorney General
Tatnall Building
P. O. Box 1401
Dover, Delaware 19901

Keith A. Orsdorff, Esquire
Department of the Public Advocate
Division of Public Interest Advocacy
520 East State Street
Trenton, New Jersey 08601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated June 16, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

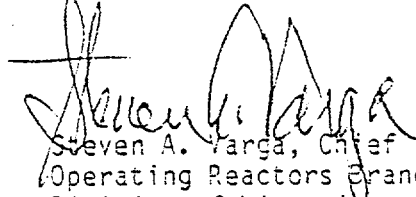
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 30, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Revise Appendix A as follows:

Remove Pages

3/4 3-23

Insert Pages

3/4 3-23

TABLE 3.3-4

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. SAFETY INJECTION, TURBINE TRIP AND FEEDWATER ISOLATION		
a. Manual Initiation	Not Applicable	Not Applicable
b. Automatic Actuation Logic	Not Applicable	Not Applicable
c. Containment Pressure--High	≤ 4.0 psig	≤ 4.5 psig
d. Pressurizer Pressure--Low	≥ 1765 psig	≥ 1755 psig
e. Differential Pressure Between Steam Lines--High	≤ 100 psi	≤ 112 psi
f. Steam Flow in Two Steam Lines-- High Coincident with T_{avg} --Low-Low or Steam Line Pressure--Low	<p>< A function defined as follows: A Δp corresponding to 40% of full steam flow between 0% and 20% load and then a Δp increasing linearly to a Δp corresponding to 110% of full steam flow at full load</p> <p>$T_{avg} \geq 543^{\circ}\text{F}$ ≥ 500 psig steam line pressure</p>	<p>< A function defined as follows: A Δp corresponding to 44% of full steam flow between 0% and 20% load and then a Δp increasing linearly to a Δp corresponding to 111.5% of full steam flow at full load</p> <p>$T_{avg} \geq 541^{\circ}\text{F}$ ≥ 480 psig steam line pressure</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-70

PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANY

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

DOCKET NO. 50-272

1.0 Introduction

As a consequence of the accident at TMI-2, implementation of a number of new requirements has been recommended for operating reactors. These new requirements are described in NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident," May 1980, and NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980. The NRC staff has also requested licensees to submit information sufficient to permit an independent evaluation of their response to these new requirements. This report provides an evaluation of the response to Action Plan Item II.E.4.2, position 5, and the Technical Specification revisions proposed by the licensee (Public Service Electric and Gas Company) in a letter dated June 16, 1981.

2.0 Evaluation

In its letter of April 23, 1981, the licensee proposed to change the minimum pressure set point for initiating containment isolation from the present value of ≤ 4.7 psi to ≤ 4 psi. The proposed value is the sum of three components:

- (1) 0.3 psig maximum allowed by Technical Specifications
- (2) 2.49 psi potential instrument error
- (3) 1.2 psi possible small pressure transient

Our consultant, the EG&G Energy Measurements Group (a subcontractor to Lawrence Livermore National Laboratory, which has the TMI Action Plan contract) has reviewed the licensee's submittals and prepared the attached technical evaluation report of the licensee's containment pressure setpoint used to isolate nonessential containment penetrations. We have reviewed this evaluation and concur in its basis and findings.

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3.0 Conclusions

The information submitted by the licensee provided sufficient details of the licensee's containment isolation pressure for the staff to conclude that the requirements of Item II.E.4.2(5) of NUREG-0737, with the additional guidelines provided by the staff, have been met. We, therefore, find that proposed changes acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 30, 1981

TECHNICAL EVALUATION OF THE RESPONSE TO POSITION NO.5
OF ITEM II.E. 4.2 OF NUREG-0737
CONTAINMENT ISOLATION SETPOINT
FOR THE
SALEM
NUCLEAR POWER PLANT
UNITS 1 AND 2

(DOCKET Nos. 50-272 and 50-311)

by

W. O. Wade

Approved for Publication

W. R. Ruvalcoba
W. R. Ruvalcoba
Department Manager

This document is UNCLASSIFIED

Derivative
Classifier:

Nicholas E. Broderick
Nicholas E. Broderick
Department Manager

Work Performed for Lawrence Livermore National Laboratory under U.S. Department of Energy
Contract No. DE-AC05-76 NVO 1183.

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INTRODUCTION AND BACKGROUND

As a consequence of the incident at TMI-2, implementation of a number of new requirements has been recommended for operating reactors. These new requirements are described in NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident," May 1980, and NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980. The NRC staff has also requested licensees to submit information sufficient to permit an independent evaluation of their response to these new requirements. This report provides an evaluation of the response to Action Plan Item II.E.4.2, position 5, by the designated licensee.

DESIGN BASIS OR REVIEW CRITERIA

Position 5 requires that the containment pressure setpoint that initiates containment isolation for non-essential system containment vessel penetrations be at, or reduced to, "...the minimum compatible with normal operating conditions."

TECHNICAL EVALUATION

Response evaluation is based upon the values provided for the following parameters:

- (1) The maximum observed or expected containment pressure during normal operation.
- (2) The loop error and observed drift in the pressure sensing instrumentation providing the isolation signal (see note).
- (3) The containment isolation pressure setpoint.

NOTE: The clarification document (NUREG-0737) provided only the expected margin for instrument error and did not specify acceptable values for instrument drift or atmospheric changes contributing to the total sensing loop error. Additional staff guidance established a limit of 3.0 psi for an isolation setpoint margin over the normal containment pressure to account for total loop error. In addition, for subatmospheric containments, a 3.0 psi setpoint margin over atmospheric pressure is also considered acceptable.

In consideration of these values, the isolation pressure setpoint is to be as low as practical without increasing the probability of inadvertent activation of the isolation signal.

CONCLUSIONS

The letter of April 23, 1981, submitted by Public Service Electric and Gas Company, provided sufficient information to conclude that the containment isolation pressure setpoint for Salem Nuclear Power Plant, Units 1 and 2, meets the NUREG-0660/0737 requirements and is within the additional limiting guidelines provided by the NRC staff.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-272PUBLIC SERVICE ELECTRIC AND GAS COMPANY
PHILADELPHIA ELECTRIC COMPANY,
DELMARVA POWER AND LIGHT COMPANY, AND
ATLANTIC CITY ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 37 to Facility Operating License No. DPR-70, issued to Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees), which revised Technical Specifications for operation of the Salem Nuclear Generating Station, Unit No. 1 (the facility) located in Salem County, New Jersey. The amendment is effective as of the date of issuance.

The amendment revises the Radiological Technical Specification related to the containment isolation setpoint.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.


-2-

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 16, 1981, (2) Amendment No. 37 to License No. DPR-70, (3) the Commission's related Safety Evaluation, and (4) the related Technical Evaluation Report. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C and at the Salem Free Public Library, 112 West Broadway, Salem, New Jersey. A copy of items (2), (3), and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 30th day of July, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing