

July 21, 1992

Docket No. 50-271

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Mr. L. A. Tremblay
Senior Licensing Engineer
Vermont Yankee Nuclear Power Corporation
580 Main Street
Bolton, Massachusetts 01740-1398

Dear Mr. Tremblay:

SUBJECT: ISSUANCE OF AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE
NO. DPR-28 - VERMONT YANKEE NUCLEAR POWER STATION (TAC NO. M82360)

The Commission has issued the enclosed Amendment No. 134 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. This amendment is in partial response to your application dated December 27, 1991.

This amendment reinstates the surveillance frequency required for testing the automatic closure of primary containment isolation valves. The first part of your request is being processed by a separate letter dated July 21, 1992.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly FEDERAL REGISTER Notice.

Sincerely,

Original signed by Patrick Sears

Patrick Sears, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 134 to License No. DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

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[Signature]

Mr. L. A. Tremblay, Senior Licensing
Engineer

Vermont Yankee

cc:

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Mr. Richard P. Cedano, Commissioner
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Montpelier, Vermont 05602

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Public Service Board
State of Vermont
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Chairman, Board of Selectmen
Town of Vernon
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AMENDMENT NO. 134 TO DPR-28 VERMONT YANKEE NUCLEAR POWER STATION
DATED July 21, 1992

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Vermont Yankee Nuclear Power Corporation (the licensee) dated December 27, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-28 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



(for)

Victor Nerses, Acting Director
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 21, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 134

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

132

133

Insert

132

133

3.7 LIMITING CONDITION FOR OPERATION

- b. The reactor water temperature is below 212°F and the reactor coolant system is vented.
 - c. No activity is being performed which can reduce the shutdown margin below that specified in Specification 3.3.A.
 - d. The fuel cask or irradiated fuel is not being moved in the Reactor Building.
2. Core spray and LPCI pump lower compartment door openings shall be closed at all times except during passage or when reactor coolant temperature is less than 212°F.

D. Primary Containment Isolation Valves

1. During reactor power operating conditions all isolation valves listed in Table 4.7.2 and all instrument line flow check valves shall be operable except as specified in Specification 3.7.D.2.

4.7 SURVEILLANCE REQUIREMENTS

- b. Additional tests shall be performed during the first operating cycle under an adequate number of different environmental wind conditions to enable valid extrapolation of the test results.
 - c. Secondary containment capability to maintain a 0.15 inch of water vacuum under calm wind ($2 < \bar{u} < 5$ mph) conditions with a filter train flow rate of not more than 1,500 cfm, shall be demonstrated at least quarterly and at each refueling outage prior to refueling.
2. The core spray and LPCI lower compartment openings shall be checked closed daily.

D. Primary Containment Isolation Valves

1. Surveillance of the primary containment isolation valves shall be performed as follows:
- a. The operable isolation valves that are power operated and automatically initiated shall be tested for automatic initiation and the closure times specified in Table 4.7.2 at least once per operating cycle.
 - b. Operability testing of the primary containment isolation valves shall be performed in accordance with Specification 4.6.E.

3.7 LIMITING CONDITIONS FOR OPERATION

4.7 SURVEILLANCE REQUIREMENTS

2. In the event any isolation valve specified in Table 4.7.2 becomes inoperable, reactor power operation may continue provided at least one valve in each line having an inoperable valve is in the mode corresponding to the isolated condition.
3. If Specification 3.7.D.1 and 3.7.D.2 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours.

- c. At least once per quarter, with the reactor power less than 75 percent of rated, trip all main steam isolation valves (one at a time) and verify closure time.
 - d. At least twice per week, the main steam line isolation valves shall be exercised by partial closure and subsequent reopening.
2. Whenever an isolation valve listed in 4.7.2 is inoperable, the position of at least one other valve in each line having an inoperable valve shall be logged daily.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letter dated December 27, 1991, the Vermont Yankee Nuclear Power Corporation (the licensee) submitted a request for two changes to the Vermont Yankee Nuclear Power Station Technical Specifications (TS). The requested changes would (1) delete specific supervisory titles from a section of the Technical Specifications relative to the composition of the Plant Operations Review Committee (PORC) and authorize the Plant Manager to appoint members, and (2) reinstate the surveillance requirement for testing the automatic closure of primary containment isolation valves.

2.0 EVALUATION

The first proposed change has been processed by a separate letter, dated

The second proposed change restores the requirement to perform surveillance testing on primary containment isolation valves. The surveillance requirement was inadvertently deleted as a result of a previous amendment to the license. The surveillance requirement will be restored in its entirety, exactly as previously worded. This item is administrative in nature.

This proposed change has been reviewed by the Vermont Yankee Plant Operations Review Committee, and the Vermont Yankee Nuclear Safety and Audit Review Committee.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Vermont State official was notified of the proposed issuance of the amendment. The State official had no comments.

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4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 2603). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Sears

Date: July 21, 1992