

March 3, 1995

Mr. Donald A. Reid, Vice President
Operations
Vermont Yankee Nuclear Power Corporation
Ferry Road
Brattleboro, VT 05301

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SUBJECT: ISSUANCE OF AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-28, VERMONT YANKEE NUCLEAR POWER STATION (TAC NO. M91165)

Dear Mr. Reid:

The Commission has issued the enclosed Amendment No. 142 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. This amendment is in response to your application dated December 14, 1994.

This amendment revises the identification of low reactor pressure instrument trip card from a "slave" trip card designation to a "master" trip card designation.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register Notice.

Sincerely,

Original signed by
Daniel H. Dorman, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-271

Enclosures: 1. Amendment No. 142 to
License No. DPR-28
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Vermont Yankee Nuclear Power Corporation
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Sincerely,

A handwritten signature in black ink, reading "Daniel H. Dorman", is written over a horizontal line.

Daniel H. Dorman, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-271

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cc w/encls: See next page

Donald A. Reid, Vice President
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Vermont Yankee Nuclear Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.142
License No. DPR-28

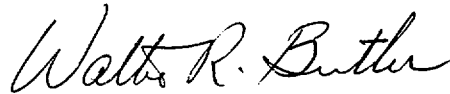
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Vermont Yankee Nuclear Power Corporation (the licensee) dated December 14, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-28 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.142 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 3, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Insert

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TABLE 3.2.1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Core Spray - A & B (Note 1)			
<u>Minimum Number of Operable Instrument Channels per Trip System</u>	<u>Trip Function</u>	<u>Trip Level Setting</u>	<u>Required Action When Minimum Conditions for Operation are Not Satisfied</u>
2	High Drywell Pressure	≤ 2.5 psig	Note 2
2	Low-Low Reactor Vessel Water Level	$\geq 82.5''$ above top of enriched fuel	Note 2
1	Low Reactor Pressure (PT-2-3-56C/D(M))	$300 \leq P \leq 350$ psig	Note 2
2	Low Reactor Pressure (PT-2-3-56A/B(M) & 52C/D(M))	$300 \leq P \leq 350$ psig	Note 2
1	Time Delay (14A-K16A & B)	≤ 10 seconds	Note 2
2	Pump (P-46-1A/B) Discharge Pressure	≥ 100 psig	Note 5
1	Auxiliary Power Monitor	--	Note 5
1	Pump Bus Power Monitor	--	Note 5
1	Trip System Logic	--	Note 5

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TABLE 3.2.1
(Cont'd)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Low Pressure Coolant Injection System A & B (Note 1)			
<u>Minimum Number of Operable Instrument Channels per Trip System</u>	<u>Trip Function</u>	<u>Trip Level Setting</u>	<u>Required Action When Minimum Conditions for Operation are Not Satisfied</u>
1	Low Reactor Pressure (PT-2-3-56C/D(M))	$300 \leq p \leq 350$ psig	Note 2
2	High Drywell Pressure (PT-10-101A-D(M))	≤ 2.5 psig	Note 2
2	Low-Low Reactor Vessel Water Level	$\geq 82.5^*$ above top of enriched fuel	Note 2
1	Time Delay (10A-K51A & B)	0 seconds	Note 5
1	Reactor Vessel Shroud Level	$\geq 2/3$ core height	Note 5
1	Time Delay (10A-K72A & B)	≤ 60 seconds	Note 5
1	Time Delay (10A-K50A & B)	≤ 5 seconds	Note 5
1	Low Reactor Pressure (PS-2-128A & B)	$100 \leq p \leq 150$ psig	Note 2
2 per pump	RHR Pump A & C Discharge Pressure	≥ 100 psig	Note 5
2	High Drywell Pressure (PT-10-101A-D(S1))	≤ 2.5 psig	Note 2

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TABLE 3.2.1
(Cont'd)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Low Pressure Coolant Injection System A & B (Note 1)

<u>Minimum Number of Operable Instrument Channels per Trip System</u>	<u>Trip Function</u>	<u>Trip Level Setting</u>	<u>Required Action When Minimum Conditions for Operation are Not Satisfied</u>
1	Time Delay (10A-K45A & B)	≤ 6 minutes	Note 5
2	Low Reactor Pressure (PT-2-3-56A/B(M) & 52C/D(M))	$300 \leq p \leq 350$ psig	Note 2
1	Auxiliary Power Monitor	--	Note 5
1	Pump Bus Power Monitor	--	Note 5
1	Trip System Logic	--	Note 5

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TABLE 4.2.1

MINIMUM TEST AND CALIBRATION FREQUENCIES
EMERGENCY CORE COOLING ACTUATION INSTRUMENTATION

Core Spray System			
<u>Trip Function</u>	<u>Functional Test(8)</u>	<u>Calibration(8)</u>	<u>Instrument Check</u>
High Drywell Pressure	(Note 1)	Once/Operating Cycle	Once Each Day
Low-Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	Once Each Day
Low Reactor Pressure (PT-2-3-56C/D(M))	(Note 1)	Once/Operating Cycle	--
Low Reactor Pressure (PT-2-3-56A/B(M) & 52C/D(M))	(Note 1)	Once/Operating Cycle	--
Pump (P-46-1A/B) Discharge Pressure	(Note 1)	Every Three Months	--
Auxiliary Power Monitor	(Note 1)	Every Refueling	Once Each Day
Pump Bus Power Monitor	(Note 1)	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	--

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TABLE 4.2.1
(Cont'd)

MINIMUM TEST AND CALIBRATION FREQUENCIES
EMERGENCY CORE COOLING ACTUATION INSTRUMENTATION

Low Pressure Coolant Injection System			
<u>Trip Function</u>	<u>Functional Test (8)</u>	<u>Calibration (8)</u>	<u>Instrument Check</u>
Low Reactor Pressure (PT-2-3-56C/D(M))	(Note 1)	Once/Operating Cycle	--
High Drywell Pressure (PT-10-101A-D(M))	(Note 1)	Once/Operating Cycle	Once Each Day
Low-Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	Once Each Day
Reactor Vessel Shroud Level	(Note 1)	Once/Operating Cycle	--
Low Reactor Pressure (PS-2-128A/B)	(Note 1)	Every Three Months	--
RHR Pump Discharge Pressure	(Note 1)	Every Three Months	--
High Drywell Pressure (PT-10-101A-D(S1))	(Note 1)	Once/Operating Cycle	--
Low Reactor Pressure (PT-2-3-56A/B) (M) & 52C/D(M))	(Note 1)	Once/Operating Cycle	--
Auxiliary Power Monitor	(Note 1)	Every Refueling Outage	Once Each Day
Pump Bus Power Monitor	(Note 1)	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	--



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letter dated December 14, 1994, the Vermont Yankee Nuclear Power Corporation (the licensee) submitted a request for changes to the Vermont Yankee Nuclear Power Station Technical Specifications (TSs). The requested changes would revise the identification of low reactor pressure instrument trip card from a "slave" trip card designation to a "master" trip card designation.

2.0 EVALUATION

The proposed changes to instrument identifications in Tables 3.2.1 and 4.2.1 revise the identification of low reactor pressure instrument trip cards from a "slave" trip card designation to a "master" trip card designation. The licensee plans to install a modification during the 1995 refueling outage to satisfy the instrument diversity requirements of the Anticipated Transient Without Scram Rule. As a result of the modification, the emergency core cooling system low reactor pressure circuitry will be comprised of "master" trip cards only. Therefore, the "slave" designation in the current TSs would become obsolete.

The licensee has reviewed the design change pursuant to 10 CFR 50.59(a)(2) and concluded that the change does not involve an unreviewed safety question. Additionally, neither the proposed TS amendment nor the modification change the design basis, protective function, redundancy, trip point, or logic of the system.

The change of identification of a low reactor pressure instrument in Table 4.2.1 from PT-2-128A/B to PS-2-128A/B corrects a typographical error, is consistent with designation of that instrument in Table 3.2.1 on Page 39 of the TS, and is purely administrative.

Therefore, the NRC staff concludes that the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Vermont State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 6315). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Dorman

Date: March 3, 1995