

April 25, 1990

Mr. L. A. Tremblay
Licensing Engineer
Vermont Yankee Nuclear Power Corporation
580 Main Street
Bolton, Massachusetts 01740-1398

Dear Mr. Tremblay:

SUBJECT: ISSUANCE OF AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE
NO. DPR-28 - VERMONT YANKEE NUCLEAR POWER STATION (TAC NO. 76082)

The Commission has issued the enclosed Amendment No.121 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. This amendment is in response to your application dated March 2, 1990.

This amendment modifies Technical Specifications (TS) 6.1 and 6.2 and their related Bases. Specifically, it removes the organization charts from the administrative control requirements of the TS. Appropriate changes have been made to administrative control requirements to compensate for the removal of the organizational charts.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register Notice.

This completes action under TAC 76082.

Sincerely,

15/

Morton B. Fairtile, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No.121 to License No. DPR-28
- 2. Safety Evaluation

cc w/enclosures:
See next page

OFC	: PDI-3/PM	: PDI-3/LA	: OGC	: PDI-3/D	:	:
NAME	: MB Fairtile	: MRushbrook	: [Signature]	: RHwessman	:	:
DATE	: 4/2/89	: 4/12/89	: 4/16/89	: 4/25/89	:	:

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3. This license amendment is effective immediately.

FOR THE NUCLEAR REGULATORY COMMISSION

Morton B. Fairtile for

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1990

OFC	: PDI-3/PM	: PDI-3/LA	: OGC	: PDI-3/D	:	:
NAME	: MBFairtile	: MRuslock	: <i>PPA</i>	: <i>MBS for</i>	:	:
DATE	: <i>4/2/89</i>	: <i>4/12/89</i>	: <i>4/16/89</i>	: <i>4/25/89</i>	:	:

OFFICIAL RECORD COPY

AMENDMENT NO. 121 TO DPR-28 VERMONT YANKEE NUCLEAR POWER STATION DATED April 25, 1990

DISTRIBUTION:

Docket File 50-271 ←

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 25, 1990

Docket No. 50-271

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Licensing Engineer
Vermont Yankee Nuclear Power Corporation
580 Main Street
Bolton, Massachusetts 01740-1398

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A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register Notice.

This completes action under TAC 76082.

Sincerely,

Morton B. Fairtile
Morton B. Fairtile, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 121 to License No. DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. L. A. Tremblay

-2-

April 25, 1990

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Atomic Safety and Licensing Board
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. L. A. Tremblay

-3-

April 25, 1990

cc:

Mr. Gustave A. Linenberger, Jr.
Administrative Judge
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Washington, D.C. 20555

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Washington, D.C. 20555

Adjudicatory File (2)
Atomic Safety and Licensing Board
Panel Docket
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Vermont Yankee Nuclear Power Corporation (the licensee) dated March 2, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-28 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective immediately.

FOR THE NUCLEAR REGULATORY COMMISSION

Norton B. Fairtle acting for

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
190	190
190a	190a
191	191
192	192
194	194
195	195
196	196
206	206

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6.0 ADMINISTRATIVE CONTROLS

Administrative controls are the written rules, orders, instructions, procedures, policies, practices, and the designation of authorities and responsibilities by the management to obtain assurance of safety and quality of operation and maintenance of a nuclear power reactor. These controls shall be adhered to.

6.1 ORGANIZATION

- A. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organizational positions. These relationships shall be documented and updated, as appropriate, in the form of organizational charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Yankee Operational Quality Assurance Manual.
- B. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those on-site activities necessary for safe operation and maintenance of the plant. (Succession to this responsibility during his absence shall be delegated in writing.)
- C. The Manager of Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- D. Conduct of operations of the plant will be in accordance with the following minimum conditions.
 - 1. An individual qualified in radiation protection procedures shall be present on-site at all times when there is fuel in the reactor.
 - 2. Minimum shift staffing on-site shall be in accordance with Table 6.1.1.
 - 3. A dedicated, licensed Senior Operator shall be in charge of any reactor core alteration.
 - 4. Qualifications with regard to educational background experience, and technical specialities of the key supervisory personnel listed below shall apply and be maintained in accordance with the levels described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants".

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- a. Plant Manager
- b. Superintendent(s)
- c. Chemistry Supervisor
- d. Radiation Protection Supervisor
- e. Operations Supervisor (See Item D.7, Page 190a)
- f. Reactor and Computer Supervisor
- g. Maintenance Supervisor
- h. Instrument and Control Supervisor
- i. Shift Supervisors

- 5. The Radiation Protection Supervisor or Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).
 - 6. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.
 - 7. If the Operations Supervisor does not possess a Senior Operator License, then an Assistant Operations Supervisor shall be designated that does possess a Senior Operator License. All instructions to the shift crews involving licensed activities shall than be approved by designated Assistant Operations Supervisor.
 - 8. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate on-site manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- E. A Fire Brigade of at least 5 members shall be maintained on-site at all times. # This excludes 2 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.

#Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

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6.2 REVIEW AND AUDIT

Organizational units for the review and audit of plant operations shall be constituted and have the responsibilities and authorities outlined below:

A. Plant Operations Review Committee

1. Membership

- a. Chairman: Plant Manager
- b. Vice-Chairman: Superintendent(s)
- c. Engineering Support Supervisor
- d. Operations Supervisor
- e. Maintenance Supervisor
- f. Reactor and Computer Supervisor
- g. Chemistry Supervisor
- h. Instrument and Control Supervisor
- i. Radiation Protection Supervisor

2. Qualifications

The qualifications of the regular members of the Plant Operations Review Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level at least equal to or higher than as described in Specification 6.1.

3. Meeting Frequency: Monthly, and as required, on call of the Chairman.

4. Quorum: Chairman or Vice-Chairman plus four members or their designated alternates.

NOTE: For purposes of satisfying a quorum, a Vice-Chairman may be considered a member providing that Vice-Chairman is not presiding over the meeting.

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5. Designated alternates shall be from other plant personnel in the appropriate disciplines or as selected by the Plant Manager; however, there shall be no more than three (3) alternates serving on the committee at any one time.
6. Responsibilities
 - a. Review proposed normal, abnormal, and emergency operating procedures. Review all proposed maintenance procedures and proposed changes to those procedures; and any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
 - b. Review proposed tests and experiments.
 - c. Review proposed changes to Technical Specifications.
 - d. Review proposed changes or modifications to plant systems or equipment, which changes would require a change in procedures in (a) above.
 - e. Review plant operations to detect any potential safety hazards.
 - f. Investigate reported instances of violations of Technical Specifications, such investigations to include reporting, evaluation, and recommendations to prevent recurrence, to the Manager of Operations.
 - g. Perform special reviews and investigations and render reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Committee.
7. Authority
 - a. The Plant Operation Review Committee shall be advisory.
 - b. The Plant Operation Review Committee shall recommend to the Plant Manager approval or disapproval of proposals under Items 6 (a) through (d) above.
 1. In the event of disagreement between the recommendations of the Plant Operation Review Committee and the actions contemplated by the Plant Manager, the course determined by the Plant Manager to be the more conservative will be followed with immediate notification to the Manager of Operations.

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- c. The Plant Operation Review Committee shall make tentative determinations as to whether or not proposals considered by the Committee involve unreviewed safety questions. This determination shall be subject to review by the Nuclear Safety Audit and Review Committee.

8. Records

Minutes shall be kept at the plant of all meetings of the Plant Operation Review Committee and copies shall be sent to the Manager of Operations and the Nuclear Safety Audit and Review Committee.

B. Nuclear Safety Audit and Review Committee

1. The Committee shall consist of at least six (6) persons:

- a. Chairman
- b. Vice Chairman
- c. Four technically qualified persons who are not members of the plant staff.
- d. No more than three members shall be selected from the organization reporting to the Manager of Operations.
- e. The Committee will obtain advice and counsel from scientific or technical personnel employed by the Company or other organizations whenever the Committee considers it necessary to obtain further scientific or technical assistance in carrying out its responsibilities.
- f. The Committee membership and its Chairman and Vice Chairman shall be appointed as specified in the Yankee Quality Assurance Manual

2. Qualifications

The Committee shall consist of a minimum of six (6) members of designated alternates who as a group employ expertise in the following areas:

- a. Nuclear Power Plant Technology
- b. Reactor Operations
- c. Utility Operations

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- C. Procedures prepared for A and B above shall be reviewed and approved by the applicable Department Supervisor and the Plant Manager, or his designee (superintendent level).
- D. Temporary changes to procedures described in Specification 6.5.A above which do not change the intent of the original procedure, may be made with the concurrence of two individuals holding senior operator licenses. Such changes shall be documented and subsequently reviewed by the PORC and approved by the Plant Manager or his designee.
- E. Temporary changes to procedures described in Specification 6.5.B may be made with the concurrence of an individual holding a senior operator license and the health physicist on duty.
- F. Licensed radioactive sealed sources shall be leak tested for contamination. Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement state as follows:
 - 1. Each licensed sealed source, except startup sources previously subjected to core flux, containing radioactive materials, other than Hydrogen 3, with half-life greater than thirty days and in any form, other than gas, shall be tested for leakage and/or contamination at intervals not to exceed six months.
 - 2. The periodic leak test required does not apply to sealed sources that are stored and are not being used. The sources exempted from this test shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer. In the absence of a certificate from a transferrer indicating that a leak test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.
 - 3. Each sealed startup source shall be tested within 31 days prior to being subjected to core flux and following repair or maintenance to the source.

The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.

Notwithstanding the periodic leak tests required by this Technical Specification, any licensed sealed source is exempt from such leak test when the source contains 100 microcuries or less of beta and/or gamma emitting material or 5 microcuries or less of alpha emitting material.

A special report shall be prepared and submitted to the Commission within 90 days if source leakage tests reveal the presence of ≥ 0.005 microcuries of removable contamination.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

INTRODUCTION

By letter dated March 2, 1990, the Vermont Yankee Nuclear Power Corporation (the licensee) proposed changes to Technical Specifications (TS) 6.1 Organization and 6.2 Review and Audit. The proposed changes would remove Figure 6.1.1. Offsite Support Organization, and Figure 6.1.2 Plant Operations Organization, and replace them with a narrative description of the offsite and onsite organizations functional requirements in TS 6.1. Guidance for these proposed changes to the TS was provided to licensees and applicants by Generic Letter 88-06, dated March 22, 1988.

BACKGROUND

Consistent with the guidance provided in the Standard Technical Specifications, Specifications 6.2.1 and 6.2.2 of the administrative control requirements have referenced offsite and unit (onsite) organization charts that are provided as figures to these sections. On a plant specific basis, these organization charts have been provided by applicants and included in the TS issued with the operating license. Subsequent restructuring of either the offsite or unit organizations, following the issuance of an operating license, has required licensee to submit a license amendment for NRC approval to reflect the desired change in these organizations. As a consequence, organizational changes have necessitated the need to request an amendment of the operating license.

Because of these limitations on organizational structure, the nuclear industry has highlighted this as an area for improvement in the TS. The Shearon Harris licensee proposed changes to remove organization charts from its TS under the lead-plant concept that included the endorsement of the proposed changes by the Westinghouse Owners Group. In its review of the Shearon Harris proposal, the staff concluded that most of the essential elements of offsite and onsite organization charts are captured by other regulatory requirements, notably, Appendix B to 10 CFR 50. However, there were aspects of the organizational structure that are important to ensure that the administrative control requirements of 10 CFR 50.36 would be met and that would not be retained with the removal of the organization charts. The applicable requirements are

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those administrative controls that are necessary to ensure safe operation of the facility. Therefore, those aspects of the organization charts for Shearon Harris that were essential for conformance with regulatory requirements were added (1) to Specification 6.2.1 to define functional requirements for the offsite organizations and (2) to Specification 6.2.2 to define qualification requirements of the unit staff. (These TS sections relate to Shearon Harris and are equivalent to 6.1 and 6.2 at Vermont Yankee).

By letter dated January 27, 1988, the staff issued Amendment No. 3 to Facility Operating License NPF-63 for the Shearon Harris Nuclear Power Plant that incorporated these changes to their TS. Subsequently the staff developed guidance on an acceptable format for license amendment requests to remove the organization charts from TS. Generic Letter 88-06 provides this guidance to all power reactors.

EVALUATION

The licensee's proposed changes to its TS are in accordance with the guidance provided by Generic Letter 88-06 and addressed the items listed below.

- (1) Specifications 6.1 and 6.2 were revised to delete the references to Figures 6.1.1 and 6.1.2 that were removed from the TS.
- (2) Functional requirements of the offsite and onsite organizations were defined and added to Specification 6.1 and they are consistent with the guidance provided in Generic Letter 88-06. The specification notes that implementation of these requirements is documented in the Yankee Operational Quality Assurance Manual.
- (3) The senior reactor operator and reactor license qualified positions, identified on the organization chart for the plant staff, were noted as being required by Table 6.1.1. Therefore, this requirement will be retained as a requirement following the removal of the organization chart for the plant staff.
- (4) Consistent with requirements to document the offsite and onsite organization relationship in the form of organization charts, the licensee has confirmed that this documentation has been designated for inclusion to the next update of the Yankee Operational Quality Assurance Manual.
- (5) The licensee has confirmed that no specifications, other than those noted in item (1) above, include references to the figures of the organization charts that are being removed from TS for their plant. Hence, this is not an applicable consideration, with regard to the need to redefine referenced requirements as a result of the removal of these figures.

On the basis of its review of the above items, the staff concludes that the licensee has provided an acceptable response to these items as addressed in the NRC guidance on removing organization charts from the administrative control requirements of the TS. Furthermore, the staff finds that these changes are consistent with the staff's generic finding on the acceptability of such changes as noted in Generic Letter 88-06. Accordingly, the staff finds the proposed changes to be acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22 (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration, which was published in the Federal Register (55 FR 10546) on March 21, 1990. The Commission consulted with the State of Vermont. No public comments were received, and the State of Vermont did not have any comments.

On the basis of the considerations discussed above, the staff concludes that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Thomas G. Dunning
Vernon L. Rooney

Dated: April 25, 1990