March 29, 1989

Docket No. 50-271

Mr. R. W. Capstick
Licensing Engineer
Vermont Yankee Nuclear Power
Corporation
580 Main Street
Bolton, Massachusetts 01740-1398

Dear Mr. Capstick:

SUBJECT: ISSUANCE OF AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-28 (TAC#62776) VERMONT YANKEE NUCLEAR POWER STATION

The Commission has issued the enclosed Amendment No. 110 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. This amendment consists of changes to the Technical Specifications in response to your application dated August 26, 1986. This amendment changes the Technical Specifications to reflect changes related to calibration, surveillance requirements and trip settings associated with the use of analog instrumentation.

Your request for revision of the Pressure Suppresion Chamber - Reactor Building Vacuum Breaker System operability requirements, which was also included in your August 26, 1986 application, will be reviewed separately under a new TAC number. This license amendment closed out TAC No.62776.

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/S/

Morton B. Fairtile, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 110 to DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

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*See previous concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 29, 1989

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Sincerely,

Morton B. Fairtile

Morton B. Fairtile, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

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1. Amendment No.110to DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

Amendment No. 110 to Vermont Yankee Nuclear Power Station Facility Operating License DPR-28

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Vermont Yankee Nuclear Power Corporation (the licensee) dated August 26, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-28 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 110 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 30 days after the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 29, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 119

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
35	35
36	36
37	37
50	50
51	51
52	52
55	55
56	56
57	57
59	59

TABLE 3.2.1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Core Spray - A & B (Note 1)

Minimum Number of Operable Instrument Channels per Trip System	Trip Function	Trip Level Setting	Required Action When Minimum Conditions for Operation are Not Satisfied
2	High Drywell Pressure	≤2.5 psig	Note 2
2	Low-Low Reactor Vessel Water Level	≥82.5" above top of enriched fuel	Note 2
1	Low Reactor Pressure (PT-2-3-56C/D(S1)	300 ≤ P ≤ 350 psig	Note 2
2	Low Reactor Pressure (PT-2-3-56A/B(S1) & 52C/D(M))	300 ≤ P ≤ 350 psig	Note 2
1	Time Delay (14A-K16A & B)	≤10 seconds	Note 2
2	Pump 14-1A, Discharge Pressure	≥100 psig	Note 5
1	Auxiliary Power Monitor	--	Note 5
1	Pump Bus Power Monitor	--	Note 5
1	High Sparger Pressure	≤5 psig	Note 5
1	Trip System Logic	--	Note 5

TABLE 3.2.1 (Cont)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Low Pressure Coolant Injection System A & B (Note 1)

Minimum Number of Operable Instrument Channels per Trip System	Trip Function	Trip Level Setting	Required Action When Minimum Conditions for Operation are Not Satisfied
1	Low Reactor Pressure (PT-2-3-56C/D(S1))	300 ≤ p ≤ 350 psig	Note 2
2	High Drywell Pressure (PT-10-101A-D(M))	≤2.5 psig	Note 2
2	Low-Low Reactor Vessel Water Level	≥82.5" above top of enriched fuel	Note 2
1	Time Delay (10A-K51A & B)	0 seconds	Note 5
1	Reactor Vessel Shroud Level	≥2/3 core height	Note 5
1	Time Delay (10A-K72A & B)	≤60 seconds	Note 5
1	Time Delay (10A-K50A & B)	≤5 seconds	Note 5
1	Low Reactor Pressure (PS-2-128A & B)	100 ≤ p ≤ 150 psig	Note 2
2 per pump	RHR Pump A & C Discharge Pressure	≥100 psig	Note 5
2	High Drywell Pressure (PT-10-101A-D(S1))	≤2.5 psig	Note 2

TABLE 3.2.1 (Cont)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

Low Pressure Coolant Injection System A & B (Note 1)

Minimum Number of Operable Instrument Channels per Trip System	Trip Function	Trip Level Setting	Required Action When Minimum Conditions for Operation are Not Satisfied
1	Time Delay (10A-K45A & B)	$\leq$6 minutes	Note 5
2	Low Reactor Pressure (PT-2-3-56A/B(S1) & 52C/D(M))	300 $\leq$ p $\leq$ 350 psig	Note 2
1	Auxiliary Power Monitor	--	Note 5
1	Pump Bus Power Monitor	--	Note 5
1	Trip System Logic	--	Note 5

TABLE 4.2.1

MINIMUM TEST AND CALIBRATION FREQUENCIES

EMERGENCY CORE COOLING ACTUATION INSTRUMENTATION

Core Spray System

Trip Function	Functional Test(8)	Calibration(8)	Instrument Check
High Drywell Pressure	(Note 1)	Once/Operating Cycle	Once Each Day
Low-Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	Once Each Day
Low Reactor Pressure (PT-2-3-56C/D(S1))	(Note 1)	Once/Operating Cycle	---
Low Reactor Pressure (PT-2-3-56A/B(S1) & 52C/D(M))	(Note 1)	Once/Operating Cycle	---
Pump 14-1A, Discharge Pressure	(Note 1)	Every Three Months	---
Auxiliary Power Monitor	(Note 1)	Every Refueling	Once Each Day
Pump Bus Power Monitor	(Note 1)	None	Once Each Day
High Sparger Pressure	(Note 1)	Every Three Months	---
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---

TABLE 4.2.1 (continued)

Low Pressure Coolant Injection System			
Trip Function	Function Test(8)	Calibration(8)	Instrument Check
Low Reactor Pressure (PT-2-3-56C/D(S1))	(Note 1)	Once/Operating Cycle	---
High Drywell Pressure (PT-10-101A-D(M))	(Note 1)	Once/Operating Cycle	Once Each Day
Low-Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	Once Each Day
Reactor Vessel Shroud Level	(Note 1)	Once/Operating Cycle	---
Low Reactor Pressure (PT-2-128A/B)	(Note 1)	Every Three Months	---
RHR Pump Discharge Pressure	(Note 1)	Every Three Months	---
High Drywell Pressure (PT-10-101A-D(S1))	(Note 1)	Once/Operating Cycle	---
Low Reactor Pressure (PT-2-3-56A/B)(S1) & 52C/D(M))	(Note 1)	Once/Operating Cycle	---
Auxiliary Power Monitor	(Note 1)	Every Refueling Outage	Once Each Day
Pump Bus Power Monitor	(Note 1)	None	Once Each Day
LPCI Crosstie Monitor	None	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---

TABLE 4.2.1 (Cont)

High Pressure Coolant Injection System			
Trip Function	Functional Test(8)	Calibration(8)	Instrument Check
Low-Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	Once Each Day
Low Condensate Storage Tank Water Level	(Note 1)	Every Three Months	---
High Drywell Pressure	(Note 1)	Once/Operating Cycle	Once Each Day
Bus Power Monitor	(Note 1)	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---

TABLE 4.2.2 (Cont)

MINIMUM TEST AND CALIBRATION FREQUENCIES

HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION INSTRUMENTATION

Trip Function	Functional Test(8)	Calibration(8)	Instrument Check
High Reactor Water Level	(Note 1)	Once/Operating Cycle	---
High Steam Line Space Temperature	(Note 1)	Each Refueling Outage	---
High Steam Line D/P (Steam Line Break)	(Note 1)	Every Three Months	---
Low HPCI Steam Supply Pressure	(Note 1)	Every Three Months	---
Main Steam Line Tunnel Temperature	(Note 1)	Each Refueling Outage	---
Bus Power Monitor	(Note 1)	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---

TABLE 4.2.2 (Cont)

MINIMUM TEST AND CALIBRATION FREQUENCIES

REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION INSTRUMENTATION

Trip Function	Functional Test(8)	Calibration(8)	Instrument Check
Main Steam Line Tunnel Temperature	(Note 1)	Each Refueling Outage	---
High Steam Line Space Temperature	(Note 1)	Each Refueling Outage	---
High Steam Line D/P Including Time Delay Relays (Steam Line Break)	(Note 1)	Every Three Months	---
High Reactor Water Level	(Note 1)	Once/Operating Cycle	---
Low RCIC Steam Supply Pressure	(Note 1)	Every Three Months	---
Bus Power Monitor	(Note 1)	None	Once Each Day
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---

TABLE 4.2.3

MINIMUM TEST AND CALIBRATION FREQUENCIES

REACTOR BUILDING VENTILATION AND STANDBY GAS TREATMENT SYSTEM ISOLATION

Trip Function	Functional Test(8)	Calibration(8)	Instrument Check
Low Reactor Vessel Water Level	(Note 1)	Once/Operating Cycle	---
High Drywell Pressure	(Note 1)	Once/Operating Cycle	---
Reactor Building Vent Exhaust Radiation	Monthly	Every Three Months	Once Each Day
Refueling Floor Zone Radiation	Monthly	Every Three Months	Once Each Day During Refueling
Reactor Building Vent Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---
Standby Gas Treatment Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)	---
Logic Bus Power Monitor	(Note 1)	None	Once Each Day

TABLE 4.2.5

MINIMUM TEST AND CALIBRATION FREQUENCIES

CONTROL ROD BLOCK INSTRUMENTATION

Trip Function	Functional Test	Calibration
Startup Range Monitor		
a. Upscale	Notes 4 and 6	Note 6
b. Detector Not Fully Inserted	Note 6	Note 6
Intermediate Range Monitor		
a. Upscale	Notes 4 and 6	Note 6
b. Downscale	Notes 4 and 6	Note 6
c. Detector Not Fully Inserted	Note 6	Note 6
Average Power Range Monitor		
a. Upscale (Flow Bias)	Notes 1 and 4	Every Three Months
b. Downscale	Notes 1 and 4	Every Three Months
Rod Block Monitor		
a. Upscale (Flow Bias)	Notes 1 and 4	Every Three Months
b. Downscale	Notes 1 and 4	Every Three Months
Trip System Logic	Once/Operating Cycle	Once/Operating Cycle (Note 3)
High Water Level in Scram Discharge Volume	Every Three Months	Refueling Outage

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 110

TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letter dated August 26, 1986, Vermont Yankee Nuclear Power Corporation (VYNPC or the licensee) submitted proposed Change No. 134, requesting an amendment to Appendix A, Technical Specification (TS), of Facility Operating License No. DPR-28. The proposal would make changes in the calibration frequencies for certain Emergency Core Cooling System (ECCS) Actuation Instrumentation, to reflect the installation of an analog trip system, and make various administrative changes to limiting conditions for operations and surveillance requirements of Protective Instrument systems.

The proposal also requested changes to the Limiting Condition for Operation and Bases for operation with one of the Pressure Suppression Chamber-Reactor Building Vacuum Breaker Systems out of service. This item will be addressed in a separate Safety Evaluation.

PROPOSED CHANGES

The proposed changes, as reviewed, consist of three items: (1) revise the designation of trip functions in TS Table 3.2.1 and TS Table 4.2.1 by adding actual instrument numbers for low reactor pressure and high drywell pressure ECCS Actuation Instrumentation of the LPCI and CS Systems, (2) change the calibration frequencies of the reactor vessel shroud level and high drywell pressure trip functions of the ECCS Actuation Instrumentation for the LPCI System in TS Table 4.2.1, and (3) modify the surveillance requirements in TS Tables 4.2.1 (ECCS Actuation Instrumentation), 4.2.2 (HPCI and RCIC Isolation Instrumentation), 4.2.3 (Reactor Building Ventilation and Standby Gas Treatment System Isolation Instrumentation) and 4.2.5 (Control Rod Block Instrumentation) by deletion of reference to "Note 2" for Logic System Functional Tests.

REVIEW CRITERIA/REQUIREMENTS

(1) NUREG-0123, Standard Technical Specifications for General Electric Boiling Water Reactors, Revision 3.

(2) Safety Evaluation Supporting Amendment No. 58 to Facility Operating License No. DPR-28, dated November 3, 1980.

(3) Safety Evaluation Supporting Amendment No. 76 to Facility Operating License No. DPR-28, dated March 28, 1983.

2.0 DISCUSSION AND EVALUATION

ITEM 1

The licensee states that the current numbering used in Tables 3.2.1 and 4.2.1 to differentiate trip functions is undefined and, in some cases, may be misleading. In addition, common instruments which provide inputs into more than one system are labeled differently in the tables for different systems. The licensee proposes to delete the existing labels and include the actual equipment numbers in order to identify the instruments, enhance clarity and minimize the potential for misinterpretations. Because common instruments provide the low reactor pressure permissive signals to the ECCS Actuation Instrumentation for both the CS and LPCI Systems, the proposal would also add a restriction on the Trip Level Sepoint of the low reactor pressure permissive functions for the ECCS Actuation Instrumentation of the CS System in TS Table 3.2.1 from its current valve of "$\geq$300 psig" to "300 $\leq$ P $\leq$ 350 psig."

The staff review of the proposed changes determined they are administrative in nature and do not involve a change to the limiting conditions for operation or the surveillance requirements. The changes would clarify the TS and reduce the potential for misinterpretations. Therefore, the proposed changes to Trip Function nomenclature in TS Tables 3.2.1 and 4.2.1 are acceptable.

ITEM 2

The proposal would change the calibration frequency of the reactor vessel shroud level and high drywell pressure Trip Functions of the ECCS Actuation Instrumentation for the LPCI System in TS Table 4.2.1 from "Every 3 Months" to "Once/Operating Cycle". The TS definition of "Operating Cycle" is "the interval between the end of one refueling outage and the end of the next subsequent refueling outage." The nominal length of an operating cycle is 18 months.

The licensee states that the replacement of level and pressure switches with more reliable analog instrumentation justifies the decrease in the calibration frequency and that similar changes to calibration requirements have been previously approved for analog instrumentation at Vermont Yankee.

By License Amendment No. 58, dated November 3, 1980, the staff approved installation at Vermont Yankee of an improved safety instrumentation system referred to as the analog trip system. The staff approved changes in calibration frequencies, similar to the present request, for channels converted to the analog system in licensee Amendment No. 58 and subsequently, for additional channels, in License Amendment No. 76, dated March 28, 1983. The staff finds the present request to be in accordance with the guidance for calibration frequencies for analog instruments in the Standard Technical Specifications and to be consistent with the previously approved license amendments.

Based upon the above, the staff finds the proposed change in calibration frequencies for reactor vessel shroud level and high drywell pressure to be acceptable.

ITEM 3

The proposal would change the surveillance requirements for Logic System Functional Tests of the ECCS Actuation Instrumentation (TS Table 4.2.1), HPCI and RCIC Isolation Instrumentation (TS Table 4.2.2), Reactor Building Ventilation and Standby Gas Treatment System Isolation Instrumentation (TS Table 4.2.3) and Control Rod Block Instrumentation (TS Table 4.2.5) by removing references to "Note 2" from these tables. "Note 2" states that, "During each refueling outage, simulated automatic actuation which opens all pilot valves shall be performed such that each trip system logic can be verified independent of its redundant counterpart."

The licensee states that the note is not applicable to these particular tables and that its removal is proposed to improve and simplify the Technical Specifications.

The staff review of the proposed change determined that the basis for "Note 2" was to exempt the actuation of pilot valves from the Logic System Functional Tests if such actuation would produce an unacceptable plant transient during normal power operations. Staff review of the functional control diagrams of the affected systems determined that

"Note 2" is not applicable to CS and LPCI Actuation Instrumentation, the HPCI and RCIC Isolation Instrumentation and the Control Rod Block Instrumentation since these systems do not employ pilot valves. In addition, for the HPCI Actuation Instrumentation and the Reactor Building Ventilation and Standby Gas Treatment System Isolation Instrumentation, the review determined that these system's pilot valves could be tested during normal power operations without producing an unacceptable plant transient. The staff therefore concludes that the deletion of "Note 2" from the tables indicated is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the surveillance requirements of facility components located within the restricted area, as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: 1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and 2) such activities will be conducted in compliance with the Commission's regulations; and 3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATE: March 29, 1989

PRINCIPAL CONTRIBUTORS: Lawrence J. Wink
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