

MAR 31 1987

Docket No. 50-271

Mr. R. W. Capstick
Licensing Engineer
Vermont Yankee Nuclear Power
Corporation
1671 Worcester Road
Framingham, Massachusetts 01701

Dear Mr. Capstick:

The Commission has issued the enclosed Amendment No. 99 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. The amendment consists of changes to the Technical Specifications in partial response to your application dated January 30, 1979, as clarified by letter dated November 27, 1984. The remainder of the requests contained in your January 30, 1979 application and November 27, 1984 clarification pertain to our continuing review of your inservice Inspection and Inservice Testing programs, and we are acting upon these requests separately.

The amendment changes the Technical Specifications to require that the inservice examinations of the piping, components, and their supports be performed in accordance with 10 CFR 50.55a(b)(2) rather than by an earlier edition of the ASME Code Section XI.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by

Vernon L. Rooney, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No. 99 to License No. DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. R. W. Capstick
Vermont Yankee Nuclear Power Corporation

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Vermont Yankee Nuclear Power Corporation (the licensee) dated January 30, 1979, as supplemented November 27, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-28 is hereby amended to read as follows:

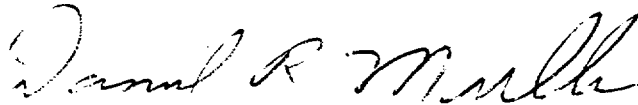
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(2) Technical Specifications

The Technical Specifications, contained in Appendix A, as revised through Amendment No. 99 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "Daniel R. Muller", is written over the typed name.

Daniel R. Muller, Project Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 99

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages

108a-added

109

3.6 LIMITING CONDITION FOR OPERATION

E. Structural Integrity and Operability Testing

The structural integrity and the operability of the safety-related systems and components shall be maintained at the level required by the original acceptance standards throughout the life of the plant.

4.6 SURVEILLANCE REQUIREMENT

E. Structural Integrity and Operability Testing

1. Inservice inspection of safety-related components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
2. Operability testing of safety-related pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

3.6 LIMITING CONDITION FOR OPERATION

F. Jet Pumps

1. Whenever there is recirculation flow with the reactor in the startup/hot standby or run modes, jet pump integrity and operability shall be checked daily by verifying that the following two conditions do not occur simultaneously:
 - a. The recirculations pump flow differs by more than 10% from the established speed-flow characteristics.
 - b. The indicated total core flow is more than 10% greater than the core flow value derived from established power-core flow relationship.
2. Additionally, when operating with one recirculation pump, the diffuser to lower plenum differential pressure shall be checked daily, and the differential pressure of any jet pump in the idle loop shall not vary by more than 10% from established patterns.

4.6 SURVEILLANCE REQUIREMENT

F. Jet Pumps

1. Whenever the reactor is in the startup/hot standby or run modes, all jet pumps shall be intact and all operating jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in a cold situation condition within 24 hours.
2. Flow indication from each of the twenty jet pumps shall be verified prior to initiation of reactor startup from a cold shutdown condition.
3. The indicated core flow is the sum of the flow indication from each of the twenty jet pumps. If flow indication failure occurs for two or more jet pumps, immediate corrective action shall be taken. If flow indication for all but one jet pump cannot be obtained within 12 hours an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

10 CFR 50.55a(g) requires ASME Code Class 1, 2 and 3 components (including supports) of a boiling water-cooled nuclear reactor facility to meet the requirements set forth in Section XI, Rules for Inservice Inspection of Nuclear Power Plant Components, of the ASME Boiler and Pressure Vessel Code. Each facility is required to have an inspection program plan which is updated every ten years to meet the requirements of the latest approved edition and addenda of Section XI. This program plan is submitted to the NRC for review of the licensee's compliance with the Regulation which entails verifying that the correct systems, sampling plans, and non-destructive examination methods have been incorporated, and evaluating the efficacy of proposed alternative examinations and tests to be performed in lieu of the requirements.

By letter dated January 30, 1979, Vermont Yankee Nuclear Power Corporation proposed a modification to the Technical Specifications for the Vermont Yankee Nuclear Power Station. Amongst the many changes proposed was a change to section 4.6.E which presently requires that the inservice examinations of the piping, components, and their supports be performed in accordance with an early edition of the ASME Code Section XI. Because this request was included in a group of requests which are still under review, and a delay acting on this request would result in unnecessary and burdensome testing, we are acting on this request separately from the other requests.

2.0 EVALUATION

Technical Specification 4.6.E for the Vermont Yankee Nuclear Power Station requires an inservice inspection program which essentially conforms to the requirements of the 1974 Edition ASME Section XI, up to and including the 1975 Summer Addenda except piping examinations (Cat. F & J) shall be per the requirements of ASME Code Section XI, 1974 Edition, 1976 Summer Addenda, Appendix III. The regulation 10 CFR 50.55a(b) requires that the plant be examined in accordance with the 1977 Edition of Section XI with Addenda to Summer of 1979. Since the Technical Specification examination is to a different edition of the ASME Code than the regulation requirements, the plant must examine to both sets of requirements with the inherent conflicts and extra overlapping examinations. Examination to both sets of requirements, provides marginal or no extra benefits considering the burden of doing the extra examination.

The licensee proposes to modify the requirements of 4.6.E to require that the inservice inspection of the plant be performed in accordance with Section XI of the Code Edition required by the regulation 10 CFR 50.55a(g). This will keep the Technical Specification requirement in accordance with the regulations as modified throughout the life of the plant without the extra burden of conforming to two different editions of the ASME Boiler and Pressure Vessel Code Section XI. This change is in accordance with the regulations and is acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Turovlin

Dated March 31, 1987