September 9, 1988

Docket No. 50-271

Mr. R. W. Capstick
Licensing Engineer
Vermont Yankee Nuclear Power
Corporation
1671 Worcester Road
Framingham, Massachusetts 01701

Dear Mr. Capstick:

SUBJECT: ISSUANCE OF AMENDMENT NO. 108 TO DRP-28 - VERMONT YANKEE NUCLEAR POWER STATION (TAC#68731)

The Commission has issued the enclosed Amendment No. 108 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. This amendment consists of changes to the Technical Specifications in response to your application dated May 23, 1988 with clarification provided by letter dated August 15, 1988.

This amendment changes the Technical Specifications to permit the use of the fuel type designated as GE 8X8EB.

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Vernon L. Rooney, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:
1. Amendment No. 108 to DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

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Please make changes shown in SER 9/8/88 Changes made 9/8/88

OFC	:PDI-3	:PDI-3	:OGC	:DIR/PDI-3	: RSB		
NAME	:VRooney:ck	:MRushbrook	:	:RWessman	:WHodges	:	:
DATE	:8/26/88	:8/29/88	:8/29/88	:9/9/88	:8/26/88	:	:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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cc w/enclosures:
See next page

Amendment No. 108 to DPR-28 - Vermont Yankee Nuclear Power Station

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Vermont Yankee Nuclear Power Station

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Adjudicatory File (2)
Atomic Safety and Licensing Board
Panel Docket
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Vermont Yankee Nuclear Power Corporation (the licensee) dated May 23, 1988 as supplemented August 15, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-28 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications, contained in Appendix A, as revised through Amendment No.108 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 30 days after the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard H. Wessman, Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 9, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
180a	180a
180b	180b
180f	180f
-	180-n7*
-	180-n8*
189	189

*Indicates new page

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.11 REACTOR FUEL ASSEMBLIES

Applicability:

The Limiting Conditions for Operation associated with the fuel rods apply to these parameters which monitor the fuel rod operating conditions.

Objective:

The Objective of the Limiting Conditions for Operation is to assure the performance of the fuel rods.

Specifications:

A. Average Planar Linear Heat Generation Rate (APLHGR)

During steady state power operation, the APLHGR for each type of fuel as a function of average planar exposure shall not exceed the limiting values shown in Tables 3.11-1A through J. For single recirculation loop operation, the limiting values shall be the values from Tables 3.11-1B through E and Table 3.11-1G through J listed under the heading "Single Loop Operation." These values are obtained by multiplying the values for two loop operation by 0.83. If at any time during steady-steate operation it is determined by normal surveillance that the limiting value for APLHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed

4.11 REACTOR FUEL ASSEMBLIES

Applicability:

The Surveillance Requirements apply to the parameters which monitor the fuel rod operating conditions.

Objective:

The Objective of the Surveillance Requirements is to specify the type and frequency of surveillance to be applied to the fuel rods.

Specifications:

A. Average Planar Linear Heat Generation Rate (APLHGR)

The APLHGR for each type of fuel as a function of average planar exposure shall be determined daily during reactor operation at ≥25% rated thermal power.

LIMITING CONDITIONS FOR OPERATION

limits. If the APLHGR isnot returned to within prescribed limits within two (2) hours, the reactor shall be brought to the shutdown conditions within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

B. Linear Heat Generation Rate (LHGR)

During steady state power operation, the linear heat generation rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the maximum allowable LHGR specified in Table 1.

If at any time during steady state operation it is determined by normal surveillance that the limiting value for LHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. If the LHGR is not returned to within the prescribed limits within two (2) hours, the reactor shall be brought to shutdown condition within 36 hours. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits.

SURVEILLANCE REQUIREMENTS

B. Linear Heat Generation Rate (LHGR)

The LHGR as a function of core height shall be checked daily during reactor operation at ≥25% rated thermal power.

C. Minimum Critical Power Ratio

MCPR shall be determined daily during reactor power operation at ≥25% rated thermal power and following any change in power level or distribution that would cause operation with a limiting control rod pattern as described in the bases for Specification 3.3.B.6.

Table 1

SIGNIFICANT INPUT PARAMETERS TO THE
LOSS-OF-COOLANT ACCIDENT ANALYSIS

Plant Parameters:

Core Thermal Power	1664 MWt, which corresponds to 105% of rated steam flow
Vessel Steam Output	6.75×10^6 lbm/h, which corresponds to 105% of rated steam flow
Vessel Steam Dome Pressure	1055 psia
Recirculation Line Break Area for Large Breaks - Discharge	2.26 ft^2 (DBA)
- Suction	4.14 ft^2
Number of Drilled Bundles	220

Fuel Parameters:

	Fuel Type	Fuel Bundle Geometry	Peak Technical Specification Linear Heat Generation Rate (kW/ft)	Design Axial Peaking Factor	Initial Minimum Critical Power Ratio*
A.	7D230	7 x 7	18.5	1.4	1.2
B.	8D219	8 x 8	13.4	1.4	1.2
C.	8D274L	8 x 8	13.4	1.4	1.2
D.	8D274H	8 x 8	13.4	1.4	1.2
E.	8D274 (High Gd)	8 x 8	13.4	1.4	1.2
F.	LTA	8 x 8	13.4	1.4	1.2
G.	8DPB289 & P8DPB289	8 x 8	13.4	1.4	1.2
H.	BP8DRB299	8 x 8	13.4	1.4	1.2
I.	BD324B	8 x 8EB	14.4	1.4	1.2
J.	BD326B	8 x 8EB	14.4	1.4	1.2

* To account for the 2% uncertainty in bundle power required by Appendix K, the SCAT calculation is performed with an MCPR of 1.18 (i.e., 1.2 divided by 1.02) for a bundle with an initial MCPR of 1.20.

TABLE 3.11-1I

MAPLHGR Versus Average Planar Exposure

Plant: Vermont Yankee

Fuel Type: BD324B

Average Planar Exposure (MWd/t)	MAPLHGR (kW/ft) for Two Loop Operation			
	Majority Lattice	Shutdown Margin Zone	Power Peaking Zone	Natural Ends
200.0	11.76	11.24	11.71	11.50
1,000.0	11.90	11.42	11.83	11.30
2,000.0	12.05	11.61	11.96	11.28
3,000.0	12.21	11.85	12.15	11.33
5,000.0	12.51	12.17	12.40	11.47
7,000.0	12.63	12.54	12.63	11.61
10,000.0	12.80	12.80	12.80	11.72
14,400.0	12.80	12.80	12.80	11.15
15,000.0	12.75	12.74	12.74	11.07
20,000.0	12.07	12.05	12.06	10.29
25,000.0	11.41	11.39	11.40	9.50
35,000.0	10.14	10.12	10.12	7.93
43,360.0	8.80	8.73	8.74	4.66
50,000.0	6.08	5.99	6.02	-

Source: NEDO-21697, August 1977 (Revised)

Average Planar Exposure (MWd/t)	MAPLHGR (kW/ft) for Single Loop Operation*			
	Majority Lattice	Shutdown Margin Zone	Power Peaking Zone	Natural Ends
200.0	9.76	9.32	9.71	9.54
1,000.0	9.87	9.47	9.81	9.37
2,000.0	10.00	9.63	9.92	9.36
3,000.0	10.13	9.83	10.08	9.40
5,000.0	10.38	10.10	10.29	9.52
7,000.0	10.48	10.40	10.48	9.63
10,000.0	10.62	10.62	10.62	9.72
14,400.0	10.62	10.62	10.62	9.25
15,000.0	10.58	10.57	10.57	9.18
20,000.0	10.01	10.00	10.00	8.54
25,000.0	9.47	9.45	9.46	7.88
35,000.0	8.41	8.39	8.39	6.58
43,360.0	7.30	7.24	7.25	3.86
50,000.0	5.04	4.97	4.99	-

* MAPLHGR for single loop operation is obtained by multiplying MAPLHGR for two loop operation by 0.83.

VYNPS

TABLE 3.11-1J

MAPLHGR Versus Average Planar Exposure

Plant: Vermont Yankee

Fuel Type: BD326B

Average Planar Exposure (MWd/t)	MAPLHGR (kW/ft) for Two Loop Operation			
	Majority Lattice	Shutdown Margin Zone	Power Peaking Zone	Natural Ends
200.0	11.80	11.35	11.76	11.50
1,000.0	11.86	11.42	11.79	11.30
2,000.0	11.97	11.56	11.88	11.28
3,000.0	12.10	11.74	11.99	11.33
5,000.0	12.48	12.16	12.33	11.47
7,000.0	12.69	12.66	12.69	11.61
10,000.0	12.90	12.90	12.90	11.72
14,400.0	12.90	12.90	12.90	11.15
15,000.0	12.84	12.82	12.82	11.07
20,000.0	12.14	12.12	12.12	10.29
25,000.0	11.46	11.44	11.45	9.50
35,000.0	10.17	10.15	10.16	7.93
43,360.0	8.94	8.87	8.91	4.66
50,000.0	6.25	6.17	6.22	-

Source: NEDO-21697, August 1977 (Revised)

Average Planar Exposure (MWd/t)	MAPLHGR (kW/ft) for Single Loop Operation*			
	Majority Lattice	Shutdown Margin Zone	Power Peaking Zone	Natural Ends
200.0	9.79	9.42	9.76	9.54
1,000.0	9.84	9.47	9.78	9.37
2,000.0	9.93	9.59	9.86	9.36
3,000.0	10.04	9.74	9.95	9.40
5,000.0	10.35	10.09	10.23	9.52
7,000.0	10.53	10.50	10.53	9.63
10,000.0	10.70	10.70	10.70	9.72
14,400.0	10.70	10.70	10.70	9.25
15,000.0	10.65	10.64	10.64	9.18
20,000.0	10.07	10.05	10.05	8.54
25,000.0	9.51	9.49	9.50	7.88
35,000.0	8.44	8.42	8.43	6.58
43,360.0	7.42	7.36	7.39	3.86
50,000.0	5.18	5.12	5.16	-

* MAPLHGR for single loop operation is obtained by multiplying MAPLHGR for two loop operation by 0.83.

5.5 Spent and New Fuel Storage

A. The new fuel storage facility shall be such that the effective multiplication factor (K_{eff}) of the fuel when dry is less than 0.90 and when flooded is less than 0.95.

B. The K_{eff} of the fuel in the spent fuel storage pool shall be less than or equal to 0.95.

C. Spent fuel storage racks may be moved (only) in accordance with written procedures which ensure that no rack modules are moved over fuel assemblies.

D. The number of spent fuel assemblies stored in the spent fuel pool shall not exceed 2,000.

E. The maximum core geometry infinite lattice multiplication factor of any segment of the fuel assembly stored in the spent fuel storage pool or the new fuel storage facility shall be less than or equal to 1.31 at 20°C.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 108

TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letter dated May 23, 1988 with clarification submitted August 15, 1988 the Vermont Yankee Nuclear Power Corporation (the licensee) requested changes to the Vermont Yankee Radiological Technical Specifications (TS) as incorporated in Facility Operating License DPR-28. These changes:

1. Revise Limiting Conditions for Operation (LCO) 3.11A to allow the addition of two new tables of APLHGR limits for the two GE 8X8EB fuel types to be used in the next operating cycle.

2. Revise LCO 3.11B to include vendor recommended LHGR limiting values for the two GE 8X8EB fuel types to be used in the next operating cycle.

3. Revise Section 5.5E to specify the peak uncontrolled infinite lattice multiplication factor appropriate for the two GE 8X8EB fuel types as a means of assuring compliance with Section 5.5A and 5.5B.

2.0 EVALUATION

With regard to Part 1 of the proposed change: the APLHGR limits for the two new GE 8X8EB fuel types, as shown on Tables 3.11-1I and 3.11-1J, were calculated with the same methods used to calculate the limits used currently for other fuel types in the Technical Specification, Tables 3.11-1A through 3.11-1H.

The two GE 8X8EB fuel types proposed for use in Vermont Yankee have multiple lattices which are arranged axially. Appropriate MAPLHGR limits, which have been determined by approved thermal-mechanical and loss of coolant accident (LOCA) analyses calculations, will be applied to each of these regions. There was extensive interaction between the staff, GE and utilities in deciding on an acceptable format for presentation of this information, suitable for plant use and staff requirements for TS. The process computer contains, and acts on, full details of the MAPLHGR information. The agreed upon TS present axial lattice MAPLHGR as a function of burnup. A proprietary report, reviewed by the staff and available to the Vermont Yankee engineering staff, provides complete details of the lattice definitions and MAPLHGR limits (reference 3).

With regard to Part 2 of the proposed change:

The new fuel is the GE extended burnup fuel, GE 8X8EB. This fuel type has been approved in the Safety Evaluation Report for Amendment 10 to GESTAR II (Ref 1 and 2). The specific descriptions of this fuel have been submitted in reference 3. The fuel designation is BD224 and BD326B. The specific fuel description is acceptable.

The proposed LHGR limit for the GE8X8EB fuel is 14.4 kW/ft (rather than the 13.4 for other GE 8X8 fuel). This LHGR has been reviewed and accepted for this fuel in the GE extended burnup fuel review (Ref. 1). (See the referrals in Reference 1 to References 18 and 19. These references are responses to questions and presentations relating to the GE 8X8EB fuel which provide information on the 14.4 kW/ft LHGR.) This LHGR is acceptable for use of this fuel in the Vermont Yankee reactor core.

With regard to Part 3 of the proposed change: Section 5.5E was added to the Technical Specifications by Amendment 37, in order to provide a method of ensuring compliance with the effective multiplication factor safety limit of less than or equal to 0.95 for fuel storage stated in Section 5.5B of the Technical Specifications. The current 16 grams of U-235 per axial centimeter stated in Section 5.5E is not the best measure of the primary variable which affects the effective multiplication factor of the stored fuel.

In order to more directly control the reactivity worth of fuel assemblies that may contain varying amounts of poison, a more useful variable is the maximum, cold, infinite lattice multiplication factor K_∞. The approach of using a K_∞ design basis has been approved in other applications, and is used in the staff approved General Electric reload analysis approach (as given in GESTAR II, NEDE-24011-P-A-8, May 1986).

In the staff safety evaluation supporting License Amendment No. 104, dated May 20, 1988, the staff considered the use of a k_∞ basis for the Vermont Yankee spent fuel pool and in Amendment 104 authorized the storage of such fuel. The safety evaluation states that "the transfer to a fuel assembly K_∞ value of 1.31 is acceptable."

Part 3 of the proposed change, which substitutes a k_∞ limit of 1.31 for the axial enrichment design limit is, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

1. Letter (and attachment) from C. Thomas, NRC, to J. Charnley, GE, dated May 28, 1985, "Acceptance for Referencing of Licensing Topical Report NEDE-24011-P-A-6, Amendment 10".

2. GESTAR II, NEDE-24011, Revision 8, "General Electric Standard Application for Reactor Fuel".

3. Letter (and attachment) from R.W. Capstick, VYNPC, to U.S.N.R.C Document Control Desk, dated August 15, 1988, "Response to NRC Request for Supporting Document - Vermont Yankee Proposed Change Number 144".

Principal Contributor: V. Rooney

Dated: September 9, 1988