

Docket No.: 50-271

December 29, 1987

Mr. R. W. Capstick
Licensing Engineer
Vermont Yankee Nuclear Power
Corporation
1671 Worcester Road
Framingham, Massachusetts 01701

SUBJECT: ISSUANCE OF AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO.
DPR-28 VERMONT YANKEE NUCLEAR POWER STATION (TAC NO. 65338)

Dear Mr. Capstick:

The Commission has issued the enclosed Amendment No. 101 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. The amendment consists of changes to the Technical Specifications in response to your application dated April 28, 1987, as clarified by letter dated November 2, 1987.

The amendment revises the Technical Specifications to reflect administrative changes to Section 6 of the Technical Specifications. More specifically these changes include:

1. Reorganization of the current Chemistry and Health Physics Department into two separate departments and a change to the Plant Operations Review Committee (PORC) membership to reflect the two supervisors in this area.
2. An administrative correction to include a change previously granted in Amendment 79, but inadvertently deleted in Amendment 87.
3. Elimination of a reference to a nonexistent group designation and clear definition of authority for designating PORC alternates.
4. A revision of the authority regarding review and approval of procedures.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely, *15/*

Vernon L. Rooney, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

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P PDR

Enclosures:

1. Amendment No. 101 to License No. DPR-28
2. Safety Evaluation

cc w/enclosures: See next page

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|------|--------------|--------------|-------------------------------|-------------------|---|
| OFC | : PDI-3 | : PDI-3 | : ACTDIR/PDI-3: OGC-Bethesda: | : | : |
| NAME | : VRooney:lm | : MRushbrook | : MFairfite | : <i>Woodhead</i> | : |
| DATE | : 11/11/87 | : 11/ /87 | : 11/29/87 | : 12/15/87 | : |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in dark ink, appearing to read "V. Rooney".

Vernon L. Rooney, Project Manager
Project Directorate I-3
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 101 to License No. DPR-28
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. R. W. Capstick

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Vermont Yankee Nuclear Power Station

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Mr. R. W. Capstick

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ISSUANCE OF AMENDMENT NO. 101 FACILITY OPERATING LICENSE NO. DPR-28

DISTRIBUTION:

Docket File 50-271 ←

PDI-3 r/f

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ARM/LFMP

F. Allenspach



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Vermont Yankee Nuclear Power Corporation (the licensee) dated April 28, 1987, as clarified by letter dated November 2, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-28 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications, contained in Appendix A, as revised through Amendment No. 101, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Vernon L. Rooney, Acting Director
Project Directorate I-3
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: DEC 29 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 INTRODUCTION

By letters dated April 28 and November 2, 1987, the Vermont Yankee Nuclear Power Corporation, the licensee for the Vermont Yankee Nuclear Power Station, requested amendment of the Administrative Controls section of the Vermont Yankee Technical Specifications to accommodate certain changes in organization and responsibility. In the following section are brief descriptions and evaluations of the requested changes.

2.0 EVALUATION

2.1. Figure 6.1.2 - Plant Operating Organization

The plant operating organization has been revised by splitting the Chemistry and Health Physics Department into two separate departments: the Radiation Protection Department and the Chemistry Department.

We find this change acceptable because it meets the acceptance criteria of Sections 13.1.2-13.1.3 of NUREG-0800, Standard Review Plan.

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2.2 Section 6.1.D.5. - Organization

The licensee has revised this section to indicate that the qualification requirements for the Radiation Protection Supervisor or the Plant Health Physicist shall meet, or exceed, the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).

We find this change acceptable because it maintains a qualification requirement in the Radiation Protection Department that meets the staff position in Regulatory Guide 1.8, Revision 1 (September 1975).

2.3 Section 6.2.A.1 - Plant Operation Review Committee - Membership

The licensee has proposed deleting the now nonexistent Chemistry and Health Physics Supervisor and the Health Physicist and by adding the new Chemistry Supervisor and Radiation Protection Supervisor to the committee membership. The total PORC membership remains the same.

We find this change acceptable because it maintains a balanced representation of disciplines.

2.4 Section 6.2.A.5 - Plant Operations Review Committee - Alternatives

The licensee has deleted from this section a reference to a no longer functioning plant group (Technical Services Group) and added that alternates for committee members will be selected by the Plant Manager.

We find this requested change acceptable because it is consistent with the Standard Technical Specifications.

2.5 Section 6.5. C - Procedures

The licensee has revised Section 6.5.C to remove the phrase "approved by the Plant Manger, or his designee and the Manager of Operations" and has replaced this with "approved by the applicable Department Supervisor and the Plant Manager or his designee (superintendent level as shown in Figure 6.1.2)."

We find this requested change acceptable because it meets the staff position that procedure approval be by predesignated individuals at a level no lower than the Vermont Yankee Plant Superintendent level.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Allenspach and V. Rooney

Dated: December 29, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 101

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by the captioned amendment number and contains marginal lines indicating the area of change.

| <u>REMOVE</u> | <u>INSERT</u> |
|---------------|---------------|
| 190 | 190 |
| 190-a | 190-a |
| 192 | 192 |
| 194 | 194 |
| 195 | 195 |
| 206 | 206 |

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6.0 ADMINISTRATIVE CONTROLS

Administrative controls are the written rules, orders, instructions, procedures, policies, practices, and the designation of authorities and responsibilities by the management to obtain assurance of safety and quality of operation and maintenance of a nuclear power reactor. These controls shall be adhered to.

6.1 ORGANIZATION

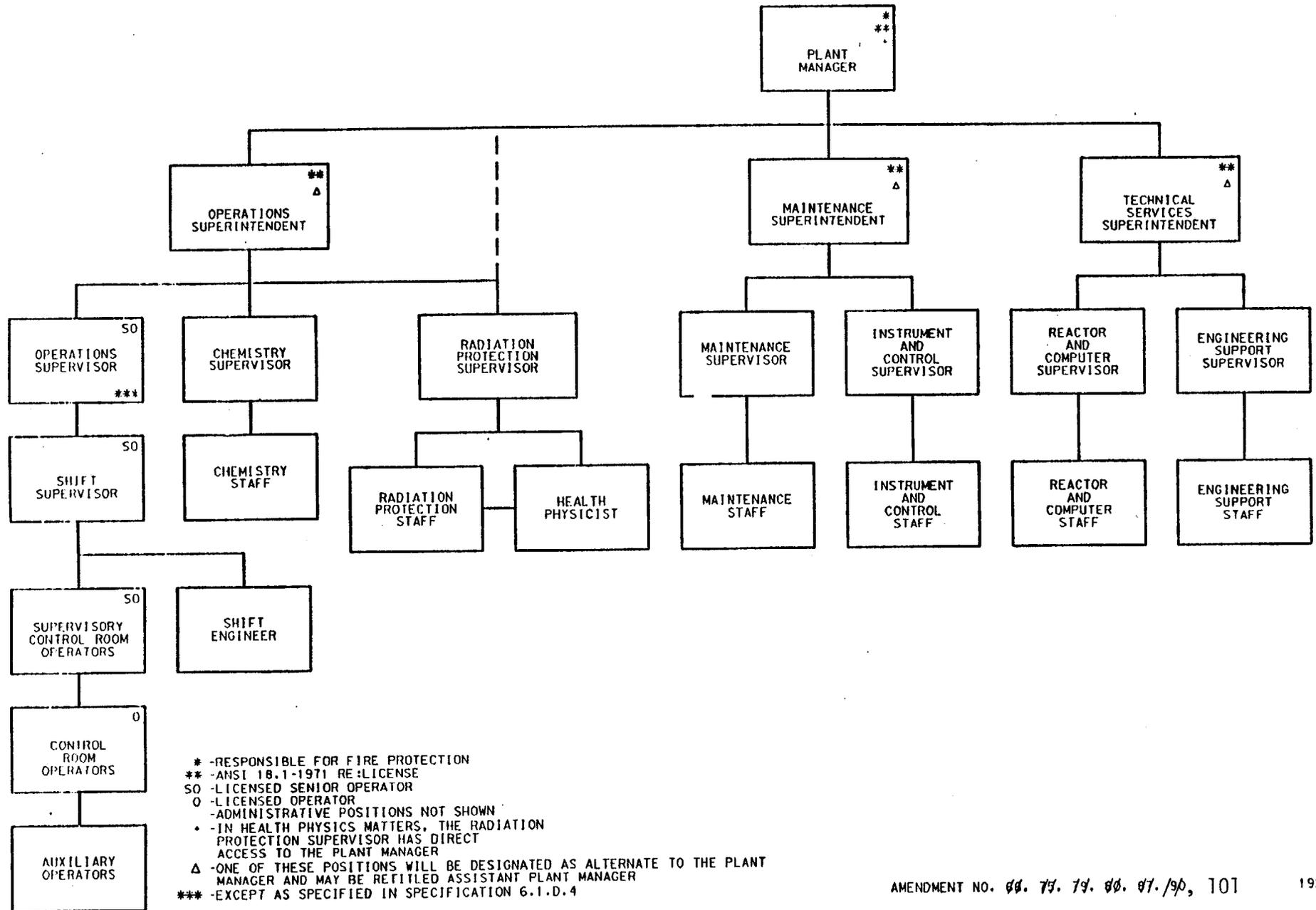
- A. The Plant Manager has on-site responsibility for the safety and efficient operation of the facility. Succession to this responsibility during his absence shall be delegated in writing.
- B. The portion of the corporate management which relates to the operation of this plant is shown in Figure 6.1.1.
- C. In all matters relating to the operation of the plant and to those Technical Specifications, the Plant Manager shall report to and be directly responsible to the Manager of Operations.
- D. Conduct of operations of the plant is shown in Figure 6.1.2 and will be in accordance with the following minimum conditions.
 1. An individual qualified in radiation protection procedures shall be present on-site at all times when there is fuel in the reactor.
 2. Minimum shift staffing on-site shall be in accordance with Table 6.1.1.
 3. A dedicated, licensed Senior Operator shall be in charge of any reactor core alteration.
 4. Qualifications with regard to educational background experience, and technical specialities of the key supervisory personnel listed below shall apply and be maintained in accordance with the levels described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants".
 - a. Plant Manager
 - b. Operations Superintendent
 - c. Technical Services Superintendent
 - d. Maintenance Superintendent
 - e. Chemistry Supervisor
 - f. Radiation Protection Supervisor
 - g. Operations Supervisor (See Item D.7, Page 190a)
 - h. Reactor and Computer Supervisor
 - i. Maintenance Supervisor
 - j. Instrument and Control Supervisor
 - k. Shift Supervisors

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5. The Radiation Protection Supervisor or Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).
 6. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.
 7. If the Operations Supervisor does not possess a Senior Operator License, then an Assistant Operations Supervisor shall be designated that does possess a Senior Operator License. All instructions to the shift crews involving licensed activities shall than be approved by designated Assistant Operations Supervisor.
- E. A Fire Brigade of at least 5 members shall be maintained on-site at all times. # This excludes 2 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.

#Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

Figure 6.1.2
PLANT OPERATIONS ORGANIZATION



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6.2 REVIEW AND AUDIT

Organizational units for the review and audit of plant operations shall be constituted and have the responsibilities and authorities outlined below:

A. Plant Operations Review Committee

1. Membership

- a. Chairman: Plant Manager
- b. Vice-Chairman: Operations Superintendent
- c. Vice-Chairman: Technical Services Superintendent
- d. Vice-Chairman: Maintenance Superintendent
- e. Engineering Support Supervisor
- f. Operations Supervisor
- g. Maintenance Supervisor
- h. Reactor and Computer Supervisor
- i. Chemistry Supervisor
- j. Instrument and Control Supervisor
- k. Radiation Protection Supervisor

2. Qualifications

The qualifications of the regular members of the Plant Operations Review Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level at least equal to or higher than as described in Specification 6.1.

3. Meeting Frequency: Monthly, and as required, on call of the Chairman.

4. Quorum: Chairman or Vice-Chairman plus four members or their designated alternates.

NOTE: For purposes of satisfying a quorum, a Vice-Chairman may be considered a member providing that Vice-Chairman is not presiding over the meeting.

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5. Designated alternates shall be from other plant personnel in the appropriate disciplines or as selected by the Plant Manager; however, there shall be no more than three (3) alternates serving on the committee at any one time.

6. Responsibilities

- a. Review proposed normal, abnormal, and emergency operating procedures. Review all proposed maintenance procedures and proposed changes to those procedures; and any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review proposed tests and experiments.
- c. Review proposed changes to Technical Specifications.
- d. Review proposed changes or modifications to plant systems or equipment, which changes would require a change in procedures in (a) above.
- e. Review plant operations to detect any potential safety hazards.
- f. Investigate reported instances of violations of Technical Specifications, such investigations to include reporting, evaluation, and recommendations to prevent recurrence, to the Manager of Operations (see Figure 6.1.1).
- g. Perform special reviews and investigations and render reports thereon as requested by the Chairman of the Nuclear Safety Audit and Review Committee.

7. Authority

- a. The Plant Operation Review Committee shall be advisory.
- b. The Plant Operation Review Committee shall recommend to the Plant Manager approval or disapproval of proposals under items 6 (a) through (d) above.
 1. In the event of disagreement between the recommendations of the Plant Operation Review Committee and the actions contemplated by the Plant Manager, the course determined by the Plant Manager to be the more conservative will be followed with immediate notification to the Manager of Operations.

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- C. Procedures prepared for A and B above shall be reviewed and approved by the applicable Department Supervisor and the Plant Manager, or his designee (superintendent level as shown on Figure 6.1.2).
- D. Temporary changes to procedures described in Specification 6.5.A above which do not change the intent of the original procedure, may be made with the concurrence of two individuals holding senior operator licenses. Such changes shall be documented and subsequently reviewed by the PORC and approved by the Plant Manager or his designee.
- E. Temporary changes to procedures described in Specification 6.5.B may be made with the concurrence of an individual holding a senior operator license and the health physicist on duty.
- F. Licensed radioactive sealed sources shall be leak tested for contamination. Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement state as follows:
 - 1. Each licensed sealed source, except startup sources previously subjected to core flux, containing radioactive materials, other than Hydrogen 3, with half-life greater than thirty days and in any form, other than gas, shall be tested for leakage and/or contamination at intervals not to exceed six months.
 - 2. The periodic leak test required does not apply to sealed sources that are stored and are not being used. The sources exempted from this test shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer. In the absence of a certificate from a transferrer indicating that a leak test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.
 - 3. Each sealed startup source shall be tested within 31 days prior to being subjected to core flux and following repair or maintenance to the source.

The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.

Notwithstanding the periodic leak tests required by this Technical Specification, any licensed sealed source is exempt from such leak test when the source contains 100 microcuries or less of beta and/or gamma emitting material or 5 microcuries or less of alpha emitting material.

A special report shall be prepared and submitted to the Commission within 90 days if source leakage tests reveal the presence of ≥ 0.005 microcuries of removable contamination.