

Distribution
 [] ket File ✓
 [] PDR
 Local PDR
 ORB#2 Rdg.
 D. Eisenhut
 S. Norris
 V. Rooney
 OELD
 SECY
 L. J. Harmon 2
 T. Barnhart 4
 L. Schneider
 D. Brinkman
 ACRS 10
 OPA Clare Miles
 R. Diggs
 NSIC
 ASLAB
 Gray File
 5 Extra

February 17, 1983

Docket No. 50-271

Mr. J. B. Sinclair
 Licensing Engineer
 Vermont Yankee Nuclear Power
 Corporation
 1671 Worcester Road
 Framingham, Massachusetts 01701

HThompson,
 (DHFS)-2

Dear Mr. Sinclair:

The Commission has issued the enclosed Amendment No. 75 to Facility Operating License No. DPR-28 for Vermont Yankee Nuclear Power Station. This amendment consists of changes to the Technical Specifications in response to your applications dated July 22, 1982, September 27, 1982, and December 7, 1982 as clarified through subsequent discussions between the NRC staff and members of your staff.

These changes to the Technical Specifications pertain to limiting conditions of operation for the standby liquid control system and administrative controls related to organizational structure.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed By;

Vernon L. Rooney, Project Manager
 Operating Reactors Branch #2
 Division of Licensing

Enclosures:

1. Amendment No. 75 to DPR-28
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures
 See next page

8303090245 830217
 PDR ADOCK 05000271
 P PDR

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB#2	OELD	DHFS
SURNAME	S. Norris	V. Rooney:pr	D. Vassallo	G. Lainas	R. Bachmann	H. L. Thompson
DATE	2/1/83	2/1/83	2/2/83	2/2/83	2/7/83	2/2/83

Mr. J. B. Sinclair

cc:

Mr. W. F. Conway
President & Chief Executive Officer
Vermont Yankee Nuclear Power Corp.
R.D. 5, Box 169
Ferry Road
Brattleboro, Vermont 05301

W. P. Murphy, Vice President &
Manager of Operations
Vermont Yankee Nuclear Power Corp.
R. D. 5, Box 169
Ferry Road
Brattleboro, Vermont 05301

Mr. Louis Heider, V. P.
Vermont Yankee Nuclear Power Corp.
1671 Worcester Road
Framingham, Massachusetts 01701

U.S. Environmental Protection Agency
Region I Office
Regional Radiation Representative
JFK Federal Building
Boston, Massachusetts 02203

John A. Ritscher, Esquire
Ropes & Gray
225 Franklin Street
Boston, Massachusetts 02110

Public Service Board
State of Vermont
120 State Street
Montpelier, Vermont 05602

New England Coalition on Nuclear
Pollution
Hill and Dale Farm
R.D. 2, Box 223
Putney, Vermont 05346

Vermont Yankee Decommissioning
Alliance
53 Frost Street
Brattleboro, Vermont 05301

Mr. Walter Zaluzny
Chairman, Board of Selectman
P.O. Box 116
Vernon, Vermont 05354

Vermont Yankee Decommissioning
Alliance
5 State Street
Box 1117
Montpelier, Vermont 05602

J. P. Pelletier, Plant Manager
Vermont Yankee Nuclear Power Corp.
P.O. Box 157
Vernon, Vermont 05354

Resident Inspector
c/o U.S. NRC
P.O. Box 176
Vernon, Vermont 05453

Raymond N. McCandless
Vermont Division of Occupational
& Radiological Health
Administration Building
10 Baldwin Street
Montpelier, Vermont 05602

Vermont Public Interest Research
Group, Inc.
43 State Street
Montpelier, VT 05602

Honorable John J. Easton
Attorney General
State of Vermont
109 State Street
Montpelier, Vermont 05602

Ronald C. Haynes
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Vermont Yankee Nuclear Power Corporation (the licensee) dated July 22, September 27 and December 7, 1982 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-28 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 75 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 17, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Revise the Technical Specifications as follows:

REMOVE

39A
80
190
190a
191
192
194
206

INSERT

39A
80
190
190a
191
192
194
206

VYNPS

TABLE 3.2.1

RECIRCULATION PUMP TRIP ACTUATION INSTRUMENTATION

Recirculation Pump Trip - A & B (Note 1)

<u>Minimum Number of Operable Instrument Channels per Trip System</u>	<u>Trip Function</u>	<u>Trip Level Setting</u>	<u>Required Action When Minimum Conditions for Operation are not satisfied</u>
2	Low-Low Reactor Vessel Water Level	\geq 6' 10.5" above top of enriched fuel	Note 2
2	High Reactor Pressure	\leq 1150 psig	Note 2
2	Time Delays	\leq 10 seconds	Note 2
1	Trip Systems Logic	---	Note 2

Amendment No. ~~58, 66, 70~~, 75

3.4 LIMITING CONDITIONS FOR OPERATION**B. Operation with Inoperable Components**

From and after the date that a redundant component is made or found to be inoperable, reactor operation is permissible during the succeeding seven days unless such component is sooner made operable.

C. Liquid Poison Tank - Boron Concentration

The liquid poison tank shall contain a boron bearing solution that satisfied the volume concentration requirements of Figure 3.4.1 and the solution temperature, including that in the pump suction piping, shall be not less than the temperature presented in Figure 3.4.2.

D. If Specification 3.4 A or B are not met an orderly shutdown shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours.

E. If Specification 3.4.A, B, or C are not met, action shall be immediately initiated to correct the deficiency. If at the end of 12 hours the system has not been restored to full operability, then a shutdown shall be initiated with the reactor in cold shutdown within 24 hours of initial discovery.

4.4 SURVEILLANCE REQUIREMENTS

Disassemble and inspect one explosion valve so that it can be established that the valve is not clogged. Both valves shall be inspected in the course of two operating cycles.

Test that the setting of the system pressure relief valves is between 1400 and 1490 psig.

B. Operation with Inoperable Components

When a component becomes inoperable its redundant component shall be demonstrated to be operable immediately and daily thereafter.

C. Liquid Poison Tank - Boron Concentration

The solution volume and temperature in the tank shall be checked at least daily.

Boron concentration shall be determined at least once a month and at any time water or boron are added or if the solution temperature drops below the limits specified by Figure 3.4.2.

6.0 ADMINISTRATIVE CONTROLS

Administrative controls are the written rules, orders, instructions, procedures, policies, practices, and the designation of authorities and responsibilities by the management to obtain assurance of safety and quality of operation and maintenance of a nuclear power reactor. These controls shall be adhered to.

6.1 ORGANIZATION

- A. The Plant Manager has on-site responsibility for the safety and efficient operation of the facility. Succession to this responsibility during his absence shall be delegated in writing.
- B. The portion of the corporate management which relates to the operation of this plant is shown in Figure 6.1.
- C. In all matters relating to the operation of the plant and to those Technical Specifications, the Plant Manager shall report to and be directly responsible to the Manager of Operations.
- D. Conduct of operations of the plant is shown in Figure 6.1.2 and will be in accordance with the following minimum conditions (see Table 6.1.1).
 1. A licensed Senior Operator and an individual qualified in radiation protection procedures shall be present on-site at all times when there is fuel in the reactor.
 2. Licensed Operators on-site shall be in accordance with Table 6.1.1, one of which must be in the Control Room at all times when fuel is in the reactor.
 3. A licensed Senior Operator shall be in charge of any refueling operation.
 4. Qualifications with regard to educational background experience, and technical specialities of the key supervisory personnel listed below shall apply and be maintained in accordance with the levels described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants".

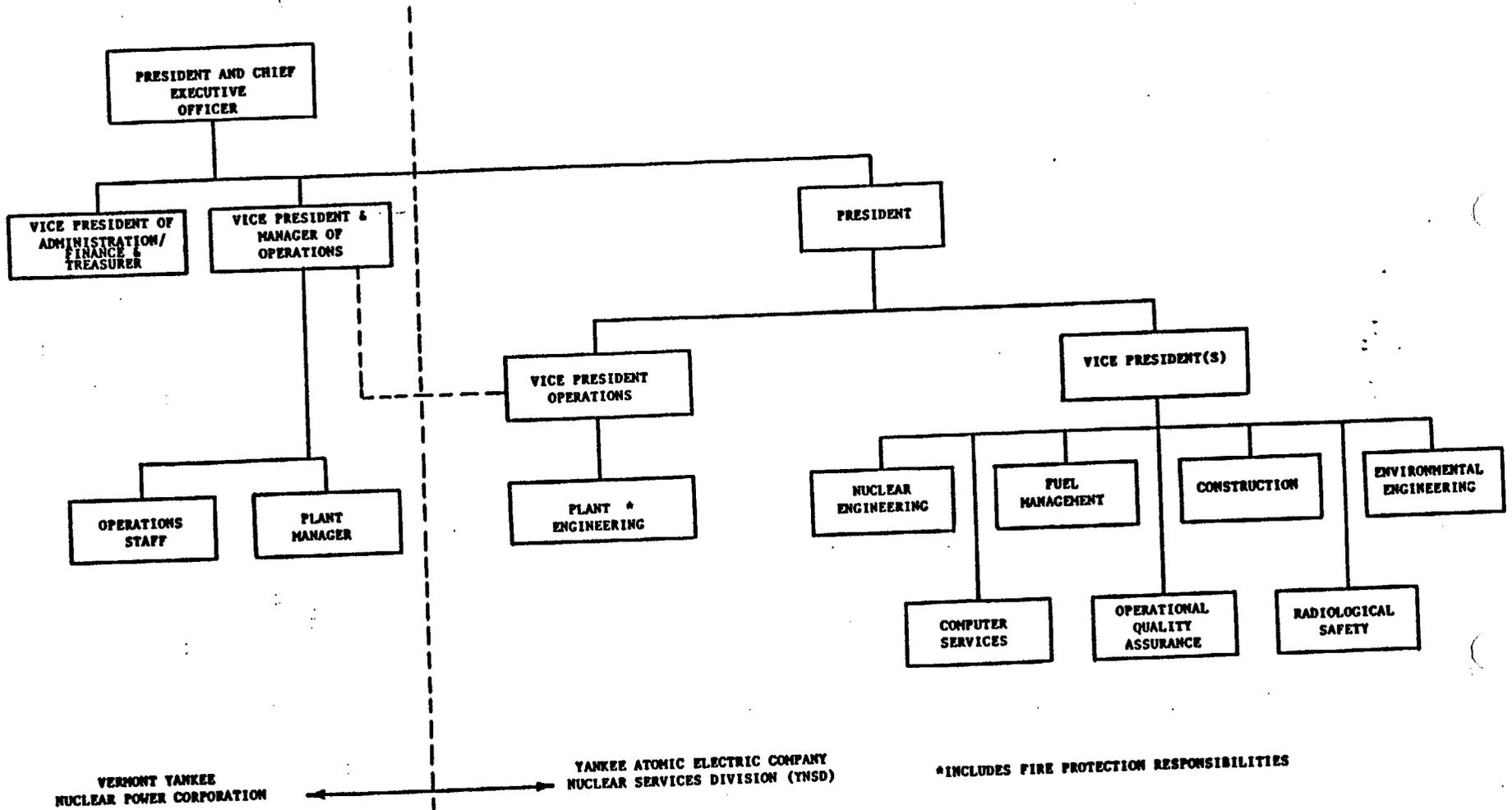
a. Plant Manager	f. Operations Supervisor
b. Operations Superintendent	g. Reactor and Computer Supervisor
c. Technical Services Superintendent	h. Maintenance Supervisor
d. Maintenance Superintendent	i. Instrument and Control Supervisor
e. Chemistry and Health Physics Supervisor	j. Shift Supervisors

VYNPS

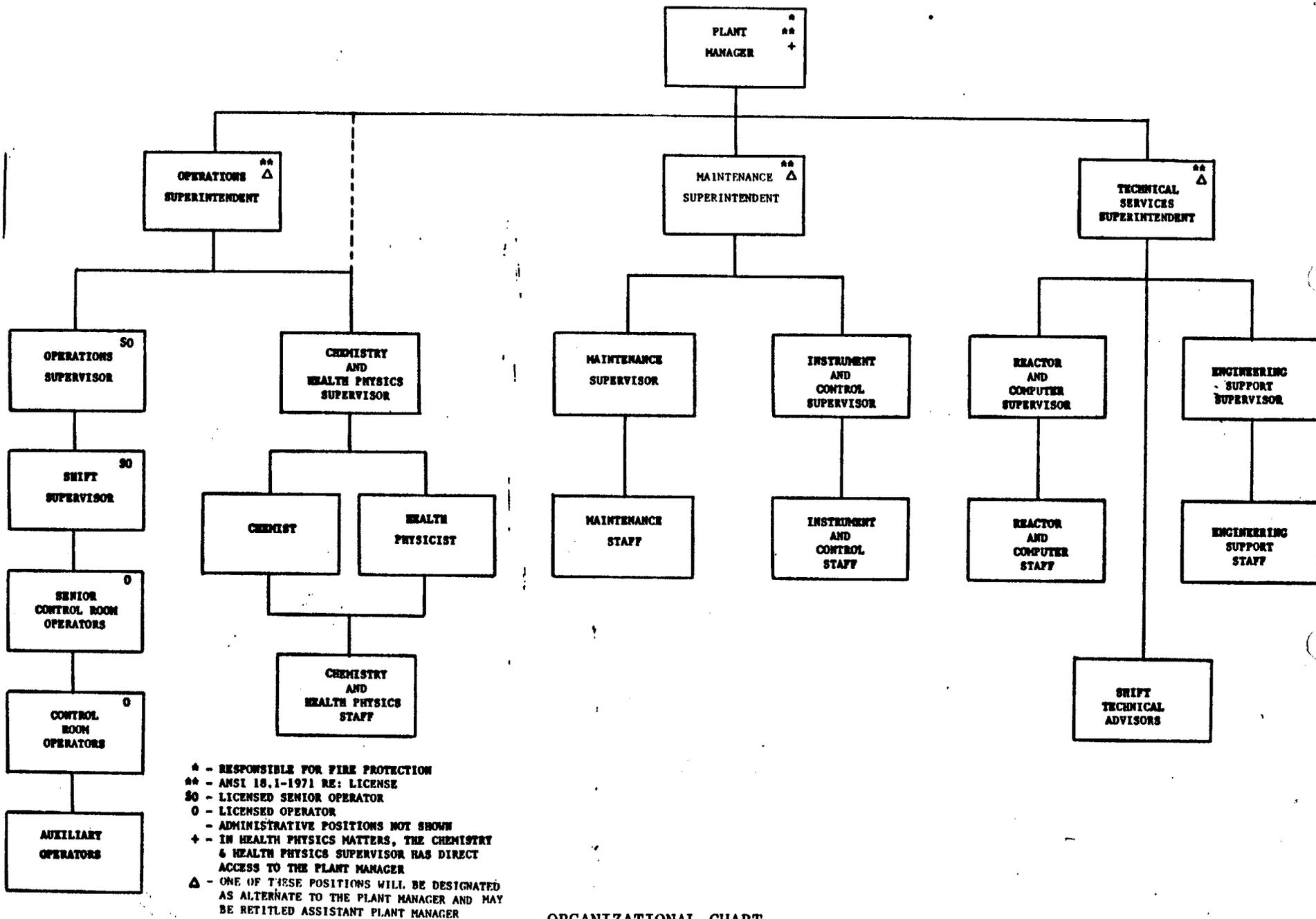
5. The Chemistry and Health Physics Supervisor or Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).
 6. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.
- E. A Fire Brigade of at least 5 members shall be maintained on-site at all times. #This excludes 2 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.

#Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

FIGURE 6.1.1
OFFSITE SUPPORT ORGANIZATION



VERMONT YANKEE NUCLEAR POWER STATION



ORGANIZATIONAL CHART
FIGURE 6.1.2

6.2 REVIEW AND AUDIT

Organizational units for the review and audit of plant operations shall be constituted and have the responsibilities and authorities outlined below:

A. Plant Operations Review Committee**1. Membership**

- a. Chairman: Plant Manager
- b. Vice-Chairman: Operations Superintendent
- c. Vice-Chairman: Technical Services Superintendent
- d. Vice-Chairman: Maintenance Superintendent
- e. Engineering Support Supervisor
- f. Operations Supervisor
- g. Maintenance Supervisor
- h. Reactor and Computer Supervisor
- i. Chemistry and Health Physics Supervisor
- j. Instrument and Control Supervisor
- k. Health Physicist

2. Qualifications

The qualifications of the regular members of the Plant Operations Review Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level at least equal to or higher than as described in Specification 6.1.

3. Meeting Frequency: Monthly, and as required, on call of the Chairman.
4. Quorum: Chairman or Vice-Chairman plus four members or their designated alternates.

NOTE: For purposes of satisfying a quorum, a Vice-Chairman may be considered a member providing that Vice-Chairman is not presiding over the meeting.

- C.. Procedures prepared for A and B above shall be reviewed and approved by the Plant Manager, or his designee, and the Manager of Operations.
- D. Temporary changes to procedures described in Specification 6.5.A above which do not change the intent of the original procedure, may be made with the concurrence of two individuals holding senior operator licenses. Such changes shall be documented and subsequently reviewed by the PORC and approved by the Plant Manager or his designee.
- E. Temporary changes to procedures described in Specification 6.5.B may be made with the concurrence of an individual holding a senior operator license and the health physicist on duty.
- F. Practice of site evacuation exercises shall be conducted annually, following emergency procedures and including a check of communications with off-site support groups. Annual reviews of the Emergency Plan shall be performed.
- G. Licensed radioactive sealed sources shall be leak tested for contamination. Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State as follows:
1. Each licensed sealed source except startup sources previously subjected to core flux, containing radioactive materials, other than Hydrogen 3, with half-life greater than thirty days and in any form, other than gas, shall be tested for leakage and/or contamination at intervals not to exceed six months.
 2. The periodic leak test required does not apply to sealed sources that are stored and are not being used. The sources exempted from this test shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer. In the absence of a certificate from a transferer indicating that a leak test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.
 3. Each sealed startup source shall be tested within 31 days prior to being subjected to core flux and following repair or maintenance to the source.

The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.

Notwithstanding the periodic leak tests required by this Technical Specification, any licensed sealed source is exempt from such leak test when the source contains 100 microcuries or less of beta and/or gamma emitting material or 5 microcuries or less of alpha emitting material.

A special Report shall be prepared and submitted to the Commission within 90 days if source leakage tests reveal the presence of ≥ 0.005 microcuries of removable contamination.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

1.0 Introduction

By letters dated July 22, 1982, September 27, 1982, and December 7, 1982, Vermont Yankee Nuclear Power Corporation (the licensee) proposed changes to the technical specifications related to (1) the action required in the event of low boron concentration for the standby liquid control system (SLCS), and (2) the plant staff organization. We have responded to these requests with a single license amendment.

2.0 Evaluation

2.1 Organizational Chart Update

By letter dated September 27, 1982 the licensee proposed revision of the technical specifications describing the Vermont Yankee organization. Existing requirements are that the Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, Personnel Selection and Training. The proposed technical specifications would permit the Chemistry and Health Physics Supervisor or the Plant Health Physicist to meet this requirement. Because the Plant Health Physicist reports to the Chemistry and Health Physics Supervisor, the proposed technical specification would preserve the lines of reporting responsibility, previously approved, and is acceptable.

In addition the September 27, 1982 proposed revision of the technical specifications describes changes in job titles and groupings of non operational work functions which are purely administrative. These changes are acceptable.

The licensee proposed a further revision to the organizational technical specifications by letter dated December 7, 1982. The position title of the Assistant Plant Manager was changed to Maintenance Superintendent. Also, it was proposed that any of the three superintendents reporting directly to the Plant Manager could have on-site responsibility for safe and efficient operation of the facility during the absence of the Plant Manager, providing the responsibility was delegated in writing. Because each of the three superintendents must meet the requirements of ANSI 1811-1971 with respect to qualifications, this change is considered acceptable.

2.2 Action Required For Low Boron Concentration

The current technical specifications do not clearly specify the actions to be taken when the standby liquid control system (SLCS) concentration/volume limits are not met. During a recent event (LER 82-03), which involved a boron concentration outside of technical specification limits, confusion existed as to the action required. The NRC Resident Inspector requested the licensee to propose technical specifications to correct the ambiguity in the existing specifications (Inspection No. 50-271/82-03). The licensee proposed such technical specifications by letter dated July 22, 1982.

We have reviewed the licensee's proposed technical specifications, as subsequently revised and agreed to by the licensee. The action required for boron concentrations outside of limits results in cold shutdown within 24 hours, which is consistent with action required in the existing technical specification for SLCS inoperability. Because actions to correct concentration or volume require some time to accomplish, a period of 12 hours will be allowed before the licensee is required to initiate shutdown.

We find that the proposed technical specifications, after modification as discussed above, clarify the requirements and provide sufficient assurance of SLCS boron concentration and volume, and are therefore, acceptable.

2.3 Correction of Errors

Two minor typographical errors from previous amendments have also been corrected in this amendment.

3.0 Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant

reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 17, 1983

Principal Contributor: V. L. Rooney

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-271VERMONT YANKEE NUCLEAR POWER CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO FACILITYOPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 75 to Facility Operating License No. DPR-28, issued to Vermont Yankee Nuclear Power Corporation which revised Technical Specifications for operation of the Vermont Yankee Nuclear Power Station (the facility) located near Vernon, Vermont. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications with respect to limiting conditions of operation for the standby liquid control system and administrative controls related to organizational structure.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the applications for amendment dated July 22, September 27, and December 7, 1982 (2) Amendment No. 75 to License No. DPR-28, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W. Washington, D. C., and at the Brooks Memorial Library, 224 Main Street, Brattleboro, Vermont 05301. A copy of item (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of February 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing