

Docket No. 50-271

MAR 2 1981

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Mr. Robert L. Smith  
Licensing Engineer  
Vermont Yankee Nuclear Power Corporation  
1671 Worcester Road  
Framingham, Massachusetts 01701

Dear Mr. Smith:

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The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. The amendment consists of changes to the Technical Specifications in response to your application dated September 12, 1980, as supplemented January 5, 1981.

These changes to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisor Augmentation, (5) Integrity of Systems Outside Containment, and (6) Iodine Monitoring.

Copies of the Safety Evaluation and a related Notice of Issuance are also enclosed.

Sincerely,

Original Signed by

*I. A. Ippolito*

Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 63 to DPR-28
2. Safety Evaluation
3. Notice

cc w/enclosures:

See next page

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P

*No legal objection to Amendment + FR Notices only!*

OFFICE	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB#2	DL:ORB#2		
SURNAME	SNorris	VRooney:ms	TAIppolito	TNoyak	JBachmann		
DATE	2/19/81	2/19/81	2/19/81	2/20/81	2/24/81		



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
March 2, 1981

Docket No. 50-271

Mr. Robert L. Smith  
Licensing Engineer  
Vermont Yankee Nuclear Power  
Corporation  
1671 Worcester Road  
Framingham, Massachusetts 01701

Dear Mr. Smith:

The Commission has issued the enclosed Amendment No. 63 to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. The amendment consists of changes to the Technical Specifications in response to your application dated September 12, 1980, as supplemented January 5, 1981.

These changes to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisor Augmentation, (5) Integrity of Systems Outside Containment, and (6) Iodine Monitoring.

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Sincerely,

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No.63 to DPR-28
2. Safety Evaluation
3. Notice

cc w/enclosures:  
See next page

Mr. Robert L. Smith

cc:

Mr. W. F. Conway  
Vice President and Manager of  
Operations  
Vermont Yankee Nuclear Power  
Corporation  
P. O. Box 157  
Vernon, Vermont 05602

Mr. Louis Heider, V.P.  
Vermont Yankee Nuclear Power  
Corporation  
25 Research Drive  
Westboro, Massachusetts 05181

John A. Ritscher, Esquire  
Rope & Gray  
225 Franklin Street  
Boston, Massachusetts 02110

Laurie Burt  
Assistant Attorney General  
Environmental Protection Division  
Attorney General's Office  
One Ashburton Place, 19th Floor  
Boston, Massachusetts 02108

New England Coalition on Nuclear  
Pollution  
Hill and Dale Farm  
West Hill - Faraway Road  
Putney, Vermont 05346

Mr. Raymond H. Puffer  
Chairman  
Board of Selectman  
Vernon, Vermont 05354

W. P. Murphy, Plant Superintendent  
Vermont Yankee Nuclear Power  
Corporation  
P. O. Box 157  
Vernon, Vermont 05354

Brooks Memorial Library  
224 Main Street  
Brattleboro, Vermont 05301

John R. Stanton, Director  
Radiation Control Agency  
Haven Drive  
Concord, New Hampshire 03301

John W. Stevens  
Conservation Society of Southern  
Vermont  
P. O. Box 256  
Townshend, Vermont 05353

Raymond N. McCandless  
Radiation Control Program  
Occupational & Radiological Hlth  
Administration Building  
Montpelier, Vermont 05602

Ronald J. Wilson  
810 18th Street, N. W.  
Suite 802  
Washington, D. C. 20006

Honorable M. Jerome Diamond  
Attorney General  
John A. Calhoun  
Assistant Attorney General  
State of Vermont  
109 State Street  
Pavilion Office Building  
Montpelier, Vermont 05602

Mr. J. E. Griffin, President  
Vermont Yankee Nuclear Power  
Corporation  
77 Grove Street  
Rutland, Vermont 05701

U. S. Environmental Protection  
Agency  
Region I Office  
ATTN: EIS COORDINATOR  
JFK Federal Building  
Boston, Massachusetts 02203

Mr. Robert L. Smith

cc:

David White  
Co-Director  
Vermont Public Interest  
Research Group, Inc.  
43 State Street  
Montpelier, Vermont 05602

Public Service Board  
State of Vermont  
120 State Street  
Montpelier, Vermont 05602

Director, Criteria and Standards  
Division  
Office of Radiation Programs (ANR-460)  
U. S. Environmental Protection Agency  
Washington, D. C. 20460

Vermont Yankee Decommissioning  
Alliance  
127 Main Street  
Brattleboro, Vermont 05301

Vermont Yankee Decommissioning  
Alliance  
5 State Street  
Box 1117  
Montpelier, Vermont 05602

Resident Inspector  
c/o U. S. NRC  
P. O. Box 176  
Vernon, Vermont 05453



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63  
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application by Vermont Yankee Nuclear Power Corporation (the licensee) dated September 12, 1980, as supplemented January 5, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility License No. DPR-28 is hereby amended to read as follows:

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B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 2, 1981

ATTACHMENT TO LICENSE AMENDMENT NO.63

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Revise Appendix A as follows:

Remove

49  
-  
60  
61  
190a  
193  
215

Insert

49  
49a  
60  
61  
190a  
193  
215

TABLE 3.2.6

POST-ACCIDENT INSTRUMENTATION

Minimum Number of Operable Instrument Channels	Parameter	Type of Indication	Instrument Range
2	Drywell Atmospheric Temperature (Note 1)	Recorder #16-19-45 Recorder #TR-1-149	0-300° F 0-300° F
2	Drywell Pressure (Note 1) Torus Pressure (Note 1)	Recorder #16-19-44	0-80 psia 0-80 psia
2	Torus Water Level (Note 3)	Meter #16-19-46A Meter #16-19-46B	0-3 ft. 0-3 ft.
2	Torus Water Temperature	Meter #16-19-48	60-180° F
2	Reactor Pressure (Note 1)	Recorder #6-97 Meter #6-90A Meter #6-90B	0-1200 psig 0-1200 psig 0-1200 psig
2	Reactor Vessel Water Level (Note 1)	Meter #2-3-91A Meter #2-3-91B	(-150)-0-(+150)"H <sub>2</sub> O (-150)-0-(+150)"H <sub>2</sub> O
1	Control Rod Position (Note 1, 2)	Meter	0-48" RPIS
1	Neutron Monitor (Note 1, 2)	Meter	0-125% Rated Flux
1	Torus Air Temperature (Note 1)	Recorder #TR-16-19-45	0-300° F
2/valve	Safety/Relief Valve Position from pressure switches (Note 4)	Lights (SRV 2-71-A through D)	Closed - Open
1/valve	Safety Valve Position from Acoustic Monitor (Note 5)	Meter Z1-2-1A/B	Closed - Open

Note 1 - From and after the date that one of these parameters is not indicated in the control room, continued reactor operation is permissible during the next seven days. If reduced to one indication of a parameter operation is permissible for 30 days.

Note 2 - Control rod position and neutron monitor instruments are considered to be redundant to each other.

TABLE 3.2.6 NOTES

- Note 3 - From and after the date that this parameter is reduced to one indication in the control room, continued reactor operation is permissible during the next thirty days. If both channels are inoperable and indication cannot be restored in six hours, an orderly shutdown shall be initiated and the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.
- Note 4 - From and after the date that safety/relief valve position from pressure switches is unavailable, reactor operation may continue provided safety/relief valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and meter 16-19-48 (torus water temperature, 60-180°F). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.
- Note 5 - From and after the date that safety valve position from the acoustic monitor is unavailable, reactor operation may continue provided safety valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and recorder 16-19-44 (drywell pressure 0-80 psia). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and in a cold shutdown condition in the following 18 hours.

TABLE 4.2.6

CALIBRATION FREQUENCIES  
POST-ACCIDENT INSTRUMENTATION

<u>Parameter</u>	<u>Calibration</u>	<u>Instrument Check</u>
Drywell Atmosphere Temperature	every 6 months	once each day
Drywell and Torus Pressure	every 6 months	once each day
Torus Water Level	every 6 months	once each shift
Torus Water Temperature	every 6 months	once each day
Reactor Pressure	every 6 months	once each day
Reactor Vessel Water Level	every 6 months	once each day
Control Rod Position	(note 5)	once each day
Neutron Monitor	Same as reactor protection systems	once each day
Torus Air Temperature	every 6 months	once each day
Safety/Relief Valve Position	every refueling outage (Note 9) (a Functional Test to be performed quarterly)	once each day
Safety Valve Position	every refueling outage (Note 9) (a Functional Test to be performed quarterly)	once each day

TABLE 4.2 NOTES

1. Initially once per month; thereafter, a longer interval as determined by test results on this type of instrumentation.
2. During each refueling outage, simulated automatic actuation which opens all pilot valves shall be performed such that each trip system logic can be verified independent of its redundant counterpart.
3. Trip system logic calibration shall include only time delay relays and timers necessary for proper functioning of the trip system.
4. This instrumentation is excepted from functional test definition. The functional test will consist of injecting a simulated electrical signal into the measurement channel.
5. Check control rod position indication while performing the surveillance requirement of section 3.3.
6. Functional tests, calibrations and instrument checks are not required when these instruments are not to be operable or tripped. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibration shall be performed prior to or during each startup or controlled shutdowns with a required frequency not to exceed once per week. Instrument checks shall be performed at least once per day during those periods when instruments are required to be operable.
7. This instrumentation is excepted from the functional test definitions and shall be calibrated using simulated electrical signals once every three months.
8. Functional tests and calibrations are not required when systems are not required to be operable.
9. The thermocouples associated with Safety/Relief Valves and Safety Valve Position, that may be used for backup position indication, shall be verified to be operable every operating cycle.

5. The Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).
  6. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.
- E. A Fire Brigade of at least 5 members shall be maintained onsite at all times. # This excludes 2 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.

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#Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

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TABLE 6.1.1

Vermont Yankee staff positions that shall be filled by personnel holding Senior Operator and Operator licenses are indicated in the following table:

<u>Title</u>	<u>License</u>
Operations Supervisor	Senior Operator
Shift Supervisor	Senior Operator
Supervisory Control Room Operator	Operator
Control Room Operator	Operator

<u>MINIMUM SHIFT CREW PERSONNEL &amp; LICENSE REQUIREMENTS</u>	<u>Conditions</u>		
	<u>Normal Operation</u>	<u>Plant Startup</u>	<u>Cold Shutdown</u>
Shift Supervisor	(1)	(1)	-
Supervisory Control Room Operator	(1)	(1)	(1)
Control Room Operator	(1)	(1)	(1)
Auxiliary Operator	(2)	(2)	(1)
Shift Technical Advisor	(1)	(1)	-
Senior Operators License	(1)	(1)	(1)
Operators License	(2)	(2)	(1)

## 6.9 ENVIRONMENTAL QUALIFICATION

- A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-28 dated October 24, 1980.
- B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise qualified.

## 6.10 INTEGRITY OF SYSTEMS OUTSIDE CONTAINMENT

A program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels will be implemented. This program shall include the following:

- 1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and
- 2) System leakage inspections, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, (5) RCIC, and (6) sample systems.

## 6.11 IODINE MONITORING

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas\* under accident conditions will be implemented. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

\* Areas requiring personnel access for establishing hot shutdown condition.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-28

VERMONT YANKEE NUCLEAR POWER CORPORATION

VERMONT YANKEE NUCLEAR POWER STATION

DOCKET NO. 50-271

I. INTRODUCTION

By letter dated September 12, 1980, as supplemented January 5, 1981, the Vermont Yankee Nuclear Power Corporation (the licensee) proposed changes to the Technical Specifications (TSs) appended to Facility Operating License No. DPR-28 for the Vermont Yankee Nuclear Power Station. The changes involve the incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. The licensee's request is in direct response to the NRC staff's letter dated July 2, 1980.

II. BACKGROUND INFORMATION

By our letter dated September 13, 1979, we issued to all operating nuclear power plants requirements established as a result of our review of the Three Mile Island Unit 2 accident. Certain of these requirements, designated Lessons Learned Category "A" requirements, were to have been completed by the licensee prior to any operation subsequent to January 1, 1980. Our evaluation of the licensee's compliance with these Category "A" items was attached to our letter to the licensee dated April 11, 1980.

In order to provide reasonable assurance that operating reactor facilities are maintained within the limits determined acceptable following the implementation of the TMI-2 Lessons Learned Category "A" items, we requested that licensees amend their TSs to incorporate additional Limiting Conditions of Operation and Surveillance Requirements, as appropriate. This request was transmitted to all licensees on July 2, 1980. Included therein were model specifications that we had determined to be acceptable. The licensee's application is in direct response to our request. Each of the issues identified by the NRC staff and the licensee's response is discussed in the Evaluation below.

III. EVALUATION

1. Emergency Power Supply/Inadequate Core Cooling

As applicable to Boiling Water Reactors (BWRs), we indicated that water level instrumentation is important to post-accident monitoring and that surveillance of this instrumentation should be performed. The licensee's response to this request stated that the current surveillance requirements for the reactor water level instrumentation at Vermont Yankee meet or exceed our guidance.

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We have reviewed the current specifications (Tables 3.2-6 and 4.2-6) and determined that water level instrumentation is included. Specification 3.2G requires that this equipment be operable during reactor power operation. Surveillance requirements for instrument checks and calibration are also included. The frequency of surveillance meets or exceeds our guidelines. Based on this review, we conclude that no changes are required to satisfy our request.

2. Valve Position Indication

Our requirements for installation of a reliable position indicating system for relief and safety valves was based on the need to provide the operator with a diagnostic aid to reduce the ambiguity between indications that might indicate either an open relief/safety valve or a small line break. Such a system did not need to be safety grade provided that backup methods of determining valve position are available.

The licensee's request would add both the primary indicating system (tail-pipe pressure switches on safety/relief valves and acoustic monitors on safety valves) and the secondary indicating system (downstream temperature detectors) to the specifications. Actions have been specified for the condition of an inoperable channel and for inoperability of both primary and backup detector channels. Additionally, surveillance requirements have been included. Based on our review, we find the licensee's recommended changes satisfy our guidelines and are acceptable.

3. Containment Isolation

Our request indicated that the specifications should include a Table of Containment Isolation Valves which reflect the diverse isolation signal requirement of this Lessons Learned issue.

The licensee's request stated that the current specifications include a requirement for diverse isolation signals and that no changes are required.

We have reviewed the current specifications (Tables 3.2.2, 4.2.2, 4.7.2a, and 4.7.2b). These tables include a listing of valves, actuation signals and surveillance requirements. Based on this review, we have determined that the current specifications satisfy our request and that no changes are necessary.

4. Shift Technical Advisor (STA)

Our request indicated that the TSs related to minimum shift manning should be revised to reflect the augmentation of an STA. The STA function includes both accident and operating experience assessment.

The licensee proposed the addition of an STA to the minimum shift crew composition and the specific qualifications of this individual.

These qualifications state that the STA shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

Our evaluation of the adequacy of the licensee's actions to provide STAs, including the interim period from January 1, 1980 to January 1, 1981, was contained in our letter dated February 26, 1980. That evaluation concluded that it was acceptable for the interim period to use, as STAs, Senior Reactor Operators with added training.

In view of the above, we have determined that the licensee's request satisfies our request and is acceptable.

5. Integrity of Systems Outside Containment

Our letter dated July 2, 1980, indicated that the license should be amended by adding a license condition related to a Systems Integrity Measurements Program. Such a condition would require the licensee to effect an appropriate program to eliminate or prevent the release of significant amounts of radioactivity to the environment via leakage from engineered safety systems and auxiliary systems, which are located outside reactor containment.

The licensee proposed such provisions by letter dated January 5, 1981. We have reviewed this proposal and determined that our request has been satisfied.

6. Iodine Monitoring

Our letter dated July 2, 1980, indicated that the license should be amended by adding a license condition related to iodine monitoring. Such a condition would require the licensee to effect a program which would ensure the capability to determine the airborne iodine concentration in areas requiring personnel access under accident conditions.

The licensee proposed such provisions by letter dated January 5, 1981. We have reviewed this proposal and determined that our request has been satisfied.

IV. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination,

we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

V. CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 2, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-271VERMONT YANKEE NUCLEAR POWER CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 63 to Facility Operating License No. DPR-28, issued to Vermont Yankee Nuclear Power Corporation (the licensee), which revised the Technical Specifications for operation of the Vermont Yankee Nuclear Power Station (the facility), located in Windham County, Vermont. The amendment is effective as of the date of issuance.

These changes to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Shift Technical Advisor Augmentation, (5) Integrity of Systems Outside Containment and (6) Iodine Monitoring.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

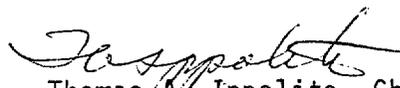
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The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated September 12, 1980, as supplemented January 5, 1981, (2) Amendment No.63 to License No. DPR-28 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Brooks Memorial Library, 224 Main Street, Brattleboro, Vermont 05301. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 2nd day of March 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing