

February 14, 1996

Mr. George A. Hunger, Jr.
Director-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 (TAC NOS. M93212 AND M93213)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendment No. 112 to Facility Operating License No. NPF-39 and Amendment No. 74 to Facility Operating License No. NPF-85 for the Limerick Generating Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated July 28, 1995.

These amendments delete the operability and surveillance requirements involving secondary containment differential pressure instrumentation.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

- Enclosures: 1. Amendment No. 112 to License No. NPF-39
2. Amendment No. 74 to License No. NPF-85
3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink, appearing to read "Frank Rinaldi", is written above the typed name.

Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

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License No. NPF-39
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3. Safety Evaluation

cc w/encs: See next page

Mr. George A. Hunger, Jr.
PECO Energy Company

Limerick Generating Station,
Units 1 & 2

cc:

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Board of Supervisors
of Limerick Township
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Linfield, PA 19468



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.112
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated July 28, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

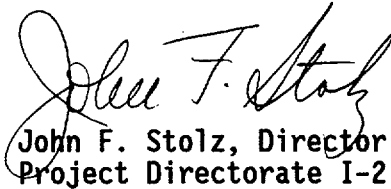
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 112, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: February 14, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 112

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Insert

3/4 3-14

3/4 3-14

3/4 3-15

3/4 3-15

3/4 3-17

3/4 3-17

3/4 3-21

3/4 3-21

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TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ^(a)</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM ^(b)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level				
1) Low, Low - Level 2	B	2	1, 2, 3	20
2) Low, Low, Low - Level 1	C	2	1, 2, 3	20
b. Drywell Pressure - High	H	2	1, 2, 3	20
c. North Stack Effluent				
Radiation - High ^(a)	W	1	1, 2, 3	23
d. Deleted				
e. Reactor Enclosure Ventilation Exhaust Duct-Radiation - High	S	2	1, 2, 3	23
f. Deleted				
g. Deleted				
h. Drywell Pressure - High/ Reactor Pressure - Low	G	2/2	1, 2, 3	26
i. Primary Containment Instrument Gas Line to Drywell Δ Pressure - Low	M	1	1, 2, 3	26
j. Manual Initiation	NA	1	1, 2, 3	24

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ^(a), ^(c)</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM ^(b)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level Low, Low - Level 2	B	2	1, 2, 3	25
b. Drywell Pressure - High	H	2	1, 2, 3	25
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	R	2	*#	25
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	R	2	*#	25
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	S	2	1, 2, 3	25
e. Deleted				
f. Deleted				
g. Reactor Enclosure Manual Initiation	NA	1	1, 2, 3	24
h. Refueling Area Manual Initiation	NA	1	*	25

TABLE 3.3.2-1 (Continued)

TABLE NOTATIONS

- (c) Actuates secondary containment isolation valves shown in Table 3.6.5.2.1-1 and/or 3.6.5.2.2-1 and signals B, H, S, and R also start the standby gas treatment system.
- (d) RWCU system inlet outboard isolation valve closes on SLCS "B" initiation. RWCU system inlet inboard isolation valve closes on SLCS "A" or SLCS "C" initiation.
- (e) Manual initiation isolates the steam supply line outboard isolation valve and only following manual or automatic initiation of the system.
- (f) In the event of a loss of ventilation the temperature - high setpoint may be raised by 50°F for a period not to exceed 30 minutes to permit restoration of the ventilation flow without a spurious trip. During the 30 minute period, an operator, or other qualified member of the technical staff, shall observe the temperature indications continuously, so that, in the event of rapid increases in temperature, the main steam lines shall be manually isolated.
- (g) Wide range accident monitor per Specification 3.3.7.5.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level		
1. Low, Low - Level 2	≥ -38 inches*	≥ -45 inches
2. Low, Low, Low - Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. North Stack Effluent Radiation - High	≤ 2.1 $\mu\text{Ci/cc}$	≤ 4.0 $\mu\text{Ci/cc}$
d. Deleted		
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	≤ 1.35 mR/h	≤ 1.5 mR/h
f. Deleted		
g. Deleted		
h. Drywell Pressure - High/ Reactor Pressure - Low	≤ 1.68 psig/ ≥ 455 psig (decreasing)	≤ 1.88 psig/ ≥ 435 psig (decreasing)
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	≥ 2.0 psi	≥ 1.9 psi
j. Manual Initiation	N.A.	N.A.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Low - Level 2	≥ -38 inches*	≥ -45 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	≤ 2.0 mR/h	≤ 2.2 mR/h
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	≤ 2.0 mR/h	≤ 2.2 mR/h
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	≤ 1.35 mR/h	≤ 1.5 mR/h
e. Deleted		
f. Deleted		
g. Reactor Enclosure Manual Initiation	N.A.	N.A.
h. Refueling Area Manual Initiation	N.A.	N.A.

* See Bases Figure B 3/4 3-1.

** The low setpoints are for the RWCU Heat Exchanger Rooms; the high setpoints are for the pump rooms.

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level	
1) Low, Low - Level 2	≤ 13 ^(a)
2) Low, Low, Low - Level 1	≤ 13 ^(a)
b. Drywell Pressure - High	≤ 13 ^(a)
c. North Stack Effluent Radiation - High	N.A.
d. Deleted	
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	N.A.
f. Deleted	
g. Deleted	
h. Drywell Pressure - High/ Reactor Pressure - Low	N.A.
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	N.A.
j. Manual Initiation	N.A.
7. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level Low, Low - Level 2	N.A.
b. Drywell Pressure - High	N.A.
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	N.A.
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	N.A.
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	N.A.
e. Deleted	

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
f. Deleted	
g. Reactor Enclosure Manual Initiation	N.A.
h. Refueling Area Manual Initiation	N.A.

TABLE NOTATIONS

- (a) Isolation system instrumentation response time specified includes 10 seconds diesel generator starting and 3 seconds for sequence loading delays.
- (b) DELETED
- * Isolation system instrumentation response time for MSIV only. No diesel generator delays assumed for MSIVs.
- ** Isolation system instrumentation response time for associated valves except MSIVs.
- # Isolation system instrumentation response time specified for the Trip Function actuating each valve group shall be added to isolation time shown in Tables 3.6.3-1, 3.6.5.2.1-1 and 3.6.5.2.2-1 for valves in each valve group to obtain ISOLATION SYSTEM RESPONSE TIME for each valve.
- ## With 45 second time delay.

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRE</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level				
1) Low, Low - Level 2	S	Q	R	1, 2, 3
2) Low, Low, Low - Level 1	S	Q	R	1, 2, 3
b. Drywell Pressure## - High	S	Q	R	1, 2, 3
c. North Stack Effluent Radiation - High	S	Q	R	1, 2, 3
d. Deleted				
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	S	Q	R	1, 2, 3
f. Deleted				
g. Deleted				
h. Drywell Pressure - High/ Reactor Pressure - Low	S	Q	R	1, 2, 3
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	N.A.	M	Q	1, 2, 3
j. Manual Initiation	N.A.	R	N.A.	1, 2, 3

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level Low, Low - Level 2	S	Q	R	1, 2, 3
b. Drywell Pressure## - High	S	Q	R	1, 2, 3
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	S	Q	R	*#
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	S	Q	R	*#
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	S	Q	R	1, 2, 3
e. Deleted				
f. Deleted				
g. Reactor Enclosure Manual Initiation	N.A.	R	N.A.	1, 2, 3
h. Refueling Area Manual Initiation	N.A.	R	N.A.	*

*Required when (1) handling irradiated fuel in the refueling area secondary containment, or (2) during CORE ALTERATIONS, or (3) during operations with a potential for draining the reactor vessel with the vessel head removed and fuel in the vessel.

**When not administratively bypassed and/or when any turbine stop valve is open.

#During operation of the associated Unit 1 or Unit 2 ventilation exhaust system.

##These trip functions (2a, 6b, and 7b) are common to the RPS actuation trip function.

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

LIMERICK - UNIT 1	PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
3/4 6-22 Amendment No. 8, 13, 15, 89, 112	025	DRYWELL PURGE SUPPLY	HV57-121(X-201A) HV57-123		5**	B,H,S,W,R	3,11,14	57
					5**	B,H,S,W,R	3,11,14	
				HV57-109 (X-201A)	6**	B,H,S,W,R	11	
				HV57-131 (X-201A)	5**	B,H,S,W,R	11	
				HV57-135	6**	B,H,S,W,R	11	
	026	DRYWELL PURGE EXHAUST	HV57-114 HV57-111 SV57-139		9	B,H,R,S	3,11,14	57
					90	B,H,R,S	11	
				FV-C-DO-101B	90	B,H,R,S	11	
	027A	CONTAINMENT INSTRUMENT GAS SUPPLY TO ADS VALVES H,M,&S	59-1128(CK)		NA			59
					45	M		
				HV59-151A				
	028A-1	RECIRC LOOP SAMPLE	HV43-1F019		10	B		43
				HV43-1F020	10	B		
	028A-2	DRYWELL H2/O2 SAMPLE	SV57-132		5	B,H,R,S	11	57
				SV57-142	5	B,H,R,S	11	
	028A-3	DRYWELL H2/O2 SAMPLE	SV57-134		5	B,H,R,S	11	57
				SV57-144	5	B,H,R,S	11	

TABLE 3.6.3-1 (Continued)
PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
			SV57-191 (X-220A)	5	B,H,R,S	11	
116	STANDBY LIQUID CONTROL	48-1F007(CK) (X-42)	HV48-1F006B	NA 60		29	48
117B-1	DRYWELL RADIATION MONITORING SUPPLY	SV26-190A	SV26-190B	5 5	B,H,R,S B,H,R,S	11 11	26
117B-2	DRYWELL RADIATION MONITORING RETURN	SV26-190C	SV26-190D	5 5	B,H,R,S B,H,R,S	11 11	26
201A	SUPPRESSION POOL PURGE SUPPLY	HV57-124 HV57-131(X-25)	HV57-109(X-25) HV57-147 HV57-121(X-25)	5** 5** 6** 6** 5**	B,H,S,W,R B,H,S,W,R B,H,S,W,R B,H,S,W,R B,H,S,W,R	3,11,14 3,11,14 11 11 11	57
	HYDROGEN RECOMBINER "B" EXHAUST	HV57-164	HV57-169	9 9	B,H,R,S B,H,R,S	3,11,14 11	
202	SUPPRESSION POOL PURGE EXHAUST	HV57-104 HV57-105	HV57-112 HV57-118 SV57-185	5** 15** 6** 5** 5	B,H,S,W,R B,H,S,R B,H,S,W,R B,H,S,R B,H,R,S	3,11,14,33 11 11,33 11 11	57
	HYDROGEN RECOMBINER "A" EXHAUST	HV57-162	HV57-166	9 9	B,H,R,S B,H,R,S	3,11,14 11	
203A(B,C,D)	RHR PUMP SUCTION		HV51-1F004A(B, C,D)	240		29,35	51
			PSV51-1F030A (B,C,D)	NA		35	

LIMERICK - UNIT 1

3/4 6-26

Amendment No. 8, 13, 15, 33, 110, 112



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated July 28, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

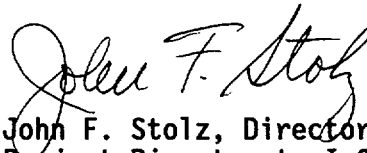
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 74 , are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: February 14, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
3/4 3-14	3/4 3-14
3/4 3-15	3/4 3-15
3/4 3-17	3/4 3-17
3/4 3-21	3/4 3-21
3/4 3-22	3/4 3-22
3/4 3-25	3/4 3-25
3/4 3-26	3/4 3-26
3/4 3-30	3/4 3-30
3/4 3-31	3/4 3-31
3/4 6-22	3/4 6-22
3/4 6-26	3/4 6-26

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ^(a)</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM ^(b)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level				
1) Low, Low - Level 2	B	2	1, 2, 3	20
2) Low, Low, Low - Level 1	C	2	1, 2, 3	20
b. Drywell Pressure - High	H	2	1, 2, 3	20
c. North Stack Effluent Radiation - High ^(a)	W	1	1, 2, 3	23
d. Deleted				
e. Reactor Enclosure Ventilation Exhaust Duct-Radiation - High	S	2	1, 2, 3	23
f. Deleted				
g. Deleted				
h. Drywell Pressure - High/ Reactor Pressure - Low	G	2/2	1, 2, 3	26
i. Primary Containment Instrument Gas Line to Drywell Δ Pressure - Low	M	1	1, 2, 3	26
j. Manual Initiation	NA	1	1, 2, 3	24

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ^(a).^(c)</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM ^(b)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level Low, Low - Level 2	B	2	1, 2, 3	25
b. Drywell Pressure - High	H	2	1, 2, 3	25
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	R	2	*#	25
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	R	2	*#	25
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	S	2	1, 2, 3	25
e. Deleted				
f. Deleted				
g. Reactor Enclosure Manual Initiation	NA	1	1, 2, 3	24
h. Refueling Area Manual Initiation	NA	1	*	25

TABLE 3.3.2-1 (Continued)

TABLE NOTATIONS

- (c) Actuates secondary containment isolation valves shown in Table 3.6.5.2.1-1 and/or 3.6.5.2.2-1 and signal B, H, S, and R also start the standby gas treatment system.
- (d) RWCU system inlet outboard isolation valve closes on SLCS "B" initiation. RWCU system inlet inboard isolation valve closes on SLCS "A" or SLCS "C" initiation.
- (e) Manual initiation isolates the steam supply line outboard isolation valve and only following manual or automatic initiation of the system.
- (f) In the event of a loss of ventilation the temperature - high setpoint may be raised by 50°F for a period not to exceed 30 minutes to permit restoration of the ventilation flow without a spurious trip. During the 30 minute period, an operator, or other qualified member of the technical staff, shall observe the temperature indications continuously, so that, in the event of rapid increases in temperature, the main steam lines shall be manually isolated.
- (g) Wide range accident monitor per Specification 3.3.7.5.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level		
1. Low, Low - Level 2	≥ -38 inches*	≥ -45 inches
2. Low, Low, Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. North Stack Effluent Radiation - High	≤ 2.1 $\mu\text{Ci/cc}$	≤ 4.0 $\mu\text{Ci/cc}$
d. Deleted		
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	≤ 1.35 mR/h	≤ 1.5 mR/h
f. Deleted		
g. Deleted		
h. Drywell Pressure - High/ Reactor Pressure - Low	≤ 1.68 psig/ ≥ 455 psig (decreasing)	≤ 1.88 psig/ ≥ 435 psig (decreasing)
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	≥ 2.0 psi	≥ 1.9 psi
j. Manual Initiation	N.A.	N.A.

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Low - Level 2	≥ -38 inches*	≥ -45 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	≤ 2.0 mR/h	≤ 2.2 mR/h
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	≤ 2.0 mR/h	≤ 2.2 mR/h
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	≤ 1.35 mR/h	≤ 1.5 mR/h
e. Deleted		
f. Deleted		
g. Reactor Enclosure Manual Initiation	N.A.	N.A.
h. Refueling Area Manual Initiation	N.A.	N.A.

* See Bases Figure B 3/4 3-1.

** The low setpoints are for the RWCU Heat Exchanger Rooms; the high setpoints are for the pump rooms.

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level	
1) Low, Low - Level 2	≤ 13 ^(a)
2) Low, Low, Low - Level 1	≤ 13 ^(a)
b. Drywell Pressure - High	≤ 13 ^(a)
c. North Stack Effluent Radiation - High	N.A.
d. Deleted	
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	N.A.
f. Deleted	
g. Deleted	
h. Drywell Pressure - High/ Reactor Pressure - Low	N.A.
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	N.A.
j. Manual Initiation	N.A.
7. <u>SECONDARY CONTAINMENT ISOLATION</u>	
a. Reactor Vessel Water Level Low, Low - Level 2	N.A.
b. Drywell Pressure - High	N.A.
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	N.A.
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	N.A.
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	N.A.
e. Deleted	

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
f. Deleted	
g. Reactor Enclosure Manual Initiation	N.A.
h. Refueling Area Manual Initiation	N.A.

TABLE NOTATIONS

- (a) Isolation system instrumentation response time specified includes 10 seconds diesel generator starting and 3 seconds for sequence loading delays.
- (b) DELETED
- * Isolation system instrumentation response time for MSIV only. No diesel generator delays assumed for MSIVs.
- ** Isolation system instrumentation response time for associated valves except MSIVs.
- # Isolation system instrumentation response time specified for the Trip Function actuating each valve group shall be added to isolation time shown in Tables 3.6.3-1, 3.6.5.2.1-1 and 3.6.5.2.2-1 for valves in each valve group to obtain ISOLATION SYSTEM RESPONSE TIME for each valve.
- ## With 45 second time delay.

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRE</u>
6. <u>PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level				
1) Low, Low - Level 2	S	Q	R	1, 2, 3
2) Low, Low, Low - Level 1	S	Q	R	1, 2, 3
b. Drywell Pressure ## - High	S	Q	R	1, 2, 3
c. North Stack Effluent Radiation - High	S	Q	R	1, 2, 3
d. Deleted				
e. Reactor Enclosure Ventilation Exhaust Duct - Radiation - High	S	Q	R	1, 2, 3
f. Deleted				
g. Deleted				
h. Drywell Pressure - High/ Reactor Pressure - Low	S	Q	R	1, 2, 3
i. Primary Containment Instrument Gas to Drywell Δ Pressure - Low	N.A.	M	Q	1, 2, 3
j. Manual Initiation	N.A.	R	N.A.	1, 2, 3

TABLE 4.3.2.1-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRE</u>
7. <u>SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level Low, Low - Level 2	S	Q	R	1, 2, 3
b. Drywell Pressure## - High	S	Q	R	1, 2, 3
c.1. Refueling Area Unit 1 Ventilation Exhaust Duct Radiation - High	S	Q	R	*#
2. Refueling Area Unit 2 Ventilation Exhaust Duct Radiation - High	S	Q	R	*#
d. Reactor Enclosure Ventilation Exhaust Duct Radiation - High	S	Q	R	1, 2, 3
e. Deleted				
f. Deleted				
g. Reactor Enclosure Manual Initiation	N.A.	R	N.A.	1, 2, 3
h. Refueling Area Manual Initiation	N.A.	R	N.A.	*

*Required when (1) handling irradiated fuel in the refueling area secondary containment, or (2) during CORE ALTERATIONS, or (3) during operations with a potential for draining the reactor vessel with the vessel head removed and fuel in the vessel.

**When not administratively bypassed and/or when any turbine stop valve is open.

#During operation of the associated Unit 1 or Unit 2 ventilation exhaust system.

##These trip functions (2a, 6b, and 7b) are common to the RPS actuation trip function.

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

LIMERICK - UNIT 2

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Amendment No. 52, 74

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
025	DRYWELL PURGE SUPPLY	HV57-221(X-201A) HV57-223		5**	B,H,S,W,R	3,11,14	57
				5**	B,H,S,W,R	3,11,14	
			HV57-209 (X-201A)	6**	B,H,S,W,R	11	
			HV57-231 (X-201A)	5**	B,H,S,W,R	11	
			HV57-235	6**	B,H,S,W,R	11	
	HYDROGEN RECOMBINER "B" INLET	HV57-263	FV57-D0-201B	9 90	B,H,R,S B,H,R,S	3,11,14 11,34	
026	DRYWELL PURGE EXHAUST	HV57-214 HV57-211 SV57-239		5**	B,H,S,W,R	3,11,14,33	57
				15**	B,H,S,R	11	
				5		10	
			HV57-215	6**	B,H,S,W,R	11,33	
			HV57-217	5**	B,H,S,R	11	
	SV57-245	5	B,H,R,S	11			
HYDROGEN RECOMBINER "A" INLET	HV57-261	FV57-D0-201A	9 90	B,H,R,S B,H,R,S	3,11,14 11,34		
027A	CONTAINMENT INSTRUMENT GAS SUPPLY TO ADS VALVES H,M,&S	59-2128(CK)	HV59-251A	NA 45	M		59
028A-1	RECIRC LOOP SAMPLE	HV43-2F019	HV43-2F020	10 10	B B		43
028A-2	DRYWELL H2/O2 SAMPLE	SV57-232	SV57-242	5 5	B,H,R,S B,H,R,S	11 11	57
028A-3	DRYWELL H2/O2 SAMPLE	SV57-234	SV57-244	5 5	B,H,R,S B,H,R,S	11 11	57

TABLE 3.6.3-1 (Continued)
PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX.ISOL. TIME.IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
			SV57-291 (X-220A)	5	B,H,R,S	11	
116	STANDBY LIQUID CONTROL	48-2F007(CK) (X-42)	HV48-2F006B	NA 60		29	48
117B-1	DRYWELL RADIATION MONITORING SUPPLY	SV26-290A	SV26-290B	5 5	B,H,R,S B,H,R,S	11 11	26
117B-2	DRYWELL RADIATION MONITORING RETURN	SV26-290C	SV26-290D	5 5	B,H,R,S B,H,R,S	11 11	26
201A	SUPPRESSION POOL PURGE SUPPLY	HV57-224 HV57-231(X-25)	HV57-209(X-25) HV57-247 HV57-221(X-25)	5** 5** 6** 6** 5**	B,H,S,W,R B,H,S,W,R B,H,S,W,R B,H,S,W,R B,H,S,W,R	3,11,14 3,11,14 11 11 11	57
	HYDROGEN RECOMBINER "B" EXHAUST	HV57-264	HV57-269	9 9	B,H,R,S B,H,R,S	3,11,14 11	
202	SUPPRESSION POOL PURGE EXHAUST	HV57-204 HV57-205	HV57-212 HV57-218 SV57-285	5** 15** 6** 5** 5	B,H,S,W,R B,H,S,R B,H,S,W,R B,H,S,R B,H,R,S	3,11,14,33 11 11,33 11 11	57
	HYDROGEN RECOMBINER "A" EXHAUST	HV57-262	HV57-266	9 9	B,H,R,S B,H,R,S	3,11,14 11	
203A(B,C,D)	RHR PUMP SUCTION		HV51-2F004A (B,C,D)	240		29,36	51
			PSV51-2F030A (B,C,D)	NA		36	

LIMERICK - UNIT 2

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Amendment No. 73,74



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 112 AND 74 TO FACILITY OPERATING

LICENSE NOS. NPF-39 AND NPF-85

PHILADELPHIA ELECTRIC COMPANY

LIMERICK GENERATING STATION, UNITS 1 AND 2

DOCKET NOS. 50-352 AND 50-353

1.0 INTRODUCTION

By letter dated July 28, 1995, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Limerick Generating Station, Units 1 and 2, Technical Specifications (TSs). The requested changes would delete the operability and surveillance requirements involving secondary containment differential pressure instrumentation. These changes are consistent with the guidelines contained in the Improved Standard Technical Specifications, General Electric Plants BWR/4 (NUREG-1433), issued September 28, 1992, and do not change the equipment configuration or operation.

2.0 EVALUATION

The licensee has requested to delete the following operability and surveillance requirements involving secondary containment differential pressure instrumentation for Units 1 and 2:

Table 3.3.2-1 (6f) (7e) (7f)
Table 3.3.2-2 (6f) (7e) (7f)
Table 3.3.2-3 (6f) (7e) (7f)
Table 4.3.2.1-1 (6f) (7e) (7f)
Table 3.6.3-1 Penetration 025 Isolation Signals U,T
Table 3.6.3-1 Penetration 026 Isolation Signals U,T
Table 3.6.3-1 Penetration 201A Isolation Signals U,T
Table 3.6.3-1 Penetration 202 Isolation Signals U,T
Table 3.6.5.2.1-1 Zone I (Unit 1) Isolation Signals U,T
Table 3.6.5.2.1-1 Zone II (Unit 2) Isolation Signals U,T
Table 3.6.5.2.2-1 Zone III Isolation Signals U,T

In addition, the licensee requested that Table Notation (c) of Table 3.3.2-1 for Units 1 and 2 be revised to delete reference to isolation signals 'U' and 'T' in support of the changes identified above.

The purpose of the containment isolation system is to prevent or limit the release of radioactive materials that may result from postulated accidents. For the primary containment, this is accomplished by providing isolation

barriers in all fluid lines that penetrate the primary containment. For the secondary containment, fission products which may leak into the secondary containment following a Loss of Coolant Accident (LOCA) or fuel handling accident are controlled by maintaining a negative pressure relative to the outside atmospheric pressure and filtering the fission products prior to release. The licensee monitors plant parameters (i.e., reactor vessel water level, primary containment pressure, system flows, room temperatures, safeguard bus voltages, and radiation levels) to identify conditions associated with a design basis accident, and to take appropriate actions to isolate primary containment, and to initiate the Standby Gas Treatment System (SGTS) and Reactor Enclosure Recirculation System (RERS) when pre-determined valves are exceeded.

When the reactor enclosure pressure is less than or equal to a negative 0.1-inch water gage, with respect to outside atmospheric pressure, a limited primary containment isolation signal and a secondary containment isolation signal are initiated by the Outside Atmosphere To Reactor Enclosure Delta Pressure-Low trip function in anticipation of a potential design basis accident. Further, the licensee takes no credit for the operation of these trip functions in any design basis accidents evaluated in the Updated Final Safety Analysis Report (UFSAR).

The licensee's justification for deleting the TS requirements for operability and surveillance of the above trip functions is based on the following reasons: (1) no credit is taken for the operation of the Outside Atmosphere To Reactor Enclosure Delta Pressure-Low and the Outside Atmosphere To Refueling Area Delta Pressure-Low trip functions in any design basis accidents evaluated in the UFSAR, and (2) an adequate number of plant parameters are monitored to detect a design basis accident and to initiate appropriate actions. The licensee also stated that all the other TS operability and surveillance requirements associated with the primary containment isolation and the secondary containment remain unchanged. Further, the design and operation of the secondary containment differential pressure automatic isolation instrumentation also remain unchanged.

The proposed amendment does not change the design, function, or operation of any plant components or safety-related systems including the secondary containment differential pressure automatic isolation instrumentation, primary containment, secondary containment, SGTS, RERS, Reactor Enclosure heating, ventilation, and air conditioning (HVAC), or Refueling Area HVAC. No changes are made to the separation, redundancy, qualification, quality assurance or fire protection requirements for the associated components and systems, nor are there any new failure modes created, and no equipment important to safety is adversely affected by this change. The licensee has further stated that the secondary containment differential pressure instrumentation will continue to initiate the appropriate actions if reactor building/refueling area differential pressure cannot be maintained. If differential pressure cannot be restored within the associated time delay, following a Loss-of-Offsite Power, the appropriate containment isolation, SGTS and RERS initiation actions will still occur. Based on the evaluation of the licensee's amendment, the staff concludes that the deletion to the operability and surveillance

requirements involving secondary containment differential pressure instrumentation is in accordance with NUREG-1433, and acceptable to the staff.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 49942). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: February 14, 1996