

June 5, 2001

Mr. Ron J. DeGregorio
Vice President Oyster Creek
AmerGen Energy Company, LLC
P.O. Box 388
Forked River, New Jersey 08731

SUBJECT: OYSTER CREEK GENERATING STATION - NRC INTEGRATED INSPECTION
REPORT 50-219/01-04

Dear Mr. DeGregorio:

On May 12, 2001, the NRC completed an inspection at your Oyster Creek reactor facility. The enclosed report presents the results of that inspection. The results of this inspection were discussed on May 24, 2001, with you and other members of your staff.

This inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

No findings of significance were identified.

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Mr. Ron J. DeGregorio

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We appreciate your cooperation. Please contact me at 610-337-5146 if you have any questions regarding this letter.

Sincerely,

/RA/

John F. Rogge, Chief
Projects Branch No. 7
Division of Reactor Projects

Docket No. 50-219
License No. DPR-16

Enclosure: Inspection Report 50-219/01-04
Attachment: Supplemental Information

cc w/encl:

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U. S. NUCLEAR REGULATORY COMMISSION

REGION I

Report No. 50-219/01-04

Docket No. 50-219

License No. DPR-16

Licensee: AmerGen Energy Company, LLC (AmerGen)

Facility: Oyster Creek Generating Station

Location: Forked River, New Jersey

Dates: April 1, 2001- May 12, 2001

Inspectors: Laura A. Dudes, Senior Resident Inspector
Thomas R. Hipschman, Resident Inspector
Gregory C. Smith, Senior Physical Security Inspector
Suresh Chaudhary, Senior Reactor Engineer, April 2 - 6, 2001
John R. McFadden, Health Physicist, May 1 - 4, 2001
Gregory Cranston, Reactor Inspector, May 14 - 18, 2001

Approved By: John F. Rogge, Chief
Projects Branch 7
Division of Reactor Projects

SUMMARY OF FINDINGS

IR 05000219-01-004, 04/01-05/12/01, AmerGen, Oyster Creek Generating Station, resident inspector report.

The inspection was conducted by resident and region based inspectors. The significance of issues is indicated by their color (green, white, yellow, red) and was determined by the Significance Determination Process (SDP) in Inspection Manual 0609. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/NRR/OVERSIGHT/index.html>.

A. Inspector Identified Findings

- No findings of significance were identified.

B. Licensee Identified Violations

- No findings of significance were identified.

Report Details

Summary of Plant Status:

Oyster Creek began the inspection period at full power and remained there for the duration of the inspection period.

1. REACTOR SAFETY Initiating Events, Mitigating Systems, Barrier Integrity (REACTOR-R)

1R04 Equipment Alignment

.1 Emergency Service Water Full System Walkdown

a. Inspection Scope

The inspector conducted a complete system walkdown of the emergency service water (ESW) system to verify operability of one redundant train should the other train be inoperable or out-of-service. The inspector also walked down that portion of the normal service water system that supplies water for the ESW keep-full system. The inspector verified that there were no equipment alignment deficiencies, no outstanding maintenance work requests that would affect the ability of the ESW system to perform its function, or any other discrepancies that could impact the function of the system and therefore potentially increase risk. There were no outstanding design issues or operator work-arounds. The inspector verified that equipment was correctly labeled and maintained.

Additionally, the inspector, based on a review of the licensee's corrective action process (CAP) documents associated with the ESW system written by the licensee over the past two years, verified that the licensee has properly identified and resolved equipment problems that could cause initiating events or impact the system's availability to mitigate events. Associated documents reviewed are listed below.

CAPs

- O2000-0977 New ESW "A" motor failed during testing.
- O2000-0988 ESW Pump 52C failed to meet acceptance criteria.
- O2000-1105 ESW Pump 52D failed surveillance.
- O2000-1162 ESW system 2 operability, 51C pump vibrations high.
- O2000-1234 Possibly the ESW piping overboard line downstream of V-3-87 is leaking.
- O2000-1291 V-3-133 leaks excessively
- O2000-2075 V-3-132 (keep-full isolation to ESW system 2) leaks by excessively.
- O2001-0044 ESW Pump 52A motor exhibited a vibration level greater than the Alert level.
- O2001-0053 ESW check valve backflow.
- O2001-0056 Vibration instruments issue.
- O2001-0094 ESW pump 52B vibration readings exceeded Alert value.
- O2001-0535 ESW pump 51A vibration above Alert level but below Action level.

Procedures, Drawings, and Other Documents

- Reactor & Turbine Building Service Water System Flow Diagram, BR 2005, Sheet 2 of 6, Rev. 80.
- Emergency Service Water System Flow Diagram, BR 2005, Sheet 4 of 6, Rev. 69.
- Station Procedure 310, "Containment Spray System Operation," Rev. 74 (procedure includes detailed instructions for the operation of the Containment Spray System and the ESW system, including valve line-ups).
- System Health Report, System 532, Emergency Service Water, 1st Quarter 2001.
- Oyster Creek Nuclear Generating Station Updated Final Safety Analysis Report, Chapter 6, Engineered Safety Features.

b. Findings

No findings of significance were identified.

.2 Condensate Transfer System Partial Walkdown

a. Inspection Scope

The inspector performed a partial walkdown of accessible areas of the condensate transfer system. The inspector used procedure 316.1, "Condensate Transfer System," to verify the valve line-up of the system to provide make-up water to the isolation condenser shell. The inspector also used portions of procedure 307, "Isolation Condenser System," to perform a walkdown of the equipment necessary to provide condensate transfer to the isolation condenser shells.

b. Findings

No findings of significance were identified.

1R05 Fire Protection

a. Inspection Scope

The inspectors conducted fire protection inspection activities consisting of plant walkdowns, discussions with fire protection personnel, and reviews of procedure 333, "Plant Fire Protection System," and the Oyster Creek Fire Hazards Analysis Report to verify that the fire program was implemented in accordance with all conditions stated in the facility license. Plant walkdowns included observations of combustible material control, fire detection and suppression equipment availability, and compensatory measures. The inspectors conducted fire protection inspections in the following areas due to the potential to impact mitigating systems:

- Upper Cable Spreading Room
- 480 Volt Switchgear Room
- Control of Combustible in Emergency Diesel Generators 1 and 2
- 4160 Vital Switchgear Room CO2 Suppression System
- Fire Hose Stations, Fire Hydrants
- Station Blackout Transformer Deluge System

b. Findings

No findings of significance were identified.

1R12 Maintenance Rule Implementation

.1 Maintenance Rule Evaluation (Biennial)

a. Inspection Scope

The inspectors reviewed the periodic evaluations required by 10 CFR 50.65 (a)(3) for Oyster Creek Station, to verify that structures, systems and components (SSCs) within the scope of the maintenance rule were included in the evaluations, and balancing of reliability and unavailability was given adequate consideration. The inspectors reviewed the licensee's most recent periodic evaluation reports. The last periodic report for Oyster Creek Station covered the period from January 1997 through December 1999.

The inspectors selected the safety significant systems that were in (a)(1) status to verify that; (1) goals and performance criteria were appropriate, (2) industry operating experience was considered, (3) corrective action plans were effective, and (4) performance was being effectively monitored. As of April 4, 2001, there were eighteen SSCs in (a)(1) status, out of which six were in the monitoring status, and twelve in various stages of evaluation and corrective action. The inspectors also reviewed the licensee's assessment of the balance between reliability and availability for these systems.

(a)(1) Systems selected for detailed review:

- Emergency Service Water
- Service Water
- Feed Water
- 120 V DC Vital Power
- 'B' Train of the Control Room HVAC

Additionally, status and documentation for the following systems were also reviewed:

- Shutdown Cooling
- Instrument Air

The inspector reviewed the following (a)(2) high safety significant systems to verify that performance was acceptable:

- 'A' Train Control Room HVAC
- Emergency Diesel Generators

b. Findings

No findings of significance were identified.

2. Emergency Service Water Maintenance Rule Review

a. Inspection Scope

The inspector conducted a maintenance rule implementation review of the ESW system to determine the effectiveness of the licensee's evaluation of associated ESW availability and reliability issues, ESW system classification as either 10 CFR 50.65 (a)(1) or (a)(2) status, existence of any applicable operation experience program issues, and how ESW maintenance rule issues are addressed in the licensee's problem identification and resolution program.

The inspector observed that the ESW system is currently 10 CFR 50.65 (a)(1) status due to repeat problems with the ESW pump impellers, which previously rendered the ESW pumps inoperable. As of April 4, 2001, all four ESW pumps had been modified to prevent recurrence of those problems by replacing the impellers with a new, keyed design. No problems have occurred to date with the new, keyed impellers. However, based on historic time to failure, additional time is required to confirm that the new keyed impeller design will correct the problem. Consequently, the ESW system appropriately remained in (a)(1) status and the licensee is appropriately keeping this issue open and tracking it in their corrective action program.

Associated documents reviewed are listed below.

CAPs

- O1998-0645 ESW pump 52D flow below Alert level but greater than Action level.
- O1999-0045 ESW pump 52B flow below Alert level.
- O1999-0056 ESW pump 52D failed surveillance test on flow.
- O1999-0678 Loud high pitch noise came from ESW pump 52C when started.
- O1999-1123 ESW pump 52D flow was out of specification below Alert level.
- O2000-0988 ESW Pump 52C failed to meet acceptance criteria.
- O2000-1105 ESW Pump 52D failed surveillance.
- O2000-1162 ESW system 2 operability, 51C pump vibrations high.
- O2000-1848 ESW pump 52B is between the low alert & low action for flow.
- O2000-2075 V-3-132 (keep-full isolation to ESW system #2) leaks by excessively.
- O2001-0044 ESW Pump 52A motor exhibited a vibration level greater than the Alert level.
- O2001-0056 Vibration instruments issue.
- O2001-0094 ESW pump 52B vibration readings exceeded Alert value.
- O2001-0535 ESW pump 51A vibration above Alert level but below Action level.

Other Documents

- System Health Report, System 532, Emergency Service Water, 1st Quarter 2001.
- Job Order (JO) 549208, permanent repair for ESW pipe supports at intake structure to replace temporary modification.
- JO 547698, replacement of the leaking pipe and valves in keep-full system.
- JO 532985 and JO 500001, replacement of corroded conduit supports for the ESW 52A and 52B pump motor power feeders.
- Temporary Modification MV2000-067, ESW system pipe supports at the intake structure.
- Oyster Creek Maintenance Rule Periodic (a)(3) Assessment, January 1, 1998 - December 31, 1999.
- Oyster Creek Maintenance Rule (a)(1) SSCS Evaluation Forms, Evaluation Number 99-005, 12/27/2000.

b. Findings

No findings of significance were identified.

.3 Maintenance Rule 10 CFR 50.65 a(2) Quarterly Review

a. Inspection Scope

The inspectors selected the following safety significant systems in (a)(2) status to verify that: (1) failed SSCs were properly characterized, (2) goals and performance criteria were appropriate, (3) corrective action plans were appropriate, and (4) performance was being effectively monitored:

- 4160V Switchgear Room
- Condensate Transfer Water System
- Condensate Storage Tank

b. Findings

No findings of significance were identified.

1R13 Maintenance Risk Assessments and Emergent Work Evaluation

34.5KV Start-up Transformer Voltage Regulator

a. Inspection Scope

The inspector reviewed JO 549191, "Bank 5 Voltage Regulator Replacement," verified the implementation of the procedure, and reviewed the risk assessment of this maintenance activity with respect to 10 CFR 50.65(a)(4). The inspector reviewed the outage risk analysis to assure that concurrent work would not negatively impact the overall safety of the facility. The inspector also reviewed CAP 2001-0800 which documented licensee identified work process issues.

b. Findings

No findings of significance were identified.

1R15 Operability Evaluations

a. Inspection Scope

The inspectors reviewed operability determinations associated with the following plant equipment deficiencies to verify that all equipment was capable of performing its design basis function and in order to determine that operability justifications were performed in accordance with procedures OC-2, "Operability Review and Analysis," and 2000-ADM-7216.01, "Corrective Action Process." In addition, where a component was determined to be inoperable, the inspectors verified the technical specification (TS) limiting condition for operation implications were properly addressed.

- High resistance noted on "A" Electromatic Relief Valve Pressure Switch PS-IA0083A, April 3, 2001, CAP 2001-532,
- Standby liquid control system squib valve continuity meter pegged high, evaluated for valve circuit operability,
- Manual vs. Plant Computer System Heat Balance Discrepancy, CAP 2001-0606.

b. Findings

No findings of significance were identified.

1R19 Post Maintenance Testing

a. Inspection Scope

The inspector reviewed and observed portions of the post maintenance testing associated with the following maintenance activities because of their function as mitigating systems and their potential role in increasing plant transient frequency. The inspectors reviewed and/or observed several post-maintenance tests (PMTs) to ensure: 1) the PMT was appropriate for the scope of the maintenance work completed; 2) the acceptance criteria were clear and demonstrated operability of the component; and 3) the PMT was performed in accordance with procedures. The following PMTs were observed:

- Troubleshoot and Repair Electrical Components, V-16-0014, Reactor Water Cleanup Valve
- JO 549191, "34.5KV Start-up Transformer Voltage Regulator"

b. Findings

No findings of significance were identified.

1R22 Surveillance Testing

.1 Electromatic Relief Valve Pressure Sensor Test and Calibration

a. Inspection Scope

The inspector reviewed surveillance procedure 602.3.004, "Electromatic Relief Valve Pressure Sensor Test and Calibration." The inspector verified that the performance and resulting data associated with the surveillance test met the requirements of technical specification requirements. The inspector also reviewed CAP 2001-0532 to verify that test discrepancies were identified and resolved in accordance with appropriate procedures.

b. Findings

No findings of significance were identified.

.2 Six Month Diesel Generator Inspection

a. Inspection Scope

The inspector reviewed procedure 636.200.2, "Six Month Diesel Generator Inspection." The inspector verified that the equipment met all requirements for operability as described in the surveillance procedure. In addition, the inspector reviewed CAP 2001-0555 to verify that problems associated with an inadvertent trip were appropriately corrected.

b. Findings

No findings of significance were identified.

.3 Diesel Generator Load Test

a. Inspection Scope

The inspector reviewed procedure 636.4.003, "Diesel Generator No. 1 Load Test," and observed portions of the surveillance test in progress. The inspector verified that the equipment met all requirements for operability as described in the surveillance procedure.

b. Findings

No findings of significance were identified.

.4 'C' Battery Room Fire Damper Functional Test

a. Inspection Scope

The inspector reviewed procedure 645.6.026, "Fire Damper Functional Test." The inspector verified that the equipment met all requirements for operability as described in the surveillance procedure.

b. Findings

No findings of significance were identified.

**2. RADIATION SAFETY
Occupational Radiation Safety (OS)**

2OS1 Access Control To Radiologically Significant Areas

a. Inspection Scope

The inspector toured the facilities and inspected procedures, procedural implementation, records, and other program documents to evaluate the effectiveness of the licensee's access controls to radiologically significant areas. The inspector observed activities in the radiologically controlled area (RCA) to verify compliance with requirements for RCA entry and exit, wearing of record dosimetry, and issuance and use of electronic dosimeters, including the set points used for dose and dose rate alarms. On three separate days, the inspector toured in the RCA, including the reactor, turbine, and new radioactive waste buildings and the outside areas within the RCA boundary. During these tours, the inspector performed independent radiation level measurements and reviewed the posting, labeling, barricading, and level of access control for locked high radiation areas (LHRAs), high radiation areas (HRAs), radiation and contamination areas, and radioactive material areas. On May 2, 2001, the inspector observed the pre-job brief for and the radiological controls implemented during the installation of the lid onto a high integrity container (HIC) and of the secondary lid onto the cask containing the HIC.

The inspector reviewed the following radiation work permits (RWPs) and surveys for the adequacy of radiological survey data, required radiological controls and personal protective equipment, and instructions to radiation workers.

- RWP# 549333, Rev. 0, "Observation/inspection, working visitors"
- RWP# 542672, Rev. 1, "Operation and maintenance of waste processing system, packaging, and shipping of radioactive material"
- Radiological survey No. N3A-01-1104, May 2, 2001, "Remove fill head and install cask and HIC lids"
- Radiological survey No. N3A-01-1105, May 2, 2001, "Depost area from a LHRA after cask removal"

The inspector also reviewed selected sections of the following procedures and documents to evaluate their adequacy and compliance with applicable regulations.

- Procedure 6630-ADM-4110.04, Rev. 8, "Radiological work process"
- Procedure 6630-ADM-4110.06, Rev. 17, "Control of locked high radiation areas (LHRAs)"
- Procedure 6630-ADM-4200.01, Rev. 6, "Radiological Surveys"
- Procedure 6633-ADM-4241.07, Rev. 15, "Personnel dosimetry requirements"
- Oyster Creek exposure summary report for the week of Monday, April 23
- Oyster Creek excellence plan - Radiological Safety
- Self-assessment SA-2001-5030, Regulatory Guide 1.16 compliance, March 7, 2001
- Year 2001 self-assessment schedule for Oyster Creek
- 10 CFR 50.75(g) Decommissioning file

The inspection included a review of the following CAP items for the appropriateness and adequacy of event categorization, immediate corrective action, corrective action to prevent recurrence, and timeliness of corrective action: CAP Nos. O2001-0307, O2001-0454, O2001-0571, O2001-0585, O2001-0632, O2001-0674, and O2001-0776.

The review was against criteria contained in 10 CFR 19.12, 10 CFR 20 (Subparts D, F, G, H, I, and J), site Technical Specifications, and site procedures.

b. Findings

No findings of significance were identified.

2OS2 ALARA Planning and Control

a. Inspection Scope

The inspector toured the facilities and inspected procedures, procedural implementation, records, and other program documents to determine the effectiveness of ALARA (As Low As Reasonably Achievable) planning and control.

The inspector reviewed the following procedure and program documents and, on May 2, 2001, observed a Rad Performance Committee meeting and a Station ALARA Committee meeting.

- Procedure 6630-ADM-4010.02, Rev. 10, "Conduct of radiological engineering"
- Procedure 6630-ADM-4110.02, Rev. 0, "ALARA suggestion program"
- Procedure 6630-ADM-4110.03, Rev. 0, "Station ALARA committee"
- Procedure ES-007, Rev. 4, "ALARA guidelines for configuration changes"
- Oyster Creek excellence plan - Radiological Safety
- Agenda for Station ALARA committee meeting on May 2, 2001
- Minutes for Station ALARA committee meeting on April 12, 2001
- Agenda for Rad Performance Committee meeting on May 2, 2001
- Minutes for Rad Performance Committee meeting on April 4, 2001

The inspector reviewed the following pre-job ALARA Reviews, associated with Radiological Engineering Reviews (RERs), for the adequacy of scope and of documentation.

- RER No. 02B, Turbine building condenser bay 1-2 - Drain tank level column replacement
- RER No. 03A, Low level rad waste storage facility (LLRWSF) - Repackage control-rod-blade guides for shipment
- RER No. 04F, Refuel floor - Remove and ship surveillance coupons

The review was against criteria contained in 10 CFR 20.1101, 10 CFR 20.1702, site Technical Specifications, and site procedures.

b. Findings

No findings of significance were identified.

**3. SAFEGUARDS
Physical Protection (PP)**

3PP4 Security Plan Changes

a. Inspection Scope

An in-office review was conducted of changes to the Physical Security, Contingency, and Training and Qualification Plans, identified as Revisions 40, 3 and 5, respectively, submitted to the NRC on August 29, 2000, in accordance with the provisions of 10 CFR 50.54(p). The review was conducted to confirm that the changes were made in accordance with 10 CFR 50.54(p), and did not decrease the effectiveness of the plans.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA1 Performance Indicator Verification

.1 Unplanned Scrams per 7000 Critical Hours

a. Inspection Scope

The inspectors reviewed performance indicator (PI) data from the 2nd quarter of 2000, through the 1st quarter of 2001, for *Unplanned Scrams per 7000 Critical Hours* to verify its accuracy. The inspectors used Nuclear Energy Institute (NEI) 99-02, Revision 0, "Regulatory Assessment Performance Indicator Guideline," as guidance.

b. Findings

No findings of significance were identified.

.2 Unplanned Power Changes Per 7000 Critical Hours

a. Inspection Scope

The inspectors reviewed performance indicator (PI) data from the 2nd quarter of 2000, through the 1st quarter of 2001, for *Unplanned Power Changes (>20%) Per 7000 Critical Hours* to verify its accuracy. The inspectors used NEI 99-02, Revision 0, "Regulatory Assessment Performance Indicator Guideline," as guidance.

b. Findings

No findings of significance were identified.

40A6 Meetings, including Exit

Exit Meeting Summary

On May 24, 2001, the resident inspectors presented the inspection results to Mr. Ron DeGregorio and other members of licensee management. The licensee acknowledged the findings presented. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

ATTACHMENT 1**SUPPLEMENTAL INFORMATION**a. Key Points of Contact

V. Aggarwal, Director, Engineering
 J. Cicalo, Radiological Protection Technician
 D. McMillan, Manager, Regulatory Affairs
 M. Massaro, Director, Outage Management
 R. DeGregorio, Vice President
 J. Grisewood, EP Manager
 R. Heffner, Radiological Engineer
 E. Harkness, Plant Manager
 R. Hillman, Manager, Chemistry & Radwaste
 A. Judson, Radiological Engineer
 J. Magee, Director, Maintenance
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 M. Moore, Radiation Protection Manager
 K. Mulligan, Director, Training
 J. Murphy, Radiological Engineer
 T. Powell, Maintenance Rule Coordinator
 J. Rogers, Senior Licensing
 P. Sawyer, Radiological Engineering Manager
 G. Scheffing, System Engineer, ESW
 G. Seals, Radiological Engineer
 D. Slear, Senior Manager, Design
 P. Scallon, Acting Manager, Assessment
 C. Wilson, Senior Manager, Operations
 K. Zadroga, Radiation Protection Supervisor

b. List of Acronyms

ADAMS	Agencywide Documents Access and Management System
ALARA	As Low As Reasonably Achievable
AmerGen	AmerGen Energy Company, LLC
CAP	Corrective Action Process
CFR	Code of Federal Regulations
DRP	Division of Reactor Projects
DRS	Division of Reactor Safety
ESW	Emergency Service Water
HIC	High Integrity Container
HRA	High Radiation Area
HVAC	Heating, Ventilation and Air Conditioning
IST	Inservice Test
JO	Job Order
LER	Licensee Event Report
LHRA	Locked High Radiation Area

LLRWSF	Low Level Rad Waste Storage Facility
NCV	Non-Cited Violation
NEI	Nuclear Energy Institute
NRC	Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
NSIC	Nuclear Safety Information Center
PI	Performance Indicator
RCA	Radiologically Controlled Area
RER	Radiological Engineering Review
RWP	Radiation Work Permit
SDP	Significance Determination Process
SSCs	System, Structure and Components
TS	Technical Specification