

Mr. A. Alan Blind  
Vice President, Nuclear Power  
Consolidated Edison Company  
of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, NY 10511

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 - AMENDMENT RE:  
ADMINISTRATIVE CHANGES TO TECHNICAL SPECIFICATIONS  
(TAC NO. MA9033)

Dear Mr. Blind:

The Commission has issued the enclosed Amendment No. \_\_\_\_\_ to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated February 14, 2000, as supplemented on May 3, 2001.

The amendment revises the TSs to correct various editorial errors and make other administrative changes. Specifically, the amendment makes administrative changes that revise: (a) Tables 3.6-1 and 4.4-1 to correct listing and editorial errors, (b) TS 3.8.B.10 to reflect the wording in 10 CFR 50.54(m)(2)(iv), (c) Figures 3.10-2 through 3.10-6 to remove these figures, (d) Table 4.1-1 to reflect change in level indication components, (e) TS 4.19.B and 6.14.1.1 to correct editorial errors, (f) TS 6.12.1 to reflect an organizational title change, and (g) TS 6.13.2 to correct a typographical error. In your May 3 letter, you requested that the proposed changes to TS 6.12.1 regarding references to the current sections of 10 CFR Part 20 be withdrawn.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

Patrick D. Milano, Senior Project Manager, Section 1  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. \_\_\_\_\_ to DPR-26  
2. Safety Evaluation

cc w/encls: See next page  
 Mr. A. Alan Blind  
 Vice President, Nuclear Power  
 Consolidated Edison Company  
 of New York, Inc.  
 Broadway and Bleakley Avenue  
 Buchanan, NY 10511

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Docket No. 50-247

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cc w/encls: See next page

ADAMS Accession No.:ML011560813

OFFICE	PM:PDI-1	LA:PDI-1	SC:SPLB	SC:SRXB	SC:IQMB
NAME	PMilano	SLittle	GHubbard	EWeiss	DDorman
DATE	06/ /01	06/ /01	06/ /01	06/ /01	06/ /01
OFFICE	OGC	(A)SC:PDI-1			
NAME		RCorreia			

DATE	06/	/01	06/	/01			
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DATED: \_\_\_\_\_

AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-26-INDIAN POINT  
UNIT 2

PUBLIC

PDI-1 Reading

OGC

R. Correia

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CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.  
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated February 14, 2000, as supplemented on May 3, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. \_\_\_\_\_, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard P. Correia, Acting Chief, Section 1  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENT NO. \_\_\_\_\_

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

3.3-13  
Table 3.6-1  
3.8-3  
3.10-2  
3.10-3  
3.10-4  
3.10-5  
3.10-6  
Table 4.1-1 (Page 2 of 8)  
Table 4.4-1 (Page 9 of 9)  
4.19-1  
6-12  
6-13

Insert Pages

3.3-13  
Table 3.6-1  
3.8-3  
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-----  
Table 4.1-1 (Page 2 of 8)  
Table 4.4-1 (Page 9 of 9)  
4.19-1  
6-12  
6-13

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. \_\_\_\_\_ TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

## 1.0 INTRODUCTION

By letter dated February 14, 2000, as supplemented on May 3, 2001, the Consolidated Edison Company of New York, Inc. (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications (TSs). The requested changes would revise the TSs to correct various editorial errors and make other administrative changes. Specifically, the amendment makes administrative changes that revise: (a) Tables 3.6-1 and 4.4-1 to correct listing and editorial errors, (b) TS 3.8.B.10 to reflect the wording in 10 CFR 50.54(m)(2)(iv), (c) Figures 3.10-2 through 3.10-6 to remove these figures, (d) Table 4.1-1 to reflect change in level indication components, (e) TS 4.19.B and 6.14.1.1 to correct editorial errors, (f) TS 6.12.1 to reflect an organizational title change, and (g) TS 6.13.2 to correct a typographical error. The May 3 letter, requested that the proposed changes to TS 6.12.1 regarding references to the current sections of 10 CFR Part 20 be withdrawn. Further, the May 3, 2000, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

## 2.0 EVALUATION

In its February 14, 2000, letter, the licensee proposed a number of administrative and editorial changes to correct errors in the facility TSs. These changes were described as:

### 2.1 Changes To Tables 3.6-1 and 4.4-1 to Correct Listing and Editorial Errors

In Table 3.6-1, "Non-Automatic Isolation Valves Open Continuously or Intermittently for Plant Operation," the licensee proposed to: (1) delete valves 990A and 990B, (2) change the valves grouping designation in Note 1 from SWN-77 series to SWN-71 series, and (3) add valves IIP-500 through IIP-507. In addition, the licensee proposed the addition of valves IIP-500 through IIP-507 to Table 4.4-1, "Containment Isolation Valves."

The licensee indicated that valves 990A and 990B are normally-closed isolation valves for the recirculation pump discharge sample line and are only designed to be opened intermittently after an accident to sample the recirculation fluid. Further, these motor-operated valves receive a close signal from the Phase A Containment Isolation Signal. Thus, valves 990A and 990B do not fit the category of non-automatic isolation valves open continuously or intermittently for plant operation and should not be listed on Table 3.6-1.

The licensee also proposed to correct the valve group series in Note (1) to Table 3.6-1 by changing SWN-77 to SWN-71. SWN-71 series is the correct group designation, and no SWN-77 series is listed in Table 3.6-1. Also, this change is consistent with a similar note in Table 4.4-1.

The licensee proposed the addition of valves IIP-500 through IIP-507 to Tables 3.6-1 and 4.4-1. These are normally closed, but intermittently operated, manually-operated isolation valves in the indication lines for steam generator level, pressurizer level, and pressurizer pressure. The licensee stated that these valves are listed in Updated Final Safety Analysis Report (UFSAR) Table 5.2-1 and the Inservice Testing Program, and thus, TS Tables 3.6-1 and 4.4-1 were being updated to show these valves.

In its letter of May 3, 2001, the licensee responded to the NRC's request for additional information of March 29, 2001, in which it clarified whether these valves were listed in the appropriate plant operating and emergency procedures. In this regard, the licensee stated that Station Operating Procedure SOP-10.6.4, "Operation and Control of Non-Automatic Containment Isolation Valves," provides the direction to ensure that these valves are closed during post-accident period. Further, the licensee stated that the appropriate controls during normal and emergency operations for these valves are included in Administrative Operating Instruction AOI-27.1.9, "Control Room Inaccessibility Safe Shutdown Control," and Check-off Lists COL-1.1, "Reactor Coolant Systems," and COL-10.6.2, "Containment Integrity."

On the basis of the above discussion, the NRC staff finds these changes to be acceptable.

## 2.2 Change to TS 3.8.B.10 to Reflect the Wording in 10 CFR 50.54(m)(2)(iv)

The current TS 3.8.B.10 specifies that "A licensed senior reactor operator shall be at the site and designated in charge of the operation whenever changes in core geometry are taking place." The licensee has proposed to revise TS 3.8.B.10 to read: "During alteration of the core (including fuel loading or transfer), a person holding a senior operator license or a senior operator license limited to fuel handling shall be present to directly supervise the activity and, during this time, this person shall not be assigned other duties."

The licensee noted it is required to comply with 10 CFR 50.54(m)(2)(iv) which states:

"Each licensee shall have present, during alteration of the core of a nuclear power unit (including fuel loading or transfer), a person holding a senior operator license or a senior operator license limited to fuel handling to directly supervise the activity and, during this time, the licensee shall not assign other duties to this person."

Thus, the licensee proposes this change to remove any ambiguity that may have existed between the current TS 3.8.B.10 requirement and 10 CFR 50.54(m)(2)(iv).

The NRC staff finds that the proposed changes to TS 3.8.B.10 are in accordance with 10 CFR 50.54(m)(2)(iv) and are acceptable.

### 2.3 Deletion of Figures 3.10-2 through 3.10-6

The licensee proposes to delete TS Figures: (a) 3.10-2, "Hot Channel Factor Normalized Operating Envelop," (b) 3.10-3, "Rod Bank Insertion Limits," 3.10-5, "Target Band on Indicated Flux Difference as a Function of Operating Power Level," and 3.10-6, "Permissible Operating Band on Indicated Flux Difference as a Function of Burnup." The licensee also noted that Figure 3.10.4 was previously deleted by Amendment No. 152, dated June 26, 1990.

The licensee stated that in Amendment No. 194, dated June 27, 1997, TS Section 3.10, "Control Rod and Power Distribution Limits," was revised and references to the Figures 3.10-2 through 3.10-6 were eliminated. In lieu of these figures, the amended section referenced the Core Operating Limits Report (COLR). Thus, TS Figures 3.10-2 through 3.10-6 should have been deleted, but were inadvertently left in, when Amendment No. 194 was issued.

The NRC staff finds that the current TSs for the power distribution and rod insertion limits reference the COLR for the applicable limits. Thus, the staff finds that the change is acceptable since TS Figures 3.10-2 through 3.10-6 should have been deleted in Amendment 194.

### 2.4 Change to Table 4.1-1 to Reflect Change in Level Indication Components

In the Remarks column for Item 14, "Boric Acid Tank Level," in TS Table 4.1-1, "Minimum Frequencies for Checks, Calibrations and Tests of Instrument Channels," it is stated that the "Bubbler tube rodded during calibration." The licensee proposes to delete this statement.

The licensee stated that the original design for Boric Acid Storage Tanks included a nitrogen bubbler system to determine the level in each tank. However, this tank level instrumentation experienced frequent problems because of plugging in the bubbler tube from the crystallization of boric acid. To rectify the problem the bubbler tube had to be rodded out to clear the blockage. Under the original configuration, the statement, "Bubbler tube rodded during calibration," was included since if the bubbler tube was not rodded during the calibration of the level sensor, there could be an inaccurate calibration which could result in erroneous level indication.

Subsequently, the licensee has modified the boric acid storage tanks level instrumentation to a non-intrusive system having no contact with the process fluid. The modification removed the bubbler system and added a microwave transmitter/receiver sensor mounted above the top of each tank with a process seal to completely isolate the instrument from the tank contents.

The NRC staff finds that the remark to rod the bubbler tube during calibration is no longer applicable due to the modification replacing the level instrumentation. This remark should have been deleted when the modification was made. Thus, the NRC staff finds the change acceptable.

### 2.5 Change To Sections 4.19.B And 6.14.1.1 To Correct Editorial Errors

In Section 4.19.B and in Section 6.14.1.1, the licensee proposes to change "Semiannual Radioactive Effluent Release Report" to "Annual Radioactive Effluent Release Report."

The licensee stated that in Amendment No. 172, dated July 21, 1994, the submittal frequency of the Radioactive Effluent Release Report was changed from semiannual to annual. In Amendment 198, issued in August 12, 1998, TSs 3.9.A.2.c, 3.9.A.5.b, 3.9.B.2.c, 4.11.A.4, 4.11.B.3, 4.11.B.4, and Table 4.11-1 were revised to replace "Semiannual Radioactive Effluent Release Report" with "Annual Radioactive Effluent Release Report." Thus, Technical Specification Sections 4.19.B and 6.14.1.1 should have been included in the prior amendment.

The NRC staff finds that the frequency was previously approved by the NRC and the proposed change to TS 4.19.B and 6.14.1.1 correct items previously overlooked in the earlier amendment.

#### 2.6 Change to Section 6.12.1 to Reference the Current Sections of 10 CFR Part 20

In a letter dated May 3, 2001, the licensee withdrew its request for changes to TS 6.12.1.

#### 2.7 Change to TS 6.12.1 to Reflect an Organizational Title Change

The licensee proposed to change the title for the "Senior Watch Supervisor" to "Shift Manager" in TS Section 6.12.1. The licensee stated that the change was in title only and not in the responsibilities or functions performed by this individual. The licensee also noted that the change in title will be included in the next UFSAR update.

The NRC staff finds that this change acceptable since the functions and responsibilities of this position have not been revised.

#### 2.8 Change to Section 6.13.2 to Correct a Typographical Error

In Section 6.13.2 change "DOR Guidelines of NUREG-0588" to "DOR Guidelines or NUREG-0588."

Technical Specification Section 6.13.1 states:

"By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors 'Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors' (DOR Guidelines), of NUREG -0588, 'Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment,' December, 1979. Copies of these documents are attached to Order of Modification of License No. DPR-26, dated October 24, 1980."

The licensee stated that the DOR Guidelines and NUREG-0588 are two separate documents. Therefore, the statement: "(DOR Guidelines), of NUREG-0588" should actually be "(DOR Guidelines), or NUREG-0588." The licensee believes that the typographical error occurred since the "r" key is just above and to the left of the "f" key on a keyboard.

The NRC staff finds that the proposed change corrects a typographical error and is acceptable.

## 2.9 Correction of Typographical Error in TS Basis Section 3.3

The licensee stated that the third from the last sentence on page 3.3-13 currently reads: "With two operable component cooling pumps, 100% redundancy will be provide." The word "provide" is grammatically incorrect and should read "provided." The staff does not object to this change since it corrects the tense of this verb and does not change the intent of this statement.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact was published in the Federal Register on July 3, 2001 (66 FR 35282).

Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of these amendments will not have a significant effect on the quality of the human environment.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Clark  
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Date: