

June 11, 1996

Mr. George A. Hunger, Jr.
Director-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 (TAC NOS. M95285 AND M95286)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendment No. 117 to Facility Operating License No. NPF-39 and Amendment No. 79 to Facility Operating License No. NPF-85 for the Limerick Generating Station (LGS), Units 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 25, 1996.

These amendments relocate the TS Traversing In-Core Probe System Limiting Condition of Operation 3/4.3.7.7 and its Bases 3/4.3.7.7, to the LGS Technical Requirements Manual and modify Note (f) of TS Table 4.3.1.1-1.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

- Enclosures: 1. Amendment No. 117 to License No. NPF-39
- 2. Amendment No. 79 to License No. NPF-85
- 3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

Docket File	MO'Brien	CGrimes
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PDI-2 Reading	OGC	ACRS
SVarga	GHill(4)	
JStolz	WPasciak, RGN-I	

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OFFICE	PDI-2/PM	PDI-2/PM	SRXB <i>to</i>	OGC	PDI-2/D
NAME	MO'Brien	FRinaldi: <i>FR</i>	RJones <i>R</i>	BMS	JStolz <i>J</i>
DATE	5/16/96	5/16/96	5/21/96	6/15/96	6/10/96

OFFICIAL RECORD COPY
DOCUMENT NAME: LI95285.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Frank Rinaldi".

Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

- Enclosures:
1. Amendment No. 117 to License No. NPF-39
 2. Amendment No. 79 to License No. NPF-85
 3. Safety Evaluation

cc w/encs: See next page

Mr. George A. Hunger, Jr.
PECO Energy Company

Limerick Generating Station,
Units 1 & 2

cc:

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Board of Supervisors
of Limerick Township
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Linfield, PA 19468



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 117
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated April 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

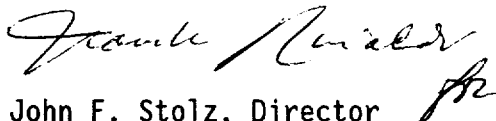
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 117 , are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: June 11, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 117

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

ix
xix
3/4 3-8
3/4 3-89
B 3/4 3-6

Insert

ix
xix
3/4 3-8
3/4 3-89
B 3/4 3-6

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>INSTRUMENTATION</u> (Continued)	
Table 4.3.7.1-1 Radiation Monitoring Instrumentation Surveillance Requirements.....	3/4 3-66
The information from pages 3/4 3-68 through 3/4 3-72 has been intentionally omitted. Refer to note on page 3/4 3-68.....	3/4 3-68
The information from pages 3/4 3-73 through 3/4 3-75 has been intentionally omitted. Refer to note on page 3/4 3-73.....	3/4 3-73
Remote Shutdown System Instrumentation and Controls....	3/4 3-76
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Source Range Monitors.....	3/4 3-88
The information from page 3/4 3-89 has been intentionally omitted. Refer to note on page	3/4 3-89
Chlorine Detection System.....	3/4 3-90
Toxic Gas Detection System.....	3/4 3-91
DELETED; Refer to note on page.....	3/4 3-92

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BASES

<u>SECTION</u>	<u>PAGE</u>
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(Deleted).....	B 3/4 3-5
Remote Shutdown System Instrumentation and Controls....	B 3/4 3-5
Accident Monitoring Instrumentation.....	B 3/4 3-5
Source Range Monitors.....	B 3/4 3-5
(Deleted).....	B 3/4 3-6
Chlorine and Toxic Gas Detection Systems.....	B 3/4 3-6
(Deleted).....	B 3/4 3-6
Loose-Part Detection System.....	B 3/4 3-7
(Deleted).....	B 3/4 3-7
Offgas Monitoring Instrumentation.....	B 3/4 3-7
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Operational Leakage.....	B 3/4 4-3
3/4.4.4 CHEMISTRY.....	B 3/4 4-3

TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
9. Turbine Stop Valve - Closure	N.A.	Q	R	1
10. Turbine Control Valve Fast Closure, Trip Oil Pressure - Low	N.A.	Q	R	1
11. Reactor Mode Switch Shutdown Position	N.A.	R	N.A.	1, 2, 3, 4, 5
12. Manual Scram	N.A.	W	N.A.	1, 2, 3, 4, 5

- (a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decades during each startup after entering OPERATIONAL CONDITION 2 and the IRM and APRM channels shall be determined to overlap for a least 1/2 decades during each controlled shutdown, if not performed within the previous 7 days.
- (c) DELETED
- (d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER $\geq 25\%$ of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER.
- (e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.
- (f) The LPRMs shall be calibrated at least once per 1000 effective full power hours (EFPH).
- (g) Verify measured core flow (total core flow) to be greater than or equal to established core flow at the existing loop flow (APRM % flow). During the startup test program, data shall be recorded for the parameters listed to provide a basis for establishing the specified relationships. Comparisons of the actual data in accordance with the criteria listed shall commence upon the conclusion of the startup test program.
- (h) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.
- (i) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (j) If the RPS shorting links are required to be removed per Specification 3.9.2, they may be reinstalled for up to 2 hours for required surveillance. During this time, CORE ALTERATIONS shall be suspended, and no control rod shall be moved from its existing position.
- (k) Required to be OPERABLE only prior to and during shutdown margin demonstrations as performed per Specification 3.10.3.

LIMERICK - UNIT 1

3/4 3-8

Amendment No. 29, 41, 53, 68, 113, 117

INSTRUMENTATION

Section 3/4.3.7.7

THE INFORMATION FROM THIS TECHNICAL SPECIFICATION
HAS BEEN RELOCATED TO THE TECHNICAL REQUIREMENTS MANUAL (TRM)

INSTRUMENTATION

BASES

3/4.3.7.7 (Deleted) - INFORMATION FROM THIS SECTION RELOCATED TO THE TRM. |

3/4.3.7.8 CHLORINE AND TOXIC GAS DETECTION SYSTEMS

The OPERABILITY of the chlorine and toxic gas detection systems ensures that an accidental chlorine and/or toxic gas release will be detected promptly and the necessary protective actions will be automatically initiated for chlorine and manually initiated for toxic gas to provide protection for control room personnel. Upon detection of a high concentration of chlorine, the control room emergency ventilation system will automatically be placed in the chlorine isolation mode of operation to provide the required protection. Upon detection of a high concentration of toxic gas, the control room emergency ventilation system will manually be placed in the chlorine isolation mode of operation to provide the required protection. The detection systems required by this specification are consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators against an Accidental Chlorine Release," February 1975.

There are three toxic gas detection subsystems. The high toxic chemical concentration alarm in the Main Control Room annunciates when two of the three subsystems detect a high toxic gas concentration. An Operate/Inop keylock switch is provided for each subsystem which allows an individual subsystem to be placed in the tripped condition. Placing the keylock switch in the INOP position initiates one of the two inputs required to initiate the alarm in the Main Control Room.

Specified surveillance intervals and maintenance outage times have been determined in accordance with GENE-770-06-1, "Bases for Changes to Surveillance Test Intervals and Allowed Out-of-Service Times for Selected Instrumentation Technical Specifications," as approved by the NRC and documented in the SER (letter to R.D. Binz, IV, from C.E. Rossi dated July 21, 1992).

3/4.3.7.9 (Deleted) - INFORMATION FROM THIS SECTION RELOCATED TO THE TRM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated April 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

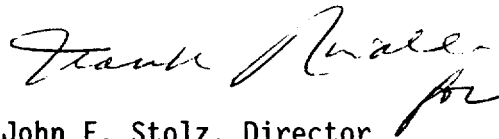
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 79, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: June 11, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

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Remove

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3/4 3-8
3/4 3-89
B 3/4 3-6

Insert

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xix
3/4 3-8
3/4 3-89
B 3/4 3-6

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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DELETED; Refer to note on page	3/4 3-92

BASES

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3/4.4.2	SAFETY/RELIEF VALVES.....	B 3/4 4-2
3/4.4.3	REACTOR COOLANT SYSTEM LEAKAGE	
	Leakage Detection Systems.....	B 3/4 4-3
	Operational Leakage.....	B 3/4 4-3
3/4.4.4	CHEMISTRY.....	B 3/4 4-3a

TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION(a)</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
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12. Manual Scram	N.A.	W	N.A.	1, 2, 3, 4, 5

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(h) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.

(i) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

(j) If the RPS shorting links are required to be removed per Specification 3.9.2, they may be reinstalled for up to 2 hours for required surveillance. During this time, CORE ALTERATIONS shall be suspended, and no control rod shall be moved from its existing position.

(k) Required to be OPERABLE only prior to and during shutdown margin demonstrations as performed per Specification 3.10.3.

INSTRUMENTATION

Section 3/4.3.7.7

THE INFORMATION FROM THIS TECHNICAL SPECIFICATION
HAS BEEN RELOCATED TO THE TECHNICAL REQUIREMENTS MANUAL (TRM)

INSTRUMENTATION

BASES

3/4.3.7.7 (Deleted) - INFORMATION FROM THIS SECTION RELOCATED TO THE TRM.

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Specified surveillance intervals and maintenance outage times have been determined in accordance with GENE-770-06-1, "Bases for Changes to Surveillance Test Intervals and Allowed Out-of-Service Times for Selected Instrumentation Technical Specifications," as approved by the NRC and documented in the SER (letter to R.D. Binz, IV, from C.E. Rossi dated July 21, 1992).

3/4.3.7.9 (Deleted) - INFORMATION FROM THIS SECTION RELOCATED TO THE TRM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 117 AND 79 TO FACILITY OPERATING
LICENSE NOS. NPF-39 AND NPF-85
PHILADELPHIA ELECTRIC COMPANY
LIMERICK GENERATING STATION, UNITS 1 AND 2
DOCKET NOS. 50-352 AND 50-353

1.0 INTRODUCTION

By letter dated April 25, 1996, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Limerick Generating Station (LGS), Units 1 and 2, Technical Specifications (TSs). The requested changes would relocate the TS Traversing In-Core Probe (TIP) System Limiting Condition for Operation (LCO) 3/4.3.7.7 and its Bases 3/4.3.7.7, to the LGS Technical Requirements Manual (TRM), and modify Note (f) of TS Table 4.3.1.1-1, "Reactor Protection System Instrumentation Surveillance Requirements" to remove its reference to the TIP system, in accordance with the guidelines contained in the Improved Standard Technical Specifications, General Electric Plants BWR/4 (NUREG-1433), issued September 28, 1992.

2.0 EVALUATION

The licensee has requested to delete the TIP system requirements, and LCO for TS Sections 3/4.3.7.7 and related Bases, and relocate them in their entirety to the LGS's TRM in accordance with NUREG-1433. In addition, the licensee has requested to modify Note (f) to TS Table 4.3.1.1-1 to delete reference to the TIP system. The licensee has indicated that future changes of these requirements will be evaluated in accordance with the provisions of 10 CFR 50.59.

Further, the licensee has indicated that, as stated in the Updated Final Safety Analysis Report (UFSAR) Section 7.7.1.6.3, the TIP system is an operational system which has no safety function. The TIP system allows calibration of the Local Power Range Monitor, but has no safety setpoints. Other TIP TS requirements would be unaffected by these changes.

The staff has issued guidance in NUREG-1433 for relocating TS requirements. NRC's letter to the BWR Owners Group, dated May 9, 1988, stated that the TIP LCO may be relocated. Further, NUREG-1433 allows for line item improvements, and states the criteria for the removal of LCOs from the TS when they are to be relocated to a licensee's controlled document like the TRM.

The licensee has addressed the criteria contained in 10 CFR 50.36 for removal and relocation of TS requirements to a licensee's controlled document. Specifically, the LGS's TIP system is not: 1) used to prevent degradation of the reactor coolant boundary, 2) part of a Design Basis Accident or a transient analysis that is based on the integrity of the fission product barrier, 3) a portion of the primary success path of a safety sequence analysis, or 4) significant to public health and safety.

The staff concurs with the licensee's determinations, and finds the proposed deletion and relocation of the above stated requirements acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 20840). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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