

August 23, 1994

Docket No. 50-352

Mr. George A. Hunger, Jr.  
Director-Licensing, MC 52A-5  
PECO Energy Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, Pennsylvania 19087-0195

Dear Mr. Hunger:

SUBJECT: REMOVAL OF CONTROLS FOR A REMOTE SHUTDOWN SYSTEM CONTROL VALVE  
AND DELETION OF ISOLATION SIGNAL FOR CERTAIN PRIMARY CONTAINMENT  
ISOLATION VALVES, LIMERICK GENERATING STATION, UNIT 1 (TAC NO.  
M89656)

The Commission has issued the enclosed Amendment No. 74 to Facility Operating License No. NPF-39 for the Limerick Generating Station, Unit 1. This amendment consists of changes to the Technical Specifications (TS) in response to your application dated June 6, 1994.

This amendment removes the controls for a remote shutdown system control valve and deletes the isolation signal for certain primary containment isolation valves from TS Tables 3.3.7.4-1 and 3.6.3-1 respectively, as a result of eliminating the steam condensing mode of the Residual Heat Removal system.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Frank Rinaldi, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 74 to License No. NPF-39
2. Safety Evaluation

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P PDR

*Handwritten initials/signature*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Frank Rinaldi".

Frank Rinaldi, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 74 to License No. NPF-39
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. George A. Hunger, Jr.  
PECO Energy Company

Limerick Generating Station,  
Units 1 & 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74  
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company (the licensee) dated June 6, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 74 , are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan C. Thadani, Acting Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 23, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided to maintain document completeness.\*

Remove

3/4 3-79  
3/4 3-80

3/4 6-29  
3/4 6-30

Insert

3/4 3-79\*  
3/4 3-80

3/4 6-29\*  
3/4 6-30

Table 3.3.7.4-1 (Continued)

RCIC SYSTEM (Continued)

HV-49-1F080	Control-Vacuum breaker outboard isolation valve
HV-49-1F084	Control-Vacuum breaker inboard isolation valve
FIC-49-1R001	Controller-RCIC discharge flow control
E51-S45	RCIC Turbine Trip Bypass

NUCLEAR BOILER SYSTEM

HSS-41-191	Control-Transfer switch
PSV-41-1F013A	Control-Main steam line safety/relief valve
PSV-41-1F013C	Control-Main steam line safety/relief valve
PSV-41-1F013N	Control-Main steam line safety/relief valve

RHR SYSTEM

HSS-51-192	Control-Transfer switch
HSS-51-193	Control-Transfer switch
HSS-51-194	Control-Transfer switch
HSS-51-195	Control-Transfer switch
HSS-51-196	Control-Transfer switch
HSS-51-197	Control-Transfer switch
HSS-51-198	Control-Transfer switch
HV-51-1F009	Control-RHR pump shutdown cooling suction inboard isolation
HV-51-1F008	Control-RHR shutdown cooling suction outboard isolation
HV-51-1F006A	Control-1A RHR loop shutdown cooling suction
HV-51-1F006B	Control-1B RHR loop shutdown cooling suction
HV-51-1F004A	Control-1A RHR pump suction
1AP202	Control-1A RHR pump

TABLE 3.3.7.4-1 (Continued)

RHR SYSTEM (Continued)

HV-43-1F023A	Control-Recirculation pump A suction valve
HSS-43-191	Control-Transfer switch
HV-51-1F007A	Control-1A RHR pump minimum flow bypass valve
HV-51-1F048A	Control-1A heat exchanger shell side bypass
HV-51-1F015A	Control-1A shutdown cooling injection valve
HV-51-1F016A	Control-Reactor containment spray
HV-51-1F017A	Control-1A RHR loop injection valve
HV-51-1F024A	Control-1A RHR loop test return
HV-51-1F027A	Control-Suppression pool sparger isolation
HV-51-1F047A	Control-1A Heat exchanger shell side inlet
HV-51-1F003A	Control-1A Heat exchanger shell side outlet
HV-51-1F049	Control-RHR Discharge to radwaste outboard isolation
HV-51-125A	Control-1A/1C test return line to suppression pool

RHR SERVICE WATER SYSTEM

HSS-12-015A-2	Control-Spray pond/cooling tower select
HSS-12-015C-2	Control-Spray pond/cooling tower select
HSS-12-016A-2	Control-Spray/bypass select
HSS-12-016C-2	Control-Spray/bypass select

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME IF APP. (SEC)(26)	ISOL. SIGNAL(S) IF APP. (20)	NOTES	P & ID
226A	RHR MINIMUM RECIRC		NV51-105A	40		4,22,29	51
226B	RHR MINIMUM RECIRC		NV51-105B	40		4,22,29	51
227	ILRT DATA ACQUISITION SYSTEM	60-1073	60-1074	NA NA			60
228D	NPCI VACUUM RELIEF	NV55-1F095	NV55-1F093	40 40	H,LA H.LA	4,11,24 11,24	55
230B	INSTRUMENTATION - DRYWELL SUMP LEVEL	--	NV61-102 NV61-112 NV61-132	45 45 45		1,23,29 23,29 23,29	61
231A	DRYWELL FLOOR DRAIN SUMP DISCHARGE	NV61-110	11V61-111	30 30	B,H B,H	11,22 11,22	61
231B	DRYWELL EQUIPMENT DRAIN TANK DISCHARGE	NV61-130	NV61-131	30 30	B,H B,H	11,22 11,22	61
235	CS PUMP MINIMUM RECIRC		NV52-1F031A	45	LFCH	5,22,29	52
236	NPCI PUMP MINIMUM RECIRC		NV55-1F012	15	LFHP	5,22	55

LIMERICK UNIT 1

3/4 6-29

Amendment No. 18, 22, 65

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

<u>PENETRATION NUMBER</u>	<u>FUNCTION</u>	<u>INBOARD ISOLATION BARRIER</u>	<u>OUTBOARD ISOLATION BARRIER</u>	<u>MAX. ISOL. TIME. IF APP. (SEC)(26)</u>	<u>ISOL. SIGNAL(S) IF APP. (20)</u>	<u>NOTES</u>	<u>P &amp; ID</u>
237-1	SUPPRESSION POOL CLEANUP PUMP SUCTION	HV52-127	PSV52-127 HV52-128	60 NA 60	B,H B,H	4,11,22 11,22 11,22	52
237-2	SUPPRESSION POOL LEVEL INSTRUMENTATION		HV52-139 SV52-139	45 6		10 10	52
238	RHR RELIEF VALVE DISCHARGE		HV-C-51-1F104B PSV51-106B	NA NA		11 19	51
239	RHR RELIEF VALVE DISCHARGE		HV-C-51-1F103A PSV51-106A	NA NA		11 19	51
241	RCIC VACUUM RELIEF	HV49-1F084	HV49-1F080	40 40	H,KA H,KA	4,11,24 11,24	49

LIMERICK - UNIT 1

3/4 6-30

Amendment No. 63, 74



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. NPF-57

PHILADELPHIA ELECTRIC COMPANY

LIMERICK GENERATING STATION, UNIT 1

DOCKET NO. 50-352

1.0 INTRODUCTION

By letter dated June 6, 1994, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Limerick Generating Station (LGS), Unit 1, Technical Specifications (TS). The requested changes would remove the controls for a remote shutdown system control valve and delete the isolation signal for certain primary containment isolation valves from TS Tables 3.3.7.4-1 and 3.6.3-1 respectively, as a result of eliminating the steam condensing mode of the Residual Heat Removal (RHR) system.

2.0 EVALUATION

The RHR system steam condensing mode has never been used at LGS, Unit 1. Further, the remainder of the steam condensing mode components have been removed from service at LGS, Unit 1, during the fifth refueling outage, as authorized by Amendment No. 65. Currently, valve HV-51-1F011A, is in the locked closed position with its electrical power source removed. The valve's handswitch does not perform any function, and will be removed from the remote shutdown panel. Therefore, the licensee has revised Table 3.3.7.4-1 to reflect this change.

Similarly, valves HV-C-51-1F103A and HV-C-51-1F104B, are locked closed in their safety-related position with their electrical power source removed. These valves are no longer required to receive an isolation signal or meet valve closure time, but provide manual containment isolation.

The proposed TS changes reflect the current physical condition in the plant. The components associated with steam condensing mode have been removed during the Unit 1 fifth refueling outage as per staff authorization in Amendment No. 65, dated January 12, 1994. Therefore, the staff concludes that the proposed TS changes will not impact the performance of the safety-related function of the RHR system components, and are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 37076). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Liu  
F. Rinaldi

Date: August 23, 1994