

June 1, 2001

Mr. Thomas F. Plunkett
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING THE
EMERGENCY DIESEL GENERATOR ALLOWED OUTAGE TIME EXTENSION
FOR TURKEY POINT PLANT, UNITS 3 AND 4 (TAC NOS. MB1408 AND
MB1409)

Dear Mr. Plunkett:

By letter dated March 12, 2001, Florida Power and Light Company requested an amendment to Technical Specification (TS) 3/4.8.1, "A. C. SOURCES" and associated TSs 3/4.4.3 and 3/4.5.2, to extend the allowed outage time for an inoperable emergency diesel generator from 72 hours to 14 days. Additionally, the request proposes to relocate TS Surveillance Requirement 4.8.1.1.2.g.1 to a licensee controlled maintenance program that will be incorporated by reference into the Updated Final Safety Analysis Report. The U.S. Nuclear Regulatory Commission staff has reviewed your submittal and finds that a response to the enclosed request for additional information is needed before we can complete the review.

This request was discussed with your staff on May 31, 2001, and it was agreed that a response would be provided within 30 days of receipt of this letter.

If you have any questions, please contact me at (301) 415-1496.

Sincerely,

/RA/

Kahtan N. Jabbour, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
EMERGENCY DIESEL GENERATOR ALLOWED OUTAGE TIME EXTENSION
AND RELOCATION OF TECHNICAL SPECIFICATION
SURVEILLANCE REQUIREMENT 4.8.1.1.2.g.1
TURKEY POINT PLANT, UNITS 3 AND 4
DOCKET NOS. 50-250 AND 50-251

1. Please discuss the current and planned activities regarding the independent peer review/certification of your probabilistic risk analysis (PRA).
2. Please provide a list of model changes and their associated risk impacts that contributed significantly to the reduction in the baseline core damage frequency following the completion of the original PRA.
3. Please describe the current model used for the potential reactor coolant pump seal loss-of-coolant accident. Describe the differences between the NUREG-1150 model and this model.
4. Please describe the PRA tools used for implementing requirement (a)(4) of the Maintenance Rule. Explain how you address multi-unit risk impact resulting from shared or cross-tied systems, structures and components.
5. Please describe plant enhancements made for the capability to mitigate potential loss-of-offsite power accidents. Specifically, discuss in detail those that removed dependency on electrical power supply or increased redundancy for mitigating systems.

Mr. T. F. Plunkett
Florida Power and Light Company

TURKEY POINT PLANT

cc:

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