



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

January 29, 1993

Docket Nos. 50-352  
and 50-353

Mr. George J. Beck  
Manager-Licensing, MC 52A-5  
Philadelphia Electric Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: LIMERICK GENERATING STATION, UNITS 1 AND 2 (TSCR NO. 90-20-0)  
(TAC NOS. M83237 AND M83238)

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. NPF-39 and Amendment No. 24 to Facility Operating License No. NPF-85 for the Limerick Generating Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated April 3, 1992, as supplemented January 21 and 22, 1993.

These amendments provide a one-time revision of the TS Surveillance Requirements for the Standby Liquid Control (SLC) system to: 1) use the daily check of the SLC pump suction piping temperature to determine system operability, rather than heat tracing system operability; 2) verify that the piping is not blocked by pumping from the storage tank to a test drum, rather than to the test tank; and 3) require only one SLC storage tank heater to be operable, rather than two which are currently required, for the upcoming Unit 2 refueling outage. We are approving the changes for both units, as requested, for consistency in operational requirements. Also, we are processing your January 12, 1993, submittal that requested changes in the TS for the SLC that follow the requirements stated in the standard TS as a separate action.

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PDR ADOCK 05000352  
P PDR

CR1

DF01

Mr. George J. Beck

- 2 -

January 29, 1993

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/S/

Frank Rinaldi, Acting Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 59 to License No. NPF-39  
Amendment No. 24 to License No. NPF-85
- 2. Safety Evaluation

cc w/enclosures:  
See next page

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Docket File	MO'Brien(2)	CGrimes, 11E21	BRuland, RGN-I
NRC & Local PDRs	FRinaldi/JShea	RJones	
PDI-2 Reading	OGC	ACRS(10)	
SVarga	DHagan, 3206	OPA	
JCalvo	GHill(4), PI-22	OC/LFMB	
CMiller	Wanda Jones, 7103	EWenzinger, RGN-I	

OFC	: PDI-2/LA	: PDI-2/PM	: OGC	: PDI-2/D	:
NAME	: MO'Brien	: FRinaldi	: rb	: CMiller	:
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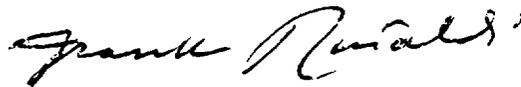
Mr. George J. Beck

- 2 -

January 29, 1993

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,



Frank Rinaldi, Acting Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 59 to  
License No. NPF-39  
Amendment No. 24 to  
License No. NPF-85
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. George J. Beck  
Philadelphia Electric Company

Limerick Generating Station,  
Units 1 & 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59  
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company (the licensee) dated April 3, 1992, as supplemented January 21 and 22, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

9302100489 930129  
PDR ADOCK 05000352  
P PDR

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 59 , are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Charles L. Miller*

Charles L. Miller, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: January 29, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 1-19  
3/4 1-20

Insert

3/4 1-19  
3/4 1-20

## REACTIVITY CONTROL SYSTEMS

### 3/4.1.5 STANDBY LIQUID CONTROL SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.1.5 The standby liquid control system, consisting of a minimum of two pumps and corresponding flow paths, shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 5\*

ACTION:

- a. In OPERATIONAL CONDITION 1 or 2:
  1. With only one pump and corresponding explosive valve OPERABLE, restore one inoperable pump and corresponding explosive valve to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
  2. With the standby liquid control system otherwise inoperable, restore the system to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- b. In OPERATIONAL CONDITION 5\*:
  1. With only one pump and corresponding explosive valve OPERABLE, restore one inoperable pump and corresponding explosive valve to OPERABLE status within 30 days or insert all insertable control rods within the next hour.
  2. With the standby liquid control system otherwise inoperable, insert all insertable control rods within 1 hour.

#### SURVEILLANCE REQUIREMENTS

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4.1.5 The standby liquid control system shall be demonstrated OPERABLE:

- a. At least once per 24 hours by verifying that:
  1. The temperature of the sodium pentaborate solution is within the limits of Figure 3.1.5-1.
  2. The available volume of sodium pentaborate solution is at least 4537 gallons.
  3. The temperature of the pump suction piping is greater than or equal to 70°F.

\*With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

b. At least once per 31 days by:

1. Verifying the continuity of the explosive charge.
2. Determining by chemical analysis and calculation\* that the available weight of sodium pentaborate is greater than or equal to 5389 lbs; the concentration of sodium pentaborate in solution is less than or equal to 13.8% and within the limits of Figure 3.1.5-1 and; the following equation is satisfied:

$$\frac{C}{13\% \text{ wt.}} \times \frac{Q}{86 \text{ gpm}} \geq 1$$

where

C = Sodium pentaborate solution (% by weight)

Q = Two pump flowrate, as determined per surveillance requirement 4.1.5.c.

3. Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. Demonstrating that, when tested pursuant to Specification 4.0.5, the minimum flow requirement of 41.2 gpm per pump at a pressure of greater than or equal to 1190 psig is met.
- d. At least once per 18 months during shutdown by:
1. Initiating at least one of the standby liquid control system loops, including an explosive valve, and verifying that a flow path from the pumps to the reactor pressure vessel is available by pumping demineralized water into the reactor vessel. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of that batch successfully fired. All injection loops shall be tested in 3 operating cycles.
  2. \*\*Demonstrating that all suction piping is unblocked by pumping from the storage tank through each discharge line, and pumping from the test tank back to the test tank through each loop.
  3. Demonstrating that 'A' storage tank heater is OPERABLE\*\*\* by verifying expected temperature rise of the sodium pentaborate solution in the storage tank after the heater is energized.

\*This test shall also be performed anytime water or boron is added to the solution or when the solution temperature drops below 70°F.

\*\*This test shall also be performed whenever suction piping temperature drops below 70°F.

\*\*\*With tank heater 'A' inoperable, verify storage tank temperature is greater than 70°F every eight (8) hours.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 24  
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company (the licensee) dated April 3, 1992, as supplemented January 21 and 22, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 24 , are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Charles L. Miller*

Charles L. Miller, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: January 29, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 24

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 1-19  
3/4 1-20

Insert

3/4 1-19  
3/4 1-20

3/4.1.5 STANDBY LIQUID CONTROL SYSTEM

LIMITING CONDITION FOR OPERATION

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3.1.5 The standby liquid control system, consisting of a minimum of two pumps and corresponding flow paths, shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 5\*

ACTION:

- a. In OPERATIONAL CONDITION 1 or 2:
  1. With only one pump and corresponding explosive valve OPERABLE, restore one inoperable pump and corresponding explosive valve to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
  2. With the standby liquid control system otherwise inoperable, restore the system to OPERABLE status within 8 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- b. In OPERATIONAL CONDITION 5\*:
  1. With only one pump and corresponding explosive valve OPERABLE, restore one inoperable pump and corresponding explosive valve to OPERABLE status within 30 days or insert all insertable control rods within the next hour.
  2. With the standby liquid control system otherwise inoperable, insert all insertable control rods within 1 hour.

SURVEILLANCE REQUIREMENTS

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4.1.5 The standby liquid control system shall be demonstrated OPERABLE:

- a. At least once per 24 hours by verifying that:
  1. The temperature of the sodium pentaborate solution is within the limits of Figure 3.1.5-1.
  2. The available volume of sodium pentaborate solution is at least 4537 gallons.
  3. The temperature of the pump suction piping is greater than or equal to 70°F.

\*With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 31 days by:

1. Verifying the continuity of the explosive charge.
2. Determining by chemical analysis and calculation\* that the available weight of sodium pentaborate is greater than or equal to 5389 lbs; the concentration of sodium pentaborate in solution is less than or equal to 13.8% and within the limits of Figure 3.1.5-1 and; the following equation is satisfied:

$$\frac{C}{13\% \text{ wt.}} \times \frac{Q}{86 \text{ gpm}} \geq 1$$

where

C = Sodium pentaborate solution (% by weight)

Q = Two pump flowrate, as determined per surveillance requirement 4.1.5.c.

3. Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. Demonstrating that, when tested pursuant to Specification 4.0.5, the minimum flow requirement of 41.2 gpm per pump at a pressure of greater than or equal to 1190 psig is met.
- d. At least once per 18 months during shutdown by:
1. Initiating at least one of the standby liquid control system loops, including an explosive valve, and verifying that a flow path from the pumps to the reactor pressure vessel is available by pumping demineralized water into the reactor vessel. The replacement charge for the explosive valve shall be from the same manufactured batch as the one fired or from another batch which has been certified by having one of that batch successfully fired. All injection loops shall be tested in 3 operating cycles.
  2. \*\*Demonstrating that all suction piping is unblocked by pumping from the storage tank through each discharge line, and pumping from the test tank back to the test tank through each loop.
  3. Demonstrating that 'A' storage tank heater is OPERABLE\*\*\* by verifying expected temperature rise of the sodium pentaborate solution in the storage tank after the heater is energized.

\*This test shall also be performed anytime water or boron is added to the solution or when the solution temperature drops below 70°F.

\*\*This test shall also be performed whenever suction piping temperature drops below 70°F.

\*\*\*With tank heater 'A' inoperable, verify storage tank temperature is greater than 70°F every eight (8) hours.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 59 AND 24 TO FACILITY OPERATING

LICENSE NOS. NPF-39 AND NPF-85

PHILADELPHIA ELECTRIC COMPANY

LIMERICK GENERATING STATION, UNITS 1 AND 2

DOCKET NOS. 50-352 AND 50-353

1.0 INTRODUCTION

By letter dated April 3, 1992, as supplemented January 21 and 22, 1993, Philadelphia Electric Company (PECo or the licensee) submitted a request for changes to the Limerick Generating Station, Units 1 and 2, Technical Specifications (TS). The requested changes would revise the TS Surveillance Requirements (SRs) for the Standby Liquid Control (SLC) system to: 1) use the daily check of the SLC pump suction piping temperature to verify system operability, rather than heat tracing operability; 2) verify that the piping is not blocked by pumping from the storage tank to a test drum, rather than to the test tank; and 3) require only one SLC storage tank heater to be operable, rather than two which are currently required. The supplemental letters provide clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 DISCUSSION

The current TS SRs do not permit removing the heat tracing system or storage tank heaters from service without declaring the SLC system inoperable. When the SLC system is declared inoperable as a result of removing the heat tracing system or storage tank heaters from service, the plant must be placed in a hot shutdown condition unless the heat tracing system or storage tank heaters are returned to service within eight (8) hours. In addition, TS SRs require that SLC fluid be pumped from the storage tank to the test tank every 18 months to determine if any heat traced piping is blocked. This flow test is also required when the heat tracing system is found to be inoperable. Following this test, portions of the system must be drained and flushed with demineralized water prior to restoring the system to operation. As a result of this flushing, over 1000 gallons of waste water is generated requiring a significant amount of manpower to ensure the proper disposal of this waste water.

Current TS SR 4.1.5.a.3 requires that the SLC system be demonstrated operable at least once per 24 hours by verifying that the heat tracing circuit installed on the pump suction piping is operable by determining that the temperature of the pump suction piping is greater than or equal to

70 degrees F. In addition, the current SR 4.1.5.d.2 stipulates that at least once per 18 months all heat traced piping be verified to be unblocked by pumping from the SLC storage tank to the test tank, and then draining and flushing the piping with demineralized water. Furthermore, current SR 4.1.5.d.3 requires that at least once per 18 months, the SLC storage tank heaters (i.e., "A" and "B") be demonstrated operable by verifying the expected temperature rise of the sodium pentaborate solution in the storage tank after the heaters are energized.

The licensee proposes to change SR 4.1.5.a.3 such that the SLC system will be demonstrated operable once per 24 hours by simply verifying that the pump suction piping temperature is equal to or greater than 70 degrees F, rather than by verifying heat tracing operability. In addition, the licensee proposes to revise the 18-month SR 4.1.5.d.2 to verify that the pump suction piping is unblocked by pumping from the SLC storage tank to test drums, rather than the test tank, and proposes to change the 18-month SR 4.1.5.d.3 to demonstrate that only the "A" storage tank heater is operable, rather than both heaters.

### 3.0 EVALUATION

The design of the SLC system is mainly described in Sections 7.4.1.2 and 15.8.3.5 of the Updated Final Safety Analysis Report (UFSAR). The SLC system provides a redundant, independent, and alternate method of making the reactor core subcritical, and maintaining it subcritical, as the reactor cools. The system makes possible an orderly and safe shutdown in the event that not enough control rods can be inserted into the reactor core to accomplish normal shutdown. The normal reactivity control systems are the Control Rod Drive (CRD) system or the Alternate Rod Insertion (ARI) system. The SLC system is designed to compensate for the positive reactivity effects associated with a reactor shutdown from rated full power to a cold shutdown condition at any time during core life.

To satisfy this design objective, a solution containing boron is injected into the reactor core. The boron absorbs thermal neutrons and, when present in sufficient concentration in the reactor, will cause the reactor to become subcritical. This neutron absorber solution is an aqueous solution of sodium pentaborate and is stored in a storage tank. The saturation temperature of the sodium pentaborate solution is approximately 60 degrees F at the recommended concentration of 13.4%. The boron injection capacity of the system also meets the requirements of 10 CFR 50.62, "Requirements for the Reduction of Risk from Anticipated Transients Without Scram (ATWS) Events for Light-Water-Cooled Nuclear Power Plants."

The equipment containing the sodium pentaborate solution is installed in an area where the ambient air temperature is maintained within the range of 65 degrees F to 104 degrees F. An electrical resistance heating system containing two heaters provides a heat source which maintains the temperature of solution in the storage tank between 75 degrees F and 85 degrees F to prevent precipitation of the sodium pentaborate from the solution. Each of

the two (2) heaters is powered from a separate Class 1E power supply. However, only one (1) heater is necessary for maintaining storage tank temperature. The second heater provides a backup heating source and is used primarily during mixing operations. In addition, heat tracing with automatic temperature control provides a heating source for the pump suction piping between the storage tank and pump inlet to prevent precipitation of sodium pentaborate in the suction piping. This piping heat tracing system is nonsafety-related and does not receive electrical power from a safety-related power supply. The heat tracing is provided because the sodium pentaborate solution, at its maximum concentration of 13.8%, has the potential to precipitate out of solution if the temperature falls below approximately 61 degrees F.

Current TS SR 4.1.5.a.3 requires that the pump suction line temperature be checked once per 24 hours to verify the operability of the heat tracing. Since the ambient temperature is almost always above the low temperature setpoint of 73 degrees F for automatic heat tracing initiation, this SR does not serve its intended purpose. Instead, this SR actually serves to demonstrate SLC system operability by verifying the suction line temperature is such that no sodium pentaborate could have precipitated from solution. The proposed change to this SR will more clearly state the safety-related function without requiring operability of the nonsafety-related heat tracing. The heat tracing system design and operation will remain the same upon implementation of this proposed change.

The licensee stated that administrative controls will be instituted to require operations personnel to check heat tracing operability if a low temperature condition is detected in the SLC pump suction piping. This will help ensure that the heat tracing is available when required. Implementation of these administrative controls will provide an opportunity for plant personnel to take compensatory measures such as adding temporary heat tracing as opposed to initiating shutdown of the unit.

The current TS SR 4.1.5.d.2 requires that, at least once per 18 months, the heat traced piping be checked to verify that it is unblocked by pumping the sodium pentaborate solution from the SLC storage tank to the test tank, and then draining and flushing the associated piping with demineralized water. This action is also required when the heat tracing circuits have been found to be inoperable. Performance of this test procedure produces a considerable amount of waste solution and injects sodium pentaborate solution into the piping system, which requires flushing to prevent precipitation. The proposed change to the SR will allow this test to be performed; however, rather than pumping from the storage tank to the test tank, the solution will be pumped to a test drum through each discharge line. In addition, flow testing will be performed by pumping demineralized water from the test tank back to the test tank through each loop. Therefore, this proposed change will reduce the amount of piping requiring flushing, thereby reducing the amount of waste water generated. The licensee will establish a baseline flowrate value following a storage tank to test tank flowpath pump run and this value will be incorporated into the revised surveillance test procedure.

The current TS SR 4.1.5.d.3 requires that, at least once per 18 months, both SLC storage tank heaters (i.e., the "A" and "B" heaters) be demonstrated operable by verifying the expected temperature rise of the sodium pentaborate solution after the heaters are energized. The proposed change to this SR would only require the "A" heater to be operable to maintain SLC system operability. The "A" heater is a 10 KW heater and is used to maintain solution temperature in both automatic and manual modes of operation. The "B" heater is a 40 KW heater and is used in the manual mode only during solution mixing operations. The basis for this proposed change is that the "B" heater, by design, is not required during normal operation to maintain SLC system operability. Heater "A" will automatically initiate in the unlikely event that the solution temperature in the storage tank drops below the setpoint of 75 degrees F. The ambient temperature in the storage tank area is generally above this setpoint. Additionally, low tank temperature is alarmed in the Main Control Room (MCR) to alert Operations personnel that the "A" heater is not functioning properly. Operations personnel would then take the necessary actions, including energizing the "B" heater, in order to maintain the required solution temperature. In the event that the "A" heater is inoperable while ambient temperatures are greater than 75 degrees F, this proposed change will require that a tank temperature check be performed every eight (8) hours. This proposed change will permit removal of the heater from service for maintenance purposes, but will still ensure that the solution temperature is maintained within the required limits to ensure SLC system operability.

We have concluded that the proposed change to the SR for determining pump suction line temperature is a more direct means of verifying SLC system operability and is acceptable. The proposed change to the SR concerning the flow blockage test will utilize a similar flowpath. The test drum will serve an identical purpose as the present test tank and will still identify any blockage which adversely impacts pump operation. The proposed change is acceptable. The proposed change to the SR to permit the "B" heater to temporarily be removed from service for maintenance without declaring the SLC inoperable is acceptable, since only the "A" heater is necessary to ensure SLC system operability.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 20515). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Clark  
F. Rinaldi

Date: January 29, 1993