

January 18, 1989

Docket No.: 50-352

Mr. George A. Hunger, Jr.
Director-Licensing
Philadelphia Electric Company
Correspondence Control Desk
P. O. Box 7520
Philadelphia, Pennsylvania 19101

Dear Mr. Hunger:

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MO'Brien	Tech Branch	

SUBJECT: MODIFICATION AND TESTING OF CONTAINMENT PENETRATIONS
(TAC NO. 71129)

RE: LIMERICK GENERATING STATION, UNIT 1

The Commission has issued the enclosed Amendment No. 15 to Facility Operating License No. NPF-39 for the Limerick Generating Station, Unit 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 1, 1988.

This amendment changes the TSs to reflect proposed modifications to certain containment penetrations to permit forward leak testing of associated isolation valves and testing of valve packing leakage.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by
Richard J. Clark

Richard J. Clark, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 15 to License No. NPF-39
2. Safety Evaluation

cc w/enclosures:
See next page

[LIMERICK]

MO'Brien
1/17/89

PDI-2/PM
RClark:mr
12/22/88
01/17/89

OGC
S H Lewis
1/11/88

PDI-2/D
WButler
1/19/88

WB

DFol
CP
11



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

January 18, 1989

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Sincerely,

A handwritten signature in black ink, appearing to read "Richard J. Clark".

Richard J. Clark, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 15 to License No. NPF-39
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. George A. Hunger, Jr.
Philadelphia Electric Company

Limerick Generating Station
Units 1 & 2

cc:

Troy B. Conner, Jr., Esquire
Conner and Wetterhahn
1747 Pennsylvania Ave., N.W.
Washington, D. C. 20006

Mr. Robert Gramm
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P. O. Box 596
Pottstown, Pennsylvania 19464

Mr. Rod Krich S7-1
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. Ted Ullrich
Manager - Unit 2 Startup
Limerick Generating Station
P. O. Box A
Sanatoga, Pennsylvania 19464

Mr. David Honan N2-1
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. John Doering
Superintendent-Operations
Limerick Generating Station
P. O. Box A
Sanatoga, Pennsylvania 19464

Mr. Grahm M. Leitch, Vice President
Limerick Generating Station
Post Office Box A
Sanatoga, Pennsylvania 19464

Thomas Gerusky, Director
Bureau of Radiation Protection
PA Dept. of Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Mr. James Lirville
U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Single Point of Contact
P. O. Box 11880
Harrisburg, Pennsylvania 17108

Mr. Thomas Kenny
Senior Resident Inspector
US Nuclear Regulatory Commission
P. O. Box 596
Pottstown, Pennsylvania 19464

Mr. Philip J. Duca
Superintendent-Technical
Limerick Generating Station
P. O. Box A
Sanatoga, Pennsylvania 19464

Mr. Joseph W. Gallagher
Vice President, Nuclear Services
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Mr. John S. Kemper
Senior Vice President-Nuclear
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that
 - A. The application for amendment by Philadelphia Electric Company (the licensee) dated November 1, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 15, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8901300234 890118
PDR ADOCK 05000352
P PNU

3. This license amendment is effective 60 days after date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 18, 1989

RDE/DA
MOC/ten
1/17/89

PDI-2/PM
RCIark:m
12/22/88

OGC
S H LewN
1/11/88

PDI-2/D
WButler
1/17/88

WB

3. This license amendment is effective 60 days after date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 18, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided to maintain document completeness.*

Remove

3/4 6-21
3/4 6-22

3/4 6-23
3/4 6-24

3/4 6-25
3/4 6-26

3/4 6-29
3/4 6-30

Insert

3/4 6-21*
3/4 6-22

3/4 6-23*
3/4 6-24

3/4 6-25*
3/4 6-26

3/4 6-29
3/4 6-30*

LIMERICK - UNIT 1

3/4 6-21

Amendment No. 2, 13, 14

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
016A	CORE SPRAY INJECTION	HV52-1F006A(CK) HV52-1F039A	HV52-1F005	NA 7 18		9,22 9,22	52
016B	CORE SPRAY INJECTION	HV52-1F006B(CK) HV52-1F039B	HV52-108(CK)	NA 7 NA		9,22 9,22	52
021	SERVICE AIR TO DRYWELL	15-1140	15-1139	NA NA			15
022	DRYWELL PRESSURE INSTRUMENTATION		HV42-147C	45		10	42
023	RECW SUPPLY TO RECIRC PUMPS	HV13-106*	HV13-108* HV13-109*	40 30 NA	C,H C,H	11, 11, 11,13	13
024	RECW RETURN FROM RECIRC PUMPS	HV13-107*	HV13-111* HV13-110*	40 30 NA	C,H C,H	11 11, 11,13	13

LIMERICK - UNIT 1

3/4 6-22

Amendment No. 8, 13, 15

TABLE 3.6.3-1 (Continued)
PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID	
025	DRYWELL PURGE SUPPLY	HV57-121(X-201A) HV57-123		5**	B,H,S,U,W,R,T	3,11,14	57	
				5**	B,H,S,U,W,R,T	3,11,14		
				6**	B,H,S,U,W,R,T	11		
			HV57-109 (X-201A)					
			HV57-131 (X-201A)	5**	B,H,S,U,W,R,T	11		
026	DRYWELL PURGE EXHAUST	HV57-114 HV57-111 SV57-139		6**	B,H,S,U,W,R,T	11,33	57	
				5**	B,H,S,U,R,T	11		
				5		10		
027A	HYDROGEN RECOMBINER "B" INLET	HV57-163		9	B,H,R,S	3,11,14	59	
				90	B,H,R,S	11		
			FV-C-D0-101B					
027A	CONTAINMENT INSTRUMENT GAS SUPPLY TO ADS VALVES H,M,&S	59-1128(CK)		90	B,H,R,S	11	59	
				NA				
028A-1	RECIRC LOOP SAMPLE	HV43-1F019		45	M		43	
				10	B,D			
028A-2	DRYWELL H2/O2 SAMPLE	SV57-132		10	B,D		57	
				5	B,H,R,S	11		
028A-3	DRYWELL H2/O2 SAMPLE	SV57-134		5	B,H,R,S	11	57	
				5	B,H,R,S	11		

LIMERICK - UNIT 1

3/4 6-23

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID	
028B	DRYWELL H2/O2 SAMPLE	SV57-133		5	B,H,R,S	11	57	
				SV57-143	5	B,H,R,S	11	
				SV57-195	5	B,H,R,S	11	
030B-1	DRYWELL PRESSURE INSTRUMENTATION		HV42-147A	45		10	42	
035A	TIP PURGE	59-1056(CK) (DOUBLE "O" RING)		NA			59	
				HV59-131	7	B,H,S	16	
035C-G	TIP DRIVES	XV59-141A-E (DOUBLE "O" RING)		NA	B,H	11,16,21	59	
				XV59-140A-E	NA		11,16	
037A-D	CRD INSERT LINES	BALL CHECK		NA		12	47	
				HCU	NA		12	
038A-D	CRD WITHDRAW LINES SDV VENTS & DRAINS			NA		12	47	
				HCU	NA		12	
				XV47-1F010	25		30	
				XV47-1F180	30		30	
				XV47-1F011	25		30	
	XV47-1F181	30		30				
039A(B)	DRYWELL SPRAY	HV51-1F021A(B)		160		4,11	51	
				HV51-1F016A(B)	160		11	
040E	DRYWELL PRESSURE INSTRUMENTATION		HV42-147D	45		10	42	
040F-2	CONTAINMENT INSTRUMENT GAS -SUCTION	HV59-101		45	C,H,S	5	59	
				HV59-102	7	C,H,S		

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
040G-1	ILRT DATA ACQUISITION	60-1057		NA		11	60
			60-1058	NA	11		
040G-2	ILRT DATA ACQUISITION	60-1071		NA		11	60
			60-1070	NA	11		
040H-1	CONTAINMENT INSTRUMENT GAS SUPPLY - HEADER 'A'	59-1005A(CK)	HV59-129A	NA 7	C,H,S		59
042	STANDBY LIQUID CONTROL	48-1F007(CK) (X-116)	HV48-1F006A	NA 60		29	48
043B	MAIN STEAM SAMPLE	HV41-1F084		10	B,D		41
			HV41-1F085	10	B,D		
044	RWCU ALTERNATE RETURN	41-1017		NA		5,31	41
			41-1016(X-9A, X-9B)	NA			
			PSV41-112	NA			
045A(B,C,D)	LPCI INJECTION 'A' (B,C,D)	HV51-1F041A*(B,C*, D*)(CK) HV51-142A*(B,C*, D*)		NA		9,22	51
				7	9,22		
			HV51-1F017A* (B,C*,D*)	38			
050A-1	DRYWELL PRESSURE INSTRUMENTATION		HV42-147B	45		10	42
053	DRYWELL CHILLED WATER SUPPLY - LOOP 'A'	HV87-128*		60	C,H	11	87
			HV87-120A*	60	C,H	11	
			HV87-125A*	60	C,H	11	

LIMERICK - UNIT 1

3/4 6-24

Amendment No. 2, 13, 15

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
054	DRYWELL CHILLED WATER RETURN - LOOP 'A'	HV87-129*		60	C,H	11	87
			HV87-121A*	60	C,H	11	
			HV87-124A*	60	C,H	11	
055	DRYWELL CHILLED WATER SUPPLY - LOOP 'B'	HV87-122*		60	C,H	11	87
			HV87-120B*	60	C,H	11	
			HV87-125B*	60	C,H	11	
056	DRYWELL CHILLED WATER RETURN - LOOP 'B'	HV87-123*		60	C,H	11	87
			HV87-121B*	60	C,H	11	
			HV87-124B*	60	C,H	11	
061-1	RECIRC PUMP 'A' SEAL PURGE	43-1004A(CK)	(XV43-103A - SEE PART B, THIS TABLE)	NA NA		15 1	43
061-2	RECIRC PUMP 'B' SEAL PURGE	43-1004B*(CK)	(XV43-103B - SEE PART B, THIS TABLE)	NA NA		15 1	43
062	DRYWELL H2/O2 SAMPLE RETURN, N2 MAKE-UP	SV57-150(X-220A)		5	B,H,R,S	11	57
			SV57-159 (X-220A)	5	B,H,R,S	11	
			HV57-116 (X-220A)	30**	B,H,R,S	11	
			SV57-190 (X-220A)	5	B,H,R,S	11	

LIMERICK - UNIT 1

3/4 6-25

Amendment No. 2.13

TABLE 3.6.3-1 (Continued)
PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
			SV57-191 (X-220A)	5	B,H,R,S	11	
116	STANDBY LIQUID CONTROL	48-1F007(CK) (X-42)	HV48-1F006B	NA 60			48
117B-1	DRYWELL RADIATION MONITORING SUPPLY	SV26-190A	SV26-190B	5 5	B,H,R,S B,H,R,S	11 11	26
117B-2	DRYWELL RADIATION MONITORING RETURN	SV26-190C	SV26-190D	5 5	B,H,R,S B,H,R,S	11 11	26
201A	SUPPRESSION POOL PURGE SUPPLY	HV57-124 HV57-131(X-25)	HV57-109(X-25) HV57-147 HV57-121(X-25)	5** 5** 6** 6** 5**	B,H,S,U,W,R,T B,H,S,U,W,R,T B,H,S,U,W,R,T B,H,S,U,W,R,T B,H,S,U,W,R,T	3,11,14 3,11,14 11 11 11	57
	HYDROGEN RECOMBINER "B" EXHAUST	HV57-164	HV57-169	9 9	B,H,R,S B,H,R,S	3,11,14 11	
202	SUPPRESSION POOL PURGE EXHAUST	HV57-104 HV57-105	HV57-112 HV57-118 SV57-185	5** 15** 6** 5** 5	B,H,S,U,W,R,T B,H,S,U,R,T B,H,S,U,W,R,T B,H,S,U,R,T B,H,R,S	3,11,14,33 11 11, 33 11 11	57
	HYDROGEN RECOMBINER "A" EXHAUST	HV57-162	HV57-166	9 9	B,H,R,S B,H,R,S	3,11,14 11	
203A(B,C,D)	RHR PUMP SUCTION		HV51-1F004A(B,C,D) PSV51-1F030A (B,C,D)	240 NA		4,22, 19,29	51
						22	

LIMERICK - UNIT 1

3/4 6-26

Amendment No. 8, 13, 15

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(36)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
225	RHR VACUUM RELIEF SUCTION	HV51-130	HV51-131	60 60	B,H B,H	4,11 11	51
226A	RHR MINIMUM RECIRC		HV51-105A	40		4,22,29	57
226B	RHR MINIMUM RECIRC		HV51-105B	40		4,22,29	51
227	ILRT DATA ACQUISITION SYSTEM	60-1073	60-1074	NA NA			60
228D	HPCI VACUUM RELIEF	HV55-1F095	HV55-1F093	40 40	H,LA H,LA	4,11,24 11,24	55
230B	INSTRUMENTATION - DRYWELL SUMP LEVEL	--	HV61-102 HV61-112 HV61-132	45 45 45		1,23,29 23,29 23,29	61
231A	DRYWELL FLOOR DRAIN SUMP DISCHARGE	HV61-110	HV61-111	30 30	B,H B,H	11,22 11,22	61
231B	DRYWELL EQUIPMENT DRAIN TANK DISCHARGE	HV61-130	HV61-131	30 30	B,H B,H	11,22 11,22	61
235	CS PUMP MINIMUM RECIRC		HV52-1F031A	45		5,22,29	52
236	HPCI PUMP MINIMUM RECIRC		HV55-1F012	15	LFHP	5,22	55

LIMERICK - UNIT 1

3/4 6-29

Amendment No. 15

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

LIMERICK - UNIT 1

3/4 6-30

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
237-1	SUPPRESSION POOL CLEANUP PUMP, SUCTION	HV52-127	PSV52-127 HV52-128	60 NA 60	B,H B,H	4,11,22 11,22 11,22	52
237-2	SUPPRESSION POOL LEVEL INSTRUMENTATION		HV52-139 SV52-139	45 6		10 10	52
238	RHR RELIEF VALVE DISCHARGE		HV-C-51-1F104B PSV51-106B PSV51-1F055B PSV51-101B	18 NA NA NA	C,G	19 19 19	51
239	RHR RELIEF VALVE DISCHARGE		HV-C-51-1F103A PSV51-106A PSV51-1F055A PSV51-101A	18 NA NA NA	C,G	19 19 19	51
240	RHR RELIEF VALVE DISCHARGE		PSV51-1F097	NA		19	51
241	RCIC VACUUM RELIEF	HV49-1F084	HV49-1F080	40 40	H,KA H,KA	4,11,24 11,24	49



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. NPF-39

PHILADELPHIA ELECTRIC COMPANY

LIMERICK GENERATING STATION, UNIT 1

DOCKET NO. 50-352

1.0 INTRODUCTION

By letter dated November 1, 1988, Philadelphia Electric Company (the licensee) requested an amendment to Facility Operating License No. NPF-39 for the Limerick Generating Station, Unit 1. The proposed amendment would change the Technical Specifications (TSs) to reflect proposed modifications to certain containment penetrations to permit forward leak testing of associated isolation valves and testing of valve packing leakage.

2.0 DISCUSSION

The design basis for the Primary Containment Isolation System allows the normal and emergency passage of fluids through the containment boundary. Isolation valves are installed at each penetration to prevent (or limit) the escape of fission products which would accumulate inside the containment during an accident. At least two barriers are required at each containment penetration. Generally, two valves are located between the containment atmosphere (or the reactor coolant system) and the outside atmosphere, to act as barriers, so that failure of any single valve would not prevent isolation of the containment penetration during an accident.

Various types of valves (i.e. gate, globe, etc.) are installed at Limerick Station for use as primary containment isolation valves. All of these valves require periodic testing for leak tightness. The methods used for periodic leak tightness testing are designated in 10 CFR Part 50, Appendix J as type A tests, type B tests and type C tests. The subject application concerns only type C testing of globe valves.

Type C testing is accomplished on individual valves and is sometimes accomplished in the "Reverse" direction, i.e., the test pressure is not from the containment direction, but is from outside the containment, directed toward the valve in the containment direction. Testing is accomplished on some valves in a manner, (or from a direction), which would not include the leakage through the valve packing, because the packing is opposite from the valve globe which effectively blocks the test pressure from the packing. In those cases where the valve is installed with the packing on the containment side of the valve, i.e., exposing the stem packing to possible containment pressures and creating a potential path for containment fluid leakage during an accident, the packing must be leak tested during type C testing.

Table 3.6.3-1 Part A of the Limerick TSs lists the primary containment isolation valves along with their function, the inboard and outboard isolation barriers, the maximum isolation time, if applicable, the isolation signal, if applicable, etc. If a particular globe valve in the Table is normally tested in the reverse direction, the reader is referred to "Note 5." "Note 5" states, "Inboard globe valve tested in the reverse direction." During the upcoming second refueling outage, which is scheduled to start January 13, 1989, Philadelphia Electric (PECo or licensee) proposes to modify certain containment penetrations whose inboard containment isolation valves have their stem packing exposed to containment atmosphere and proposes to revise testing methods in order that the stem packing be included during leak testing. The penetrations will be modified to allow periodic testing to include leakage through the valve(s) stem packing in accordance with 10 CFR Part 50, Appendix J. Modifications to these penetrations would allow "forward" testing to test valve packing leakage along with the leakage through the valve seat. For three valves, the licensee proposes to install a new blocking valve and test tap between containment and the isolation valve so that gas pressure can be applied to these valves from the containment "forward" direction during type C testing. These block valves remain open except when they are being used during type C testing. The new blocking valves will be oriented such that their packings are included in the type C test boundary. For two other valves, a revised method of testing will be used which will allow "forward" testing, including leak testing of the valve's stem packing. For these two valves, a temporary test plug and test rig will be used for type C testing from inside the containment.

Once the above modifications are completed, reference to "Note 5" in Table 3.6.3-1 for these five valves will no longer be appropriate. The licensee's proposed change to the TSs is to delete reference to "Note 5" for these five valves in Table 3.6.3-1. No other TS changes are being proposed.

3.0 EVALUATION

Test pressure used for type C testing of valves is normally required to be the same direction as containment pressure during an accident ("forward" direction) unless it can be demonstrated that the valve has the same, or more conservative, leak tightness when tested from a different direction. The proposed modifications and revised testing methods will permit "forward" testing of the valves and better testing of valve stem packing, which is preferable to the present arrangement. The licensee's analysis shows that the addition of the blocking valves and test taps have been designed and evaluated to ensure: 1) that the modifications will not affect the ability of the isolation system to limit or prevent the escape of radioactivity from containment following postulated accidents, 2) that automatic and remote manual isolation from the control room will remain the same after the proposed modifications, 3) that the isolation valves will continue to conform to General Design criteria 54, 55, 56, and 57 to the same degree as the original design,

4) that the stress and pipe support calculations have been reviewed to ensure that changes and additions satisfy the existing seismic design criteria, 5) that valve closure times are not affected by the proposed modifications, 6) that the proposed modification has been evaluated for protection against loss of functional capability from missiles and impact accidents, 7) that leak detection provisions would not be affected by the proposed modifications, 8) that no changes are proposed in the design of the electrical and mechanical redundancy and separation provisions for the isolation valves, and 9) that the design for the isolation valves will continue to maintain the containment isolation valves closed after resetting the isolation signal.

The staff has reviewed the design of the proposed modifications and has determined that they meet all appropriate requirements and are both desirable and acceptable. The modifications and revised test methods are hereby approved.

As noted above, the only proposed change to the TSs is to delete reference to "Note 5" in Table 3.6.3-1 Part A for the five penetrations. Once the modifications and revised test methods are implemented, reference to note 5 would be incorrect. The staff has determined that the proposed changes to the TSs are acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (53 FR 50334) on December 14, 1988 and consulted with the State of Pennsylvania. No public comments were received and the State of Pennsylvania did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and the security nor to the health and safety of the public.

Principal Contributor: Dick Clark

Dated: January 18, 1989