



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
SAM NUNN ATLANTA FEDERAL CENTER  
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May 29, 2001

Mr. Dale E. Young,  
Vice President,  
Crystal River Nuclear Plant (NA1B)  
ATTN: Supervisor, Licensing &  
Regulatory Programs  
15760 West Power Line Street  
Crystal River, FL 34428-6708

SUBJECT: ANNUAL ASSESSMENT LETTER - CRYSTAL RIVER 3 NUCLEAR PLANT-  
(NRC INSPECTION REPORT 50-302/01-01)

Dear Mr. Young:

On May 1, 2001, the NRC staff completed its end-of-cycle plant performance assessment of the Crystal River 3 Nuclear Plant. The end-of-cycle review for Crystal River 3 Nuclear Plant involved the participation of all technical divisions in evaluating performance indicators (PIs) and inspection results for the period from April 2, 2000, to March 31, 2001. The purpose of this letter is to inform you of our assessment of your safety performance during this period and our plans for future inspections at your facility so that you will have an opportunity to prepare for these inspections and to inform us of any planned inspections which may conflict with your plant activities.

Overall, Crystal River 3 Nuclear Plant operated in a manner that preserved public health and safety and fully met all cornerstone objectives. Plant performance for the most recent quarter was within the Licensee Response Column of the NRC's Action Matrix, based on all inspection findings being classified as having very low safety significance (Green) and all PIs indicating performance at a level requiring no additional NRC oversight (Green). Therefore, we plan to conduct only baseline inspections at your facility through May 31, 2002.

While plant performance for the most recent quarter is within the Licensee Response Column of the Action Matrix, one performance indicator entered the Regulatory Response Column in August, 2000. Leakage through a pressurizer safety valve and from a decay heat removal system valve exceeded 50 percent of the Technical Specifications limits until September 9, 2000, when the plant was shutdown for repairs. The safety valve was replaced and leak sealant repairs were performed on the decay heat valve. The reactor coolant system leakage performance indicator returned to the green band in the fourth quarter of 2000. We reviewed your activities associated with the increased leakage in a November, 2000, supplemental inspection. We concluded that your staff's root cause analysis for the leakage was acceptable and that you had taken or planned appropriate corrective actions.

The enclosed inspection plan details the scheduled inspections that will occur through May 31, 2002. The inspection plan is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of

inspector arrival onsite. Routine resident inspections are not listed due to their ongoing and continuous nature.

For your information, the NRC is in the process of aligning the inspection and assessment cycle with the calendar year (CY). In order to transition to a CY cycle (January 1- December 31), the current inspection and assessment cycle will consist of only three quarters (i.e. the second, third and fourth calendar quarters of CY 2001). As a result, for this transition period only, the number of inspection activities have been modified accordingly and a mid-cycle review letter and accompanying inspection plan will not be issued.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html> (the Public Electronic Reading Room).

If circumstances arise which cause us to change this inspection plan, we will contact you to discuss the change as soon as possible. Please contact me at (404) 562-4540 with any questions you may have regarding this letter or the inspection plan.

Sincerely,

/RA/

Leonard D. Wert, Chief  
Projects Branch 3  
Division of Reactor Projects,

Docket No. 50-302  
Licensee No. DPR-72

Enclosure: Inspection Activity Plan

cc w/encl:

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