

April 5, 1994

Docket Nos. 50-277  
and 50-278

Mr. George A. Hunger, Jr.  
Director-Licensing, MC 52A-5  
PECO Energy Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, Pennsylvania 19087-0195

Dear Mr. Hunger:

SUBJECT: ELIMINATING UNNECESSARY DIESEL GENERATOR TESTING SURVEILLANCE  
REQUIREMENTS, PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3,  
(TAC NOS. M88295 AND M88296)

The Commission has issued the enclosed Amendments Nos. 187 and 192 to  
Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic  
Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the  
Technical Specifications in response to your application dated November 17,  
1993.

These amendments revise the surveillance requirements to eliminate unnecessary  
diesel generator testing when a diesel generator or an offsite power source  
becomes inoperable. This change reduces the stresses on the diesel generators  
caused by unnecessary testing. Please inform the staff, in writing, when  
these amendments have been implemented.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be  
included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

/s/

Stephen Dembek, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

9404150190 940405  
PDR ADOCK 05000277  
P PDR

Enclosures:

1. Amendment No. 187 to DPR-44
2. Amendment No. 192 to DPR-56
3. Safety Evaluation

cc w/enclosures:  
See next page

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NRC & Local PDRs	SDembek	CBerlinger	
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JCalvo	GHill(4), P1-22	OC/LFDCB	
CMiller	EWenzinger, RGN-I	CAnderson, RGN-I	

OFC	:PDI-2/LA	:PDI-2/PM	:EELB/BC	:OGC	:PDI-2/D
NAME	:MO'Brien	:SDembek:rb:	CBerlinger:	:CMiller	:
DATE	:3/9/94	:3/9/94	:3/14/94	:3/16/94	:4/5/94

QFD 111

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 5, 1994

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and 50-278

Mr. George A. Hunger, Jr.  
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The Commission has issued the enclosed Amendments Nos. 187 and 192 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications in response to your application dated November 17, 1993.

These amendments revise the surveillance requirements to eliminate unnecessary diesel generator testing when a diesel generator or an offsite power source becomes inoperable. This change reduces the stresses on the diesel generators caused by unnecessary testing. Please inform the staff, in writing, when these amendments have been implemented.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Stephen Dembek".

Stephen Dembek, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 187 to DPR-44
2. Amendment No. 192 to DPR-56
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. George A. Hunger, Jr.  
PECO Energy Company

Peach Bottom Atomic Power Station,  
Units 2 and 3

cc:

J. W. Durham, Sr., Esquire  
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Harrisburg, Pennsylvania 17105-8469

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ATTN: Mr. G. R. Rainey, Vice President  
Peach Bottom Atomic Power Station  
Route 1, Box 208  
Delta, Pennsylvania 17314

Board of Supervisors  
Peach Bottom Township  
R. D. #1  
Delta, Pennsylvania 17314

PECO Energy Company  
ATTN: Regulatory Engineer, A1-2S  
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Route 1, Box 208  
Delta, Pennsylvania 17314

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Wayne, Pennsylvania 19087

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 187  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated November 17, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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PDR ADDCK 05000277  
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 187, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Charles L. Miller*

Charles L. Miller, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 5, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 187

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

219  
220  
220a

Insert

219  
220  
220a

LIMITING CONDITIONS FOR OPERATION**3.9.B Operation with Inoperable Equipment**

Whenever the reactor is in Run Mode or Startup Mode with the reactor not in a Cold Condition, the availability of electric power shall be as specified in 3.9.A, except as follows:

1. With one offsite circuit required by Specification 3.9.A.1 inoperable, restore at least two offsite circuits to OPERABLE status within 7 days or be in least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
  
2. With two independent off-site circuits required by Specification 3.9.A.1 inoperable, continued operation is permissible, provided the four diesel generators and associated emergency busses are operable, all core and containment cooling systems are operable and reactor power level is reduced to 25% of the design.

SURVEILLANCE REQUIREMENTS**4.9.B. Operation with Inoperable Equipment**

1. When it is determined that one offsite circuit required by Specification 3.9.A.1 is inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter.

2. None

## PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

## 3.9.B (Continued)

3. With one diesel generator inoperable restore the inoperable diesel generator and associated emergency bus to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

3. When it is determined that one diesel generator is inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter. Verify within 2 hours that required systems, subsystems, trains, components, and devices that depend on the remaining diesel generators as a source of emergency power are OPERABLE.

If the diesel generator became inoperable for any reason other than preplanned preventative maintenance, or testing, either determine within 24 hours that remaining operable diesel generators are not inoperable due to common cause failure or demonstrate the OPERABILITY of the remaining operable diesel generators by performing Surveillance Requirement 4.9.A.1.2.a.3 for one diesel at a time, within 24 hours\* and at least once per 72 hours thereafter.

\*This test is required to be completed regardless of when the inoperable diesel generator is restored to OPERABILITY for failures that are potentially generic to the remaining diesel generators and for which appropriate alternative testing cannot be designed.



## PBAPS

## LIMITING CONDITIONS FOR OPERATION

## SURVEILLANCE REQUIREMENTS

## 3.9.B (Continued)

4. With one diesel generator and one offsite circuit required by Specification 3.9.A.1 inoperable, restore at least two offsite circuits or four diesel generators to OPERABLE status within 72 hours from the time of initial loss or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

5. From and after the date that one of the 125 volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding three days within electrical safety considerations, provided repair work is initiated in the most expeditious manner to return the failed component to an operable state, and Specifications 3.5.F and 3.9.B.3 are satisfied.

4. When it is determined that one diesel generator and one offsite circuit required by Specification 3.9.A.1 are inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter. Verify within 2 hours, that required systems, subsystems, trains, components, and devices that depend on the remaining diesel generators as a source of emergency power are OPERABLE.

If the diesel generator became inoperable for any reason other than preplanned preventative maintenance, or testing, either determine within 24 hours that remaining operable diesel generators are not inoperable due to common cause failure or demonstrate the OPERABILITY of the remaining diesel generators by performing Surveillance Requirement 4.9.A.1.2.a.3 for one diesel generator at a time, within 8 hours\*.

5. None

\*This test is required to be completed regardless of when the inoperable diesel generator is restored to OPERABILITY for failures that are potentially generic to the remaining diesel generators and for which appropriate alternative testing cannot be designed.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192  
License No. DPR-56

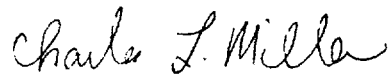
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated November 17, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 192, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles L. Miller, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 5, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 192

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

219  
220  
220a

Insert

219  
220  
220a

LIMITING CONDITIONS FOR OPERATION**3.9.B Operation with Inoperable Equipment**

Whenever the reactor is in Run Mode or Startup Mode with the reactor not in a Cold Condition, the availability of electric power shall be as specified in 3.9.A, except as follows:

1. With one offsite circuit required by Specification 3.9.A.1 inoperable, restore at least two offsite circuits to OPERABLE status within 7 days or be in least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
  
2. With two independent off-site circuits required by Specification 3.9.A.1 inoperable, continued operation is permissible, provided the four diesel generators and associated emergency busses are operable, all core and containment cooling systems are operable and reactor power level is reduced to 25% of the design.

SURVEILLANCE REQUIREMENTS**4.9.B. Operation with Inoperable Equipment**

1. When it is determined that one offsite circuit required by Specification 3.9.A.1 is inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter.
  
2. None

## PBAPS

## LIMITING CONDITIONS FOR OPERATION

## SURVEILLANCE REQUIREMENTS

## 3.9.B (Continued)

3. With one diesel generator inoperable restore the inoperable diesel generator and associated emergency bus to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

3. When it is determined that one diesel generator is inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter. Verify within 2 hours that required systems, subsystems, trains, components, and devices that depend on the remaining diesel generators as a source of emergency power are OPERABLE.

If the diesel generator became inoperable for any reason other than preplanned preventative maintenance, or testing, either determine within 24 hours that remaining operable diesel generators are not inoperable due to common cause failure or demonstrate the OPERABILITY of the remaining operable diesel generators by performing Surveillance Requirement 4.9.A.1.2.a.3 for one diesel at a time, within 24 hours\* and at least once per 72 hours thereafter.

\*This test is required to be completed regardless of when the inoperable diesel generator is restored to OPERABILITY for failures that are potentially generic to the remaining diesel generators and for which appropriate alternative testing cannot be designed.

## PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

## 3.9.B (Continued)

4. With one diesel generator and one offsite circuit required by Specification 3.9.A.1 inoperable, restore at least two offsite circuits or four diesel generators to OPERABLE status within 72 hours from the time of initial loss or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
5. From and after the date that one of the 125 volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding three days within electrical safety considerations, provided repair work is initiated in the most expeditious manner to return the failed component to an operable state, and Specifications 3.5.F and 3.9.B.3 are satisfied.

4. When it is determined that one diesel generator and one offsite circuit required by Specification 3.9.A.1 are inoperable, verify the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.9.A.1.1.a within 1 hour and once per 8 hours thereafter. Verify within 2 hours, that required systems, subsystems, trains, components, and devices that depend on the remaining diesel generators as a source of emergency power are OPERABLE.

If the diesel generator became inoperable for any reason other than preplanned preventative maintenance, or testing, either determine within 24 hours that remaining operable diesel generators are not inoperable due to common cause failure or demonstrate the OPERABILITY of the remaining diesel generators by performing Surveillance Requirement 4.9.A.1.2.a.3 for one diesel generator at a time, within 8 hours\*.

5. None

\*This test is required to be completed regardless of when the inoperable diesel generator is restored to OPERABILITY for failures that are potentially generic to the remaining diesel generators and for which appropriate alternative testing cannot be designed.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 187 AND 192 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated November 17, 1993, the Philadelphia Electric Company (the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station (PBAPS), Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would revise the surveillance requirements (SR) to eliminate unnecessary diesel generator testing when a diesel generator or an offsite power source becomes inoperable.

2.0 EVALUATION

The proposed changes to SR for TS Section 4.9.B, "Operation with Inoperable Equipment," are:

1. Delete the requirement in TS Section 4.9.B.1 to demonstrate operability of the diesel generators when an offsite power source is determined to be inoperable.
2. Revise the requirement in TS Section 4.9.B.3 and Section 4.9.B.4 to require that station personnel either determine that the remaining operable diesel generators are not also inoperable due to a common cause failure or perform SR 4.9.A.1.2.A.3 when one diesel generator is determined to be inoperable.

The current TS SR 4.9.B.1 requires that the operable emergency diesel generators (EDGs) be started to demonstrate operability when one offsite power source becomes inoperable. The intent of this additional testing is to provide added assurance that the EDGs are capable of supplying emergency power when one of the offsite power sources becomes inoperable. The current SR 4.9.B.3 requires that the operable EDGs be started to demonstrate operability when one EDG becomes inoperable due to any reason other than preplanned preventive maintenance, or testing. The current SR 4.9.B.4 requires that the



operable EDGs be started to demonstrate operability when one EDG and one offsite power source become inoperable. The additional testing in these two SR sections is to determine if a common cause failure exists.

Generic Letter (GL) 84-15, "Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability," issued in July 1984, contains the staff's recommendation to reduce the number of diesel generator "fast starts." The staff determined that these "fast starts" resulted in premature diesel engine degradation. Other diesel generator start tests were also targeted for reduction on the basis that excessive testing results in degradation of diesel engines. The requirement for the EDG surveillance testing in the current TS SR 4.9.B.1 can result in unnecessary testing of otherwise operable EDGs when an offsite power source is declared inoperable. The staff agrees that the loss of one of the offsite power sources does not imply any loss of EDG reliability or common cause failure.

The staff considers that the normal TS surveillance testing is sufficient to demonstrate reliability and assurance that the EDGs are capable of performing their intended function. Therefore, the additional testing included in the current TS SR Section 4.9.B.1 is considered excessive. Additionally, allowing station personnel to verify that the cause of one EDG being inoperable (SR Sections 4.9.B.3 and 4.9.B.4) does not affect the operability of the remaining operable EDGs will further reduce unnecessary testing of the operable EDGs. Also, the proposed change is consistent with the staff's improved standard TS ("Standard Technical Specifications, General Electric Plants, BWR/4," NUREG-1433, September 1992). Therefore, the staff finds the licensee's proposal, which eliminates unnecessary diesel generator testing when a diesel generator or an offsite power source becomes inoperable, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 628). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Liu

Date: April 5, 1994