Duke Power

Oconee Nuclear Site 7800 Rochester Highway Seneca, SC 29672 (864) 885-3107 OFFICE (864) 885-3564 FAX

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W. R. McCollum, Jr. Vice President

May 24, 2001

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, DC 20555-0001

Subject: Duke Energy Corporation Oconee Nuclear Station, Unit 2 Docket No. 50-270 End-of-Cycle-18 Steam Generator Tube Inspection -Report on Indication of Circumferential Defects

The Oconee Facility Operating License, DPR-47, License Condition No. 5, requires that following each inservice inspection of Steam Generator (SG) tubes, the Nuclear Regulatory Commission be provided information concerning any circumferential crack indications identified during the inspection. The information specified by the License Condition is provided below:

1. Describe indications of circumferential cracking in the secondary side roll (lower roll in the upper tubesheet or upper roll in the lower tubesheet) if rerolled.

One tube with a circumferential indication was identified in the upper transition of a rerolled tube. The indication had an arc length of 50 degrees and a maximum depth of 55% through wall (TW). The tube was plugged.

2. Describe indications of circumferential cracking in the original roll or heat affected zone adjacent to the tube-to-tubesheet seal weld if no reroll is present.

Thirty-two (32) tubes had circumferential indications which had not been previously rerolled. All were located in the clad area near the heat-affected zone of the tube-totubesheet weld. All 32 tubes were rerolled.

None of the above 32 tubes were predicted to fail due to the estimated LBLOCA axial load.

USNRC Document Control Desk May 24, 2001

3. Provide the best-estimate total leakage that would result from an analysis of the limiting Large Break Loss of Coolant Accident (LBLOCA) based on circumferential cracking in the original tube-to-tubesheet rolls, tube-to-tubesheet rerolls, and heat affected zones of seal welds as found during each inspection.

Duke's best estimate of the above-described leakage is 1.9 gpm based on circumferential indications in the rerolled tubes. This estimate leakage would maintain offsite dose well below the limits of 10 CFR 100.

Please contact Robert Douglas at 864-885-3073 with any questions regarding this submittal.

Very Truly Yours,

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W. R. McCollum, Jr. Site Vice President, Oconee Nuclear Station

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xc: L.A. Reyes Administrator, Region II

M. C. Shannon

NRC Senior Resident Inspector Oconee Nuclear Station

D. E. LaBarge NRR, Senior Project Manager

V.R. Autry, Director DHEC