

June 19, 1995

Mr. George A. Hunger,  
Director-Licensing, MC 62A-1  
PECO Energy Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, PA 19087-0195

SUBJECT: LOCAL LEAK RATE TEST HOLD TIME REQUIREMENTS, PEACH BOTTOM ATOMIC  
POWER STATION, UNIT NOS. 2 AND 3 (TAC NOS. M91870 AND M91871)

Dear Mr. Hunger:

The Commission has issued the enclosed Amendments Nos. 207 and 209 to Facility Operating License Nos. DPR-44 and DPR-56 for the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 22, 1995.

These amendments reduce the local leak rate test hold time specified in the TS Tables 3.7.2 through 3.7.4 from one hour to 20 minutes.

You are requested to inform the staff when you have implemented the provisions of these amendments. The requirement affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. 96-511.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely, /s/

Joseph W. Shea, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-277/50-278

Enclosures:

- 1. Amendment No. 207 to DPR-44
- 2. Amendment No. 209 to DPR-56
- 3. Safety Evaluation

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See next page PDR ADOCK 05000277  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 19, 1995

Mr. George A. Hunger, Jr.  
Director-Licensing, MC 62A-1  
PECO Energy Company  
Nuclear Group Headquarters  
Correspondence Control Desk  
P.O. Box No. 195  
Wayne, PA 19087-0195

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Sincerely,

A handwritten signature in black ink, appearing to read "Joseph W. Shea".

Joseph W. Shea, Project Manager  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-277/50-278

Enclosures:

1. Amendment No. 207 to DPR-44
2. Amendment No. 209 to DPR-56
3. Safety Evaluation

cc w/encls:  
See next page

**Mr. George A. Hunger, Jr.**  
**PECO Energy Company**

**Peach Bottom Atomic Power Station,**  
**Units 2 and 3**

cc:

**J. W. Durham, Sr., Esquire**  
**Sr. V.P. & General Counsel**  
**PECO Energy Company**  
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**Division of Nuclear Safety**  
**Pennsylvania Department of**  
**Environmental Resources**  
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**Harrisburg, Pennsylvania 17105-8469**

**PECO Energy Company**  
**ATTN: Mr. G. R. Rainey, Vice President**  
**Peach Bottom Atomic Power Station**  
**Route 1, Box 208**  
**Delta, Pennsylvania 17314**

**Board of Supervisors**  
**Peach Bottom Township**  
**R. D. #1**  
**Delta, Pennsylvania 17314**

**PECO Energy Company**  
**ATTN: Regulatory Engineer, A4-5S**  
**Peach Bottom Atomic Power Station**  
**Route 1, Box 208**  
**Delta, Pennsylvania 17314**

**Public Service Commission of Maryland**  
**Engineering Division**  
**Chief Engineer**  
**6 St. Paul Centre**  
**Baltimore, MD 21202-6806**

**Resident Inspector**  
**U.S. Nuclear Regulatory Commission**  
**Peach Bottom Atomic Power Station**  
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**Delta, Pennsylvania 17314**

**Mr. Richard McLean**  
**Power Plant and Environmental**  
**Review Division**  
**Department of Natural Resources**  
**B-3, Tawes State Office Building**  
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**Regional Administrator, Region I**  
**U.S. Nuclear Regulatory Commission**  
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**King of Prussia, Pennsylvania 19406**

**Mr. John Doering, Chairman**  
**Nuclear Review Board**  
**PECO Energy Company**  
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**Mail Code 63C-5**  
**Wayne, Pennsylvania 19087**

**Mr. Roland Fletcher**  
**Department of Environment**  
**201 West Preston Street**  
**Baltimore, Maryland 21201**

**Dr. Judith Johnsrud**  
**National Energy Committee**  
**Sierra Club**  
**433 Orlando Avenue**  
**State College, PA 16803**

**A. F. Kirby, III**  
**External Operations - Nuclear**  
**Delmarva Power & Light Company**  
**P.O. Box 231**  
**Wilmington, DE 19899**



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PECO ENERGY COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 207  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by PECO Energy Company, et al. (the licensee) dated March 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 207, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Donald A. Brinkman*  
for John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 19, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 207

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove ,

188

Insert

188

## PBAPS

NOTES FOR TABLES 3.7.2 THROUGH 3.7.4

- (1) Minimum test hold time duration for all valves and penetrations listed is 20 minutes after stabilization.
- (2) Test pressures of at least 49.1 psig for all valves and penetrations except MSIV's which are tested at 25 psig.
- (3) MSIV's acceptable leakage is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the MSIV's is 0.60 La.
- (5) Local leak tests on all testable isolation valves shall be performed per 10CFR50, Appendix J requirements.
- (6) Local leak tests on all testable penetrations shall be performed per 10CFR50, Appendix J requirements.
- (7) Personnel Air Locks shall be tested at 6-month intervals.
- (8) The personnel air locks are tested at 49.1 psig.
- (9) Identifies isolation valves that may be tested by applying pressure between the inboard and outboard valves.
- (10) Gate valves are tested in reverse direction. Test acceptable since the normal force between the seat and the disc generated by stem action alone is greater than ten (10) times the normal force induced by test differential pressure except for valves MO-10-31A,B which is 7.97. This applies to the following valves:

MO-2-74	MO-10-31A, B
MO-13-15	MO-10-18
MO-23-15	MO-12-15 (Unit #2)
MO-10-32 (Unit #2)	



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

PECO ENERGY COMPANY

PUBLIC SERVICE ELECTRIC AND GAS COMPANY

DELMARVA POWER AND LIGHT COMPANY

ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209  
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by PECO Energy Company, et al. (the licensee) dated March 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 209, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Donald A. Bierkner*  
for John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 19, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 209

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove

188

Insert

188

## PBAPS

NOTES FOR TABLES 3.7.2 THROUGH 3.7.4

- (1) Minimum test hold time duration for all valves and penetrations listed is 20 minutes after stabilization.
- (2) Test pressures of at least 49.1 psig for all valves and penetrations except MSIV's which are tested at 25 psig.
- (3) MSIV's acceptable leakage is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the MSIV's is 0.60 La.
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MO-2-74	MO-10-31A, B
MO-13-15	MO-10-18
MO-23-15	MO-12-15 (Unit #2)
MO-10-32 (Unit #2)	



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 207 AND 209 TO FACILITY OPERATING

LICENSE NOS. DPR-44 and DPR-56

PECO ENERGY COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

PEACH BOTTOM ATOMIC POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By letter dated March 22, 1995, PECO Energy Company (the licensee) submitted a request for changes to the Peach Bottom Atomic Power Station, Unit Nos. 2 and 3, Technical Specifications (TS). The requested changes would reduce the local leak rate test (LLRT) hold time specified in the TS Tables 3.7.2 through 3.7.4 from one hour to 20 minutes.

2.0 EVALUATION

Section 4.7 of the existing Peach Bottom TS specifies the requirements for Primary Containment, including equipment operability and surveillance requirements. TS 4.7.A.2.f states:

Local leak rate tests (LLRT's) shall be performed on the primary containment testable penetrations and isolation valves in accordance with Tables 3.7.2, 3.7.3 & 3.7.4 at a pressure of 49.1 psig (except for the main steam isolation valves, see below) per 10 CFR 50 Appendix J requirements.

Tables 3.7.2, 3.7.3 and 3.7.4 list individually all of the primary containment testable penetrations and testable isolation valves. Footnotes for these tables provide additional detail on the testing requirements for these penetrations. Note 1 (p188 of the TS) states that the minimum test duration for all valves and penetrations listed is one hour.

In the March 22, 1995 application, the licensee proposed to revise the footnote to read:

Minimum test hold time duration for all valves and penetrations listed is 20 minutes after stabilization.

Local leak rate tests are typically conducted by isolating the affected component and exposing the component to a test source of pressure. The test pressure is applied to the component and component leakage is determined either by pressure decay methods or flow rate methods. For pressure decay methods, the test pressure source is isolated from the component after initial pressurization and the rate at which pressure decays is measured over a period of time (referred to as the hold time). The rate of pressure decay is used to determine the leakage rate of the component. In the second method, the component remains connected to a test source for the duration of the test. Flow into the component is assumed to compensate for component leakage. During the hold time, the actual flow rate into the component is measured with flow measuring instrumentation which thus allows direct quantification of the leak rate.

Both test methods allow a period of time after initial pressurization prior to commencement of the hold time to allow for stabilization. This period allows for transient dynamic and temperature effects to stabilize, thus allowing the measurements obtained during the hold time to be representative of the leak tightness of the component.

The licensee stated that, based on a review of previous Peach Bottom LLRT test data, it concluded that no significant changes in leakage rates occur during the one hour hold time after leakage stabilization is achieved. Therefore, PECO concluded that decreasing the hold time from one hour to 20 minutes does not adversely affect the results of the LLRT.

The licensee noted that decreasing the LLRT hold time will decrease the occupational dose received by the individuals performing the test.

Appendix J to 10 CFR 50 specifies primary containment leak testing requirements for water cooled reactors. Appendix J does not specify local leak rate test methodologies to a level of detail that addresses hold times and stabilization. However, Section 6.4.3 of ANSI/ANS 56.8-1994, "Containment System Leakage Testing Requirements," states:

Pressure decay tests should have a test duration of not less than 15 minutes after stable conditions have been attained. A flow makeup test should be used if a pressure decay test cannot be performed for at least 15 minutes. No minimum test duration shall be required for a make-up flow test; however, test data shall be obtained during stable conditions.

The staff reviewed the licensee's proposed changes. The staff has previously reviewed the LLRT guidance in Section 6.4.3 of ANSI/ANS 56.8-1994 and finds this guidance acceptable. The staff concludes that the proposed changes are more conservative than the guidance in Section 6.4.3 of ANSI/ANS-56.8-1994. Thus the staff finds that the proposed changes to the Peach Bottom TS will not

adversely affect the results of LLRTs at Peach Bottom and are therefore, acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 24913). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Shea

Date: June 19, 1995